

# Applications of Nuclear Data: Introduction to FISPACT-II

ICTP-IAEA Workshop on the Evaluation of Nuclear Data for Applications

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International Centre  
for Theoretical Physics  
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# Agenda

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# Login

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We will be using the Linux Mint 18 operating system installed on these machines. If you are not familiar with non-Windows machines don't worry

**Log into the machine now using your username/password on your badge**

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We will be using the FISPACT-II code in this workshop, which is located on a shared space

/home/nfs1/smr3151/FISPACT-II-3-20

The nuclear data must be transferred to your local machine so please do the following:

- ▶ Check that your scratch space is not full:

```
machine:~ user$ df -h /* | grep scratch
```

- ▶ If the scratch space has less than 50 GB remaining *let me know* and we'll clear some space



# Installation

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To move the essential FISPACT-II code components onto your local scratch space, run the custom installation script provided in:

```
source /home/nfs1/smr3151/scripts/install_fispact.sh
Copying getting_started directory to: /scratch/
...DONE
Copying nuclear data to: /scratch/
...DONE
Unpacking nuclear data
```

This will install the nuclear data onto your local  
/scratch/smr3151/FISPACT-II-3-20/

It will also add the `fispact` executable to your `PATH` and the  
`getting_started` directory to your scratch space

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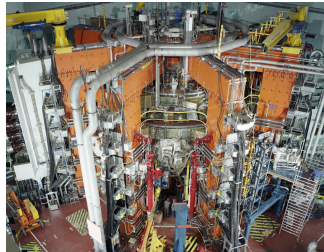
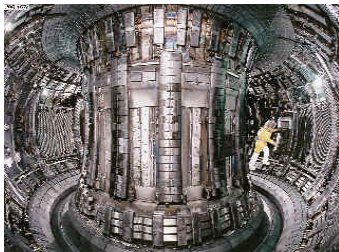
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- ▶ The UK Atomic Energy Authority (UKAEA) includes the Culham Centre for Fusion Energy (CCFE), which hosts (amongst other things) the Joint European Torus (JET):



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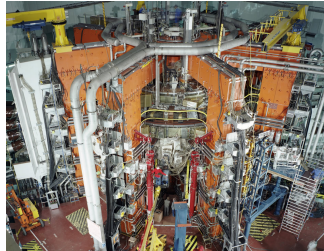
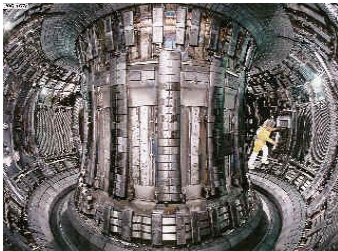
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- ▶ The UK Atomic Energy Authority (UKAEA) includes the Culham Centre for Fusion Energy (CCFE), which hosts (amongst other things) the Joint European Torus (JET):



- ▶ Fusion utilises the D-T plasma which, as was described in a previous lecture, is an excellent source of 14 MeV neutrons



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- The UKAEA developed FISPIN as an inventory code for fission fuel modelling

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- ▶ The UKAEA developed FISPIN as an inventory code for fission fuel modelling
- ▶ In the 80s and 90s it was appreciated that more sophisticated nuclear simulation codes and nuclear data was required
  - ▶ Birth of FISPACT code + EAF data = EASY





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- ▶ The UKAEA developed FISPIN as an inventory code for fission fuel modelling
- ▶ In the 80s and 90s it was appreciated that more sophisticated nuclear simulation codes and nuclear data was required
  - ▶ Birth of FISPACT code + EAF data = EASY
- ▶ Unsupportable growth of 'physicist' code and need to accommodate next-generation TENDL data led to re-development as FISPACT-II in 2009



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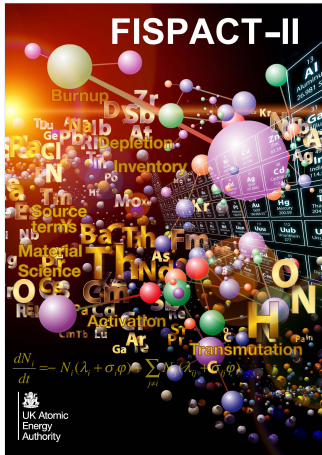
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- ▶ The UKAEA developed FISPIN as an inventory code for fission fuel modelling
- ▶ In the 80s and 90s it was appreciated that more sophisticated nuclear simulation codes and nuclear data was required
  - ▶ Birth of FISPACT code + EAF data = EASY
- ▶ Unsupportable growth of 'physicist' code and need to accommodate next-generation TENDL data led to re-development as FISPACT-II in 2009
- ▶ FISPACT-II now released through OECD-NEA Data Bank, ORNL RSICC and UKAEA - deprecated EASY not supported nor recommended



