

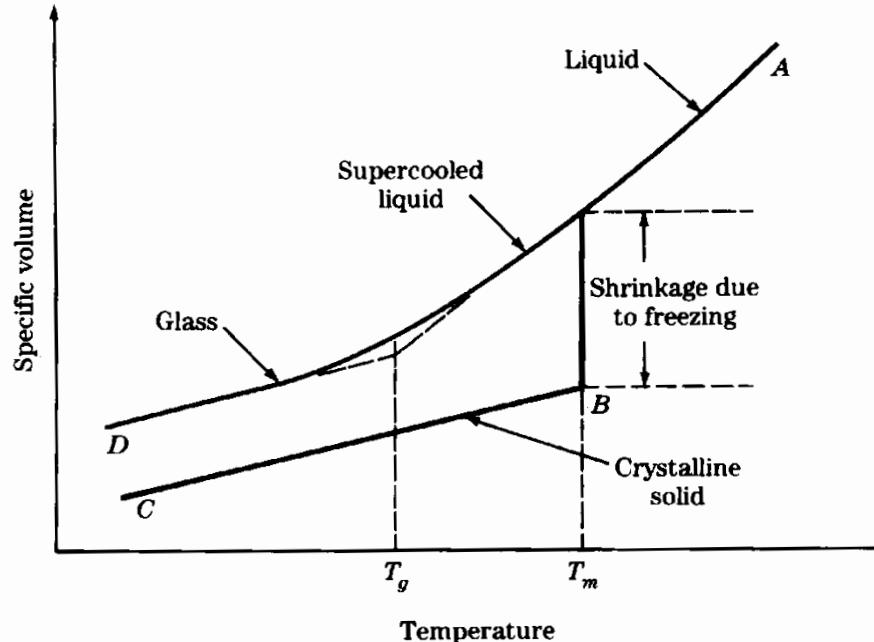
DE LA RECHERCHE À L'INDUSTRIE



www.cea.fr

Radiation damages in vitreous wasteforms

Glassy state

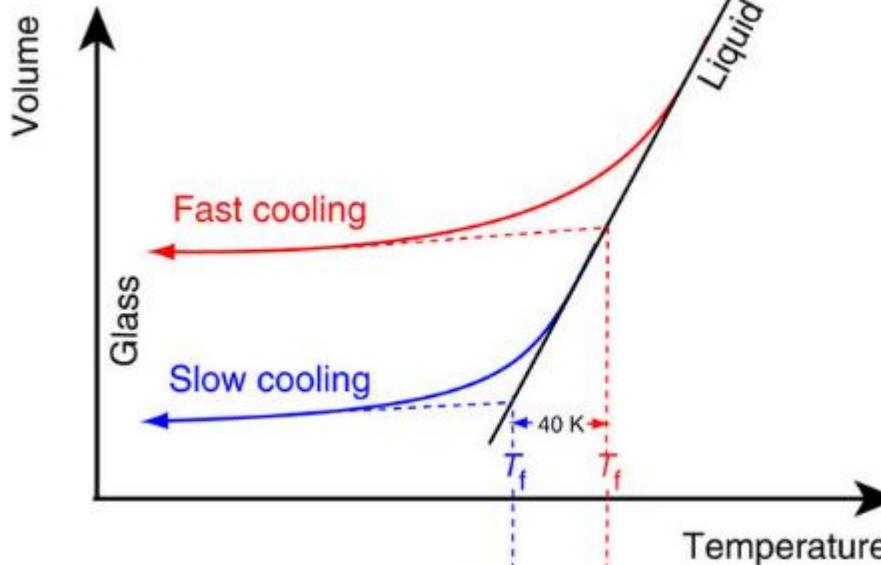


A glass (or vitreous solid) is a solid formed by rapid melt quenching.