- ▶ multiphysics platform for predicting the inventory changes in materials under both neutron and charged-particle interactions
  - ▶ calculates activation, transmutation, burn-up, dpa, PKAs, gas production, etc.
- ▶ employs the most up-to-date international nuclear data libraries containing:
  - ▶ nuclear reaction data (reaction cross sections)
  - ▶ radioactive decay data (half-lives and decay schemes)
  - ▶ fission yield data (ratios)

$$\frac{dN_i}{dt} = \underbrace{-N_i(\lambda_i + \sigma_i\phi)}_{\text{loss}} + \sum_{j \neq i} \underbrace{N_j(\lambda_{ji} + \sigma_{ji}\phi)}_{\text{creation}}$$

- coupled differential equations
  - one equation for each nuclide  $i$  at concentration  $N_i$
  - solved numerically by FISPACT-II (using Livermore ODE solver, LSODE) and used to update material composition

$$\frac{dN_i}{dt} = \underbrace{-N_i(\lambda_i + \sigma_i\phi)}_{\text{loss}} + \sum_{j \neq i} \underbrace{N_j(\lambda_{ji} + \sigma_{ji}\phi)}_{\text{creation}}$$

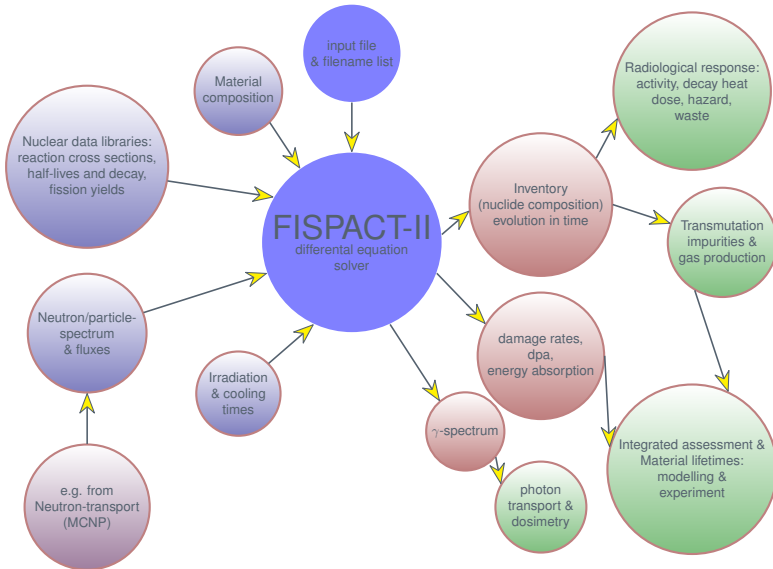
- ▶ coupled differential equations
  - ▶ one equation for each nuclide  $i$  at concentration  $N_i$
  - ▶ solved numerically by FISPACT-II (using Livermore ODE solver, LSODE) and used to update material composition
- ▶  $\sigma_{ji}$ : energy-dependent reaction cross sections for  $j \rightarrow i$  reactions (e.g.  $(n,\gamma)$ ,  $(n,\alpha)$ ,  $(n,2n)$ , etc.) from nuclear libraries collapsed with (normalised) neutron energy spectra from neutron transport;  $\sigma_i$  is sum over all  $i \rightarrow j$  reactions

$$\frac{dN_i}{dt} = \underbrace{-N_i(\lambda_i + \sigma_i\phi)}_{\text{loss}} + \sum_{j \neq i} \underbrace{N_j(\lambda_{ji} + \sigma_{ji}\phi)}_{\text{creation}}$$

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- ▶ decay constants  $\lambda_i$ ,  $\lambda_{ji}$  (from decay library of measurements)

$$\frac{dN_i}{dt} = \underbrace{-N_i(\lambda_i + \sigma_i\phi)}_{\text{loss}} + \sum_{j \neq i} \underbrace{N_j(\lambda_{ji} + \sigma_{ji}\phi)}_{\text{creation}}$$

- ▶ coupled differential equations
  - ▶ one equation for each nuclide  $i$  at concentration  $N_i$
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- ▶ decay constants  $\lambda_i$ ,  $\lambda_{ji}$  (from decay library of measurements)
- ▶ total fluxes  $\phi$  from radiation transport simulations, experiments, operational scenarios, etc.







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## Radiological outputs

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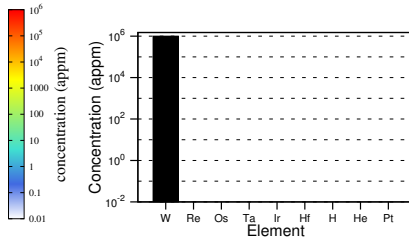
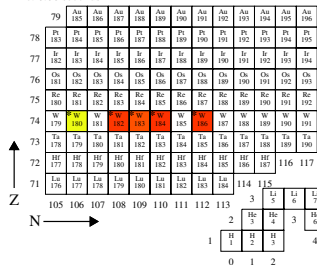
Keywords

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- ▶ Activity measured in becquerels (Bq) – number of disintegrations (decays) per second – the primary measure
  - ▶ can be separated by decay type –  $\alpha$ ,  $\beta$ ,  $\gamma$  – in FISPACT-II output
- ▶ decay heat, measured in kilowatts (kw)
  - ▶ can be separated by decay type -  $\alpha$ ,  $\beta$ ,  $\gamma$
  - ▶ how much heat will be generated in a material even when not exposed to irradiation
  - ▶ critical to determine whether cooling is needed to prevent melting
- ▶ (contact)  $\gamma$  dose rate, measured in sieverts (Sv) per hour
  - ▶  $\text{J kg}^{-1}$  deposition rate of radiation energy in biological tissue
  - ▶ there are also ingestion and inhalation hazard versions
- ▶ clearance index
  - ▶ IAEA based measure
  - ▶ a nuclide can be disposed of as if it were non radioactive when the index is less than 1

## Tungsten irradiation in fusion reactor environment

Time: 0.00 seconds





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## Example results

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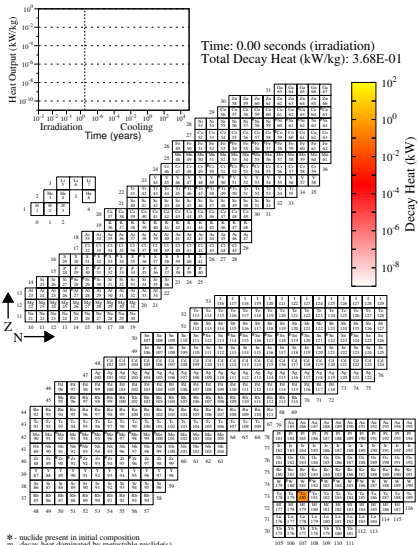
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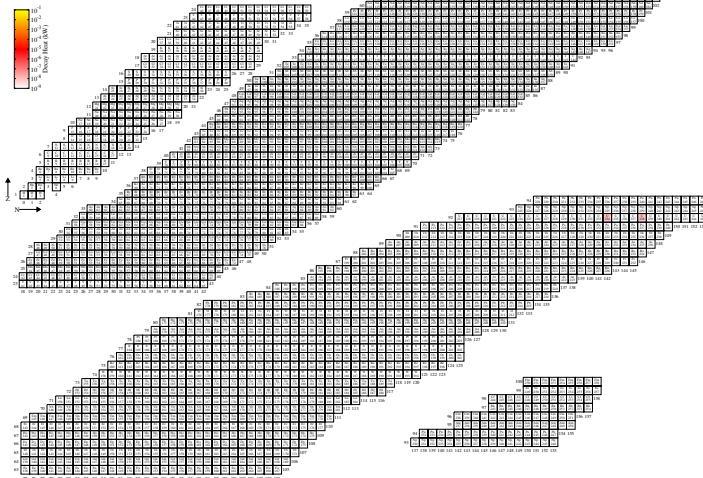
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## U235 thermal fission pulse decay heat