A glass is an amorphous solid that exhibits a glass transition phenomena at T_g .

$$\text{Relaxation time } \tau = \frac{\eta}{G}$$

Viscosity Shear Modulus



Glass properties depend on:

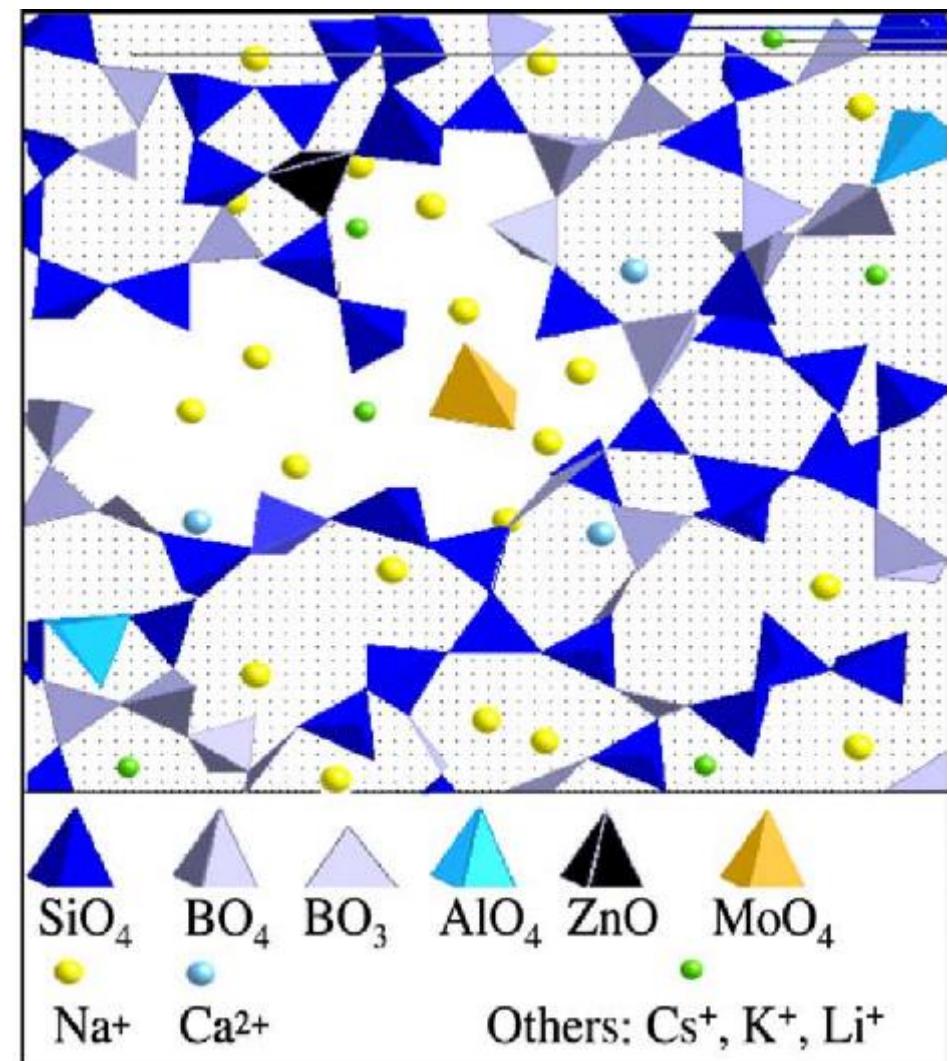
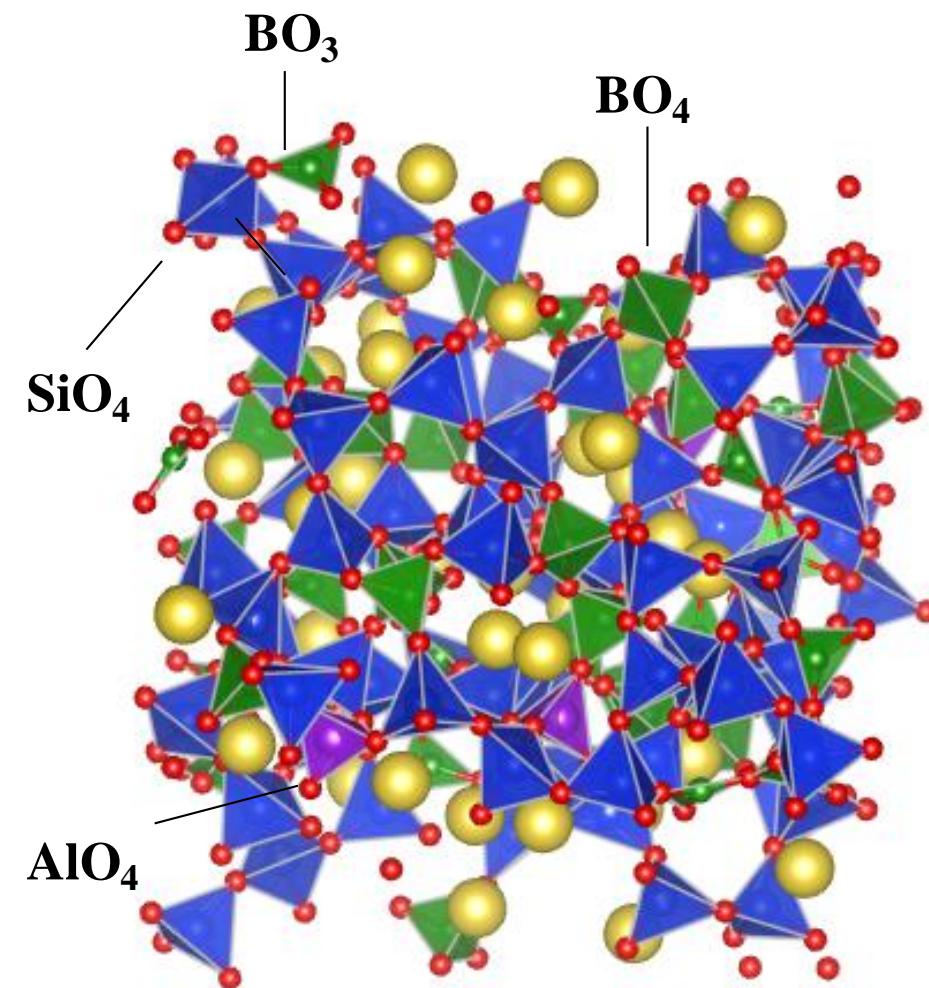
- **Chemical composition**
- **Thermal history during elaboration process**