Time: 0.00 seconds

Total Decay Heat (kW/kg): 1.838E-08





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## Example results

### U235 thermal fission pulse decay heat

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- ▶ FISPACT-II is run through terminal commands, using input files and generating output files with simulation results
  - ▶ Recommended use of Linux or OS X systems
  - ▶ Windows users have options (Cygwin, VirtualBox, etc.) and we have developed a new container solution for next release





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- ▶ The input files are precisely curated sets of **keywords** with keyword options



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EXERCISES

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  - ▶ Recommended use of Linux or OS X systems
  - ▶ Windows users have options (Cygwin, VirtualBox, etc.) and we have developed a new container solution for next release
- ▶ The input files are precisely curated sets of **keywords** with keyword options
- ▶ Use of different keyword and option combinations may result in various types of simulations





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- FISPACT-II may be run in many ways, but the most common execution process takes four stages:

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- ▶ FISPACT-II may be run in many ways, but the most common execution process takes four stages:
- ▶ Handling the nuclear data libraries and incident spectra
  - ▶ Condense fission yields and decay data
  - ▶ Collapse (fold) cross-sections with incident spectra
  - ▶ Print summary of library data (optional, but often useful)



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- ▶ Handling the nuclear data libraries and incident spectra
  - ▶ Condense fission yields and decay data
  - ▶ Collapse (fold) cross-sections with incident spectra
  - ▶ Print summary of library data (optional, but often useful)
- ▶ Setting initial conditions (material composition, etc.) & select options (output format, uncertainty quantification)



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- ▶ FISPACT-II may be run in many ways, but the most common execution process takes four stages:
- ▶ Handling the nuclear data libraries and incident spectra
  - ▶ Condense fission yields and decay data
  - ▶ Collapse (fold) cross-sections with incident spectra
  - ▶ Print summary of library data (optional, but often useful)
- ▶ Setting initial conditions (material composition, etc.) & select options (output format, uncertainty quantification)
- ▶ Simulate irradiation phases, subsidiary calculations and output data



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- ▶ FISPACT-II may be run in many ways, but the most common execution process takes four stages:
- ▶ Handling the nuclear data libraries and incident spectra
  - ▶ Condense fission yields and decay data
  - ▶ Collapse (fold) cross-sections with incident spectra
  - ▶ Print summary of library data (optional, but often useful)
- ▶ Setting initial conditions (material composition, etc.) & select options (output format, uncertainty quantification)
- ▶ Simulate irradiation phases, subsidiary calculations and output data
- ▶ Simulate cooling phases and output summary data



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## Basic execution

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- FISPACT-II is run through terminal commands:

```
machine:~ user$ fispact input files
```

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- ▶ FISPACT-II is run through terminal commands:

```
machine:~ user$ fispact input files
```

- ▶ If you are not familiar with using a terminal, **don't worry**, but please let me know so you can get the most out of this workshop



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## Basic execution

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- ▶ FISPACT-II is run through terminal commands:

```
machine:~ user$ fispact input files
```

- ▶ If you are not familiar with using a terminal, **don't worry**, but please let me know so you can get the most out of this workshop
- ▶ The code requires two files:
  - ▶ a simulation specification file `input.i`,



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- ▶ FISPACT-II is run through terminal commands:

```
machine:~ user$ fispact input files
```

- ▶ If you are not familiar with using a terminal, **don't worry**, but please let me know so you can get the most out of this workshop
- ▶ The code requires two files:
  - ▶ a simulation specification file `input.i`,
  - ▶ an amusing named `files` file that lists all input/output files

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## Anatomy of the `files` file

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- The `files` file contains the mapping of all library files that FISPACT-II will use in the calculation. Every required data must be listed – otherwise a fatal error will be issued.

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## Anatomy of the `files` file

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- ▶ The `files` file contains the mapping of all library files that FISPACT-II will use in the calculation. Every required data must be listed – otherwise a fatal error will be issued.
- ▶ These are listed in Tables 1-3 of the User Manual Section 3.1 pages 23-24

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## Anatomy of the files file

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- ▶ The files file contains the mapping of all library files that FISPACT-II will use in the calculation. Every required data must be listed – otherwise a fatal error will be issued.
- ▶ These are listed in Tables 1-3 of the User Manual Section 3.1 pages 23-24
- ▶ Comments are given by #, an example:

```
# Cross section data from TENDL-2015
xs_endf /path/to/tendl-2015/neutron/709-data
# My input spectra
fluxes /my/working/directory/my_spectra
# Decay data from ENDF/B-VII.1
dk_endf /path/to/endfb7.1/decay
```

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## Anatomy of the files file

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fluxes /my/working/directory/my_spectra
# Decay data from ENDF/B-VII.1
dk_endf /path/to/endfb7.1/decay
```

- ▶ There are up to 20 unique types of data, use the example inputs as guidance and/or the User Manual

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$$\frac{dN_i}{dt} = \underbrace{-N_i(\lambda_i + \sigma_i\phi)}_{\text{loss}} + \sum_{j \neq i} \underbrace{N_j(\lambda_{ji} + \sigma_{ji}\phi)}_{\text{creation}}$$

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$$\frac{dN_i}{dt} = \underbrace{-N_i(\lambda_i)}_{\text{loss}} + \sum_{j \neq i} \underbrace{N_j(\lambda_{ji})}_{\text{creation}}$$

- ▶ decay constants  $\lambda_i, \lambda_{ji}$  ( $\text{s}^{-1}$ )
- ▶ **GETDECAY** to read-in from pre-prepared **ARRAYX** file
  - ▶ or to create **ARRAYX**

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$$\frac{dN_i}{dt} = \underbrace{-N_i(\sigma_i\phi)}_{\text{loss}} + \sum_{j \neq i} \underbrace{N_j(\sigma_{ji}\phi)}_{\text{creation}}$$

- ▶ (neutron) fluxes  $\phi$  and energy dependent spectra in neutrons  $\text{cm}^{-2}\text{s}^{-1}$
- ▶ **GETXS** to collapse (fold) **FLUXES** file with reaction data to produce **COLLAPX** file of  $\sigma_i, \sigma_{ji}$  values (or read from it)
- ▶ **FLUX** to specify total flux  $\phi$



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## Nuclear data collapse

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- Users provide an incident particle spectrum that is used to generate '1-group' or effective cross sections

$$\bar{\sigma}_r = \sum_i \phi_i \sigma_{i,r}$$



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## Nuclear data collapse

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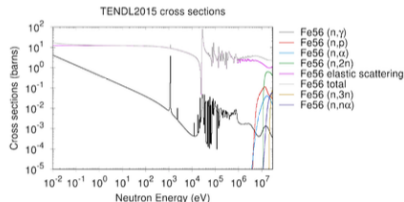
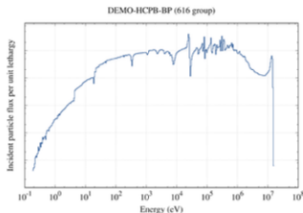
EXERCISES

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- Users provide an incident particle spectrum that is used to generate ‘1-group’ or effective cross sections

$$\bar{\sigma}_r = \sum_i \phi_i \sigma_{i,r}$$

- Requires `fluxes` input and data files: `ind_nuc` (nuclide indices), `xs_endf` (ENDF reaction data) and `prob_tab` probability tables (for self-shielding)



Fe 56 (n, a ) Cr 53 9.30007E-03+- 1.87183E+02  
 Fe 56 (n, 2p) Cr 55 1.89736E-12+- 0.00000E+00  
 Fe 56 (n, np) Mn 55 1.09412E-02+- 1.71659E+01  
 Fe 56 (n, p ) Mn 56 3.09369E-02+- 6.14934E+00  
 Fe 56 (n, E ) Fe 56 3.39432E+00+- 8.08801E-01  
 Fe 56 (n, g ) Fe 57 1.11989E-02+- 3.18853E+00

Fe 56 (n, h ) Cr 54 1.43520E-11+- 5.52180E+02  
 Fe 56 (n, t ) Mn 54 2.71346E-08+- 4.06438E+02  
 Fe 56 (n, d ) Mn 55 9.78951E-04+- 2.52961E+02  
 Fe 56 (n, 2n) Fe 55 1.10279E-01+- 1.43515E+01  
 Fe 56 (n, n ) Fe 56 3.59211E-01+- 1.13654E+01



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Condense process

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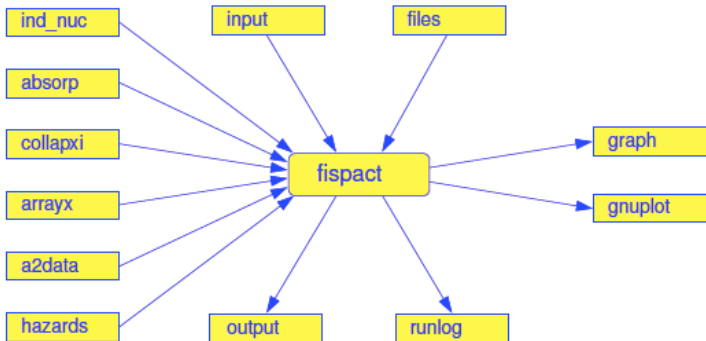
- The 'condense' of the decay data is the reading of all ENDF-6 data files into a set of data for inventory simulations ( $\lambda$  for each nuclide with branching ratios and spectral information)

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- ▶ The 'condense' of the decay data is the reading of all ENDF-6 data files into a set of data for inventory simulations ( $\lambda$  for each nuclide with branching ratios and spectral information)
- ▶ The fission yield reading also weights the energy-dependent yields:

$$Y(Z, A, I) = \frac{\sum_i Y_i(Z, A, I) \phi_i \sigma_{mt=18,i}}{\sum_i \phi_i \sigma_{mt=18,i}}$$



- The inventory simulation requires the transcribed cross section, decay and yield data along with user-supplied irradiation/cooling/response information





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## Inventory schematic

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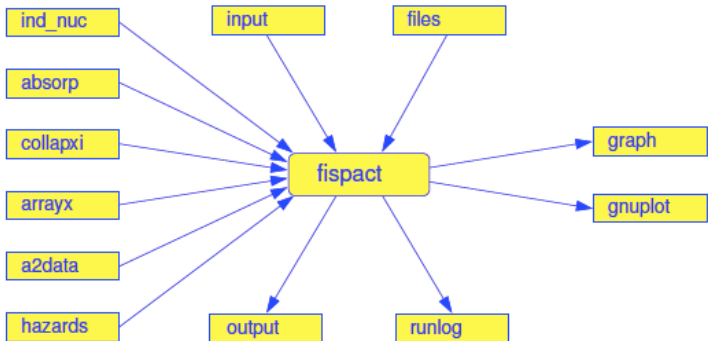
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- ▶ The inventory simulation requires the transcribed cross section, decay and yield data along with user-supplied irradiation/cooling/response information
- ▶ Variety of outputs, graph data, tabulated, etc.



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## Anatomy of input files

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EXERCISES

- ▶ The FISPACT-II inputs use keywords in specific order - use the examples for direction (<...> are comments):

```
< -- Control phase -- >
GETXS option_1 option_2
GETDECAY option_3
FISPACT
* Description of calculation
< -- Initial phase -- >
< material definition, simulation options, etc. >
...
< -- Inventory phase -- >
< irradiation definitions, cooling, outputs >
...
```

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## Keyword options

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EXERCISES

- The most recent distribution (Dec 2016 v 3-20) that is available through the NEA Data Bank, RSICC, etc., has some 100 keywords - see the manual or wiki:

[https://fispact.ukaea.uk/wiki/FISPACT-II\\_keywords](https://fispact.ukaea.uk/wiki/FISPACT-II_keywords)



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## Keyword options

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[https://fispact.ukaea.uk/wiki/FISPACT-II\\_keywords](https://fispact.ukaea.uk/wiki/FISPACT-II_keywords)
- ▶ Options are also well documented, for example  
  
GETXS option\_1 option\_2:

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## Keyword options

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- ▶ Options are also well documented, for example

GETXS option\_1 option\_2:

- ▶ option\_1 = -1 read binary group data
- ▶ option\_1 = 0 read binary collapsed data
- ▶ option\_1 = 1 read text group data with option\_2 group

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## FISPACT-II keywords

A run of FISPACT-II is controlled by a sequence of commands given in a user-supplied input file, the anatomy of which is described in the [code execution page](#). This requires a series of keywords to be specified for control, initial and inventory stages. Below are all of the keywords available in the most recent release of FISPACT-II, sorted by their phase and with a short description. Pages for each of the keywords are linked from this table which provide details of how to use each. Please note that as the code develops new keywords are added to allow access to new features while retaining backwards compatibility. For clarity, keywords added since the 3-00-00 distribution are identified with the scheme below:

Previous release 3-00-00    Current supported release 3-20-00

FISPACT-II Keywords					
Keyword	Version	Control	Initial	Invent.	Description
ALLDISPEN	3-20-00	✓			Sets the displacement energies (in eV) for all nuclides
ATDISPEN	3-20-00	✓			Sets the displacement energies (in eV) for specified elements
ATOMS	3-00-00		✓	✓	Sets initial conditions and initiates output with inventories and observables. After the <a href="#">ZERO</a> keyword will also output uncertainties
ATWO	3-00-00		✓		Causes output of legal limits for activity in transport of radioactive material
BREMSSTRAHLUNG	3-00-00		✓		Causes output of bremsstrahlung contributions for specified nuclides
CLEAR	3-00-00		✓		Causes clearance data of radionuclides to be output
CLOBBER	3-00-00	✓			Allows FISPACT-II to overwrite existing output with same name
CNVTYPE	3-20-00	✓			Allows the user to specify formalism for conversion of incident particle spectra
COVARIANCE	3-00-00	✓			Causes cross-channel covariances to be calculated (if present in nuclear data)
CULTAB	3-00-00		✓		Produces additional lines in tab files for specific post-processing tools
CUMFYLD	3-00-00	✓			Allows the cumulative fission yields to be read, rather than the default independent yields

## Keyword: GETXS

### GETXS *libxs* <*ebins*>

This keyword has two integer parameters. If the first parameter *libxs* is set to 0, then the second parameter should be omitted, and cross section data are read from the existing collapsed library (collapx file) specified in the *files* file. If *libxs* is 1, then the second parameter *ebins* gives the number of energy bins to be used in collapsing the cross section data from the ENDF library files and fluxes or *arb\_flux* files specified in the *files* file. If *libxs* is -1, then the ENDF data are read from the compressed binary version of the ENDF data stored in the file specified by *xs\_endfb* in the *files* file. Note that the value *libxs* = -1 is not valid for legacy EAF libraries. For information on the preparation of the compressed binary ENDF data files see [this page](#).

The **GETXS** keyword may also be used in the initial conditions and inventory calculation phases to modify the cross sections, for example to handle time-dependent projectile spectra and temperature changes in cross-sections.

The number of energy groups *ebins* must be consistent with the number of groups in the supplied library file. The permitted numbers of groups for cross-section data are, for the most recent public distribution, the [CCFE-709](#) for neutrons and [CCFE-162](#) for charged particles. Legacy EAF libraries and their various group structures may also be used.

Example usage, where the cross sections are re-collapsed every 20 days during some irradiation simulation:

```
< -- Control phase -- >
...
GETXS 1 709
FISPACT
```



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## Most common keywords

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Some can be used multiple times, for example re-collapse data in burn- up with spectral shift or change the flux/power normalisation – See the wiki for all details or the User Manual

FISPACT	Required to start simulation
GETXS/GETDECAY	Required to process nuclear data
END	Required to end input
TIME	Used to specify time intervals
ATOMS/STEP	Used to instruct the code to solve
HAZARD/DOSE	Used to request standard outputs
MASS/FUEL	Used to specify input material
USEFISSION	Used to turn on fission
FLUX/POWER	Used to set flux normalisation
PRINTLIB	Used to print nuclear data
ZERO	Used to zero the clock for cooling
SSFCHOOSE/	Used to for self-shielding
SSFMAS/SSFFUEL	





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- ▶ FISPACT-II comes with a `getting_started` tutorial suite that we will use in this session
- ▶ Steps:

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- ▶ FISPACT-II comes with a `getting_started` tutorial suite that we will use in this session
- ▶ Steps:
  - ▶ Open a terminal in Linux Mint and navigate to `/getting_started/FNS_Inconel`

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  - ▶ Open the user manual (online at <http://fispact.ukaea.uk>) to section 3
  - ▶ Complete the first exercises and ensure that you perform the pulsed decay heat example
- ▶ If you have already attended the NEA training course **let me know** and I'll provide more challenging examples

Lecture break for exercises



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