Glassy state

- Short Range Order,
- Medium Range Order,
- Long Range Order,

SRO : Yes
MRO : Yes
LRO : No

Polyhedra
Angle, Ring statistic

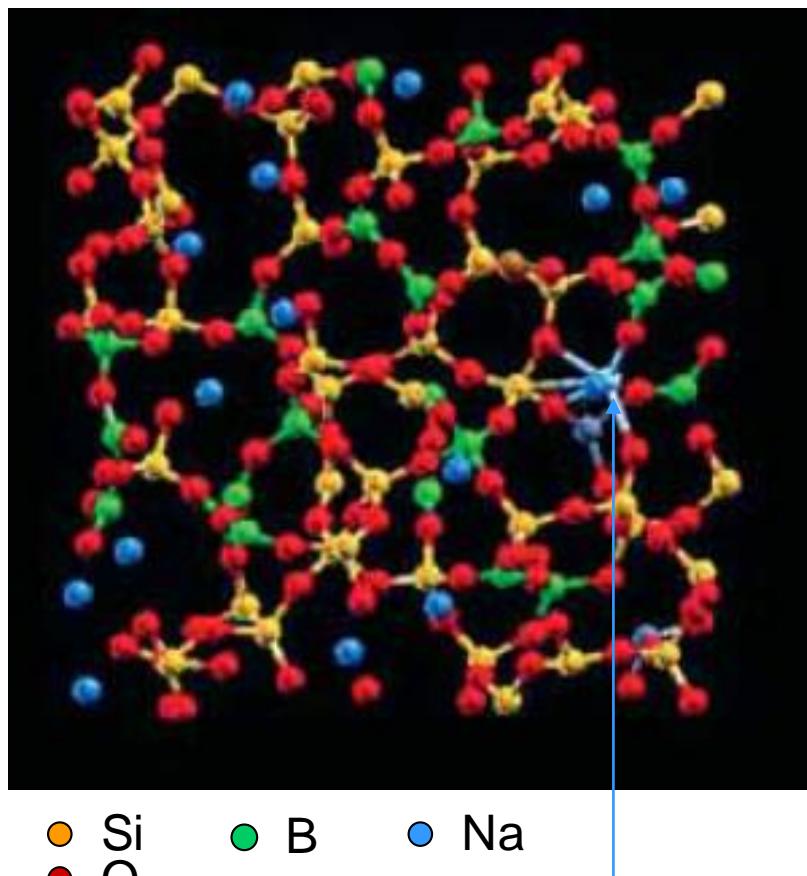


Complex oxides glasses : French Nuclear Glass

Oxide glass with around 30 oxides

Sodium alumino-borosilicate glass

L. Cormier, J.M. Delaye, D. Ghaleb, G. Calas, PRB 61 (2001) 14495



Domaine de composition chimique des verres R7T7 produits dans les ateliers industriels par AREVA - La Hague

Oxydes	Intervalle spécifié pour l'industriel (% massique)		Composition moyenne des verres industriels (% massique)
	min	max	
SiO_2	42,4	51,7	45,6
B_2O_3	12,4	16,5	14,1
Al_2O_3	3,6	6,6	4,7
Na_2O	8,1	11,0	9,9
CaO	3,5	4,8	4,0
Fe_2O_3	< 4,5		1,1
NiO	< 0,5		0,1
Cr_2O_3	< 0,6		0,1
P_2O_5	< 1,0		0,2
Li_2O	1,6	2,4	2,0
ZnO	2,2	2,8	2,5
Ox (PF+Zr+ actinides)+ Suspension de fines	7,5	18,5	17,0
Oxydes d'actinides			0,6
$\text{SiO}_2+\text{B}_2\text{O}_3+\text{Al}_2\text{O}_3$	> 60	64,4	

Monographie DEN : Le conditionnement des déchets nucléaires

Nuclear Glass or GCM: What type of radiation?

Decay Type	Radiation Emitted	Generic Equation	Model
Alpha decay	${}_{Z}^{A}\alpha$	${}_{Z}^{A}X \longrightarrow {}_{Z-2}^{A-4}X' + {}_{2}^{4}\alpha$	 Parent → Daughter Alpha Particle
Beta decay	${}_{-1}^{0}\beta$	${}_{Z}^{A}X \longrightarrow {}_{Z+1}^{A}X' + {}_{-1}^{0}\beta$	 Parent → Daughter Beta Particle
Positron emission	${}_{+1}^{0}\beta$	${}_{Z}^{A}X \longrightarrow {}_{Z-1}^{A}X' + {}_{+1}^{0}\beta$	 Parent → Daughter Positron
Electron capture	X rays	${}_{Z}^{A}X + {}_{-1}^{0}e \longrightarrow {}_{Z-1}^{A}X' + \text{X ray}$	 Parent Electron → Daughter X ray
Gamma emission	${}_{0}^{0}\gamma$	${}_{Z}^{A}X^* \xrightarrow{\text{Relaxation}} {}_{Z}^{A}X' + {}_{0}^{0}\gamma$	 Parent (excited nuclear state) → Daughter Gamma ray
Spontaneous fission	Neutrons	${}_{Z+Y}^{A+B}X \longrightarrow {}_{Z}^{A}X' + {}_{Y}^{B}X' + {}_{0}^{1}n$	 Parent (unstable) → Daughters ENERGY Neutrons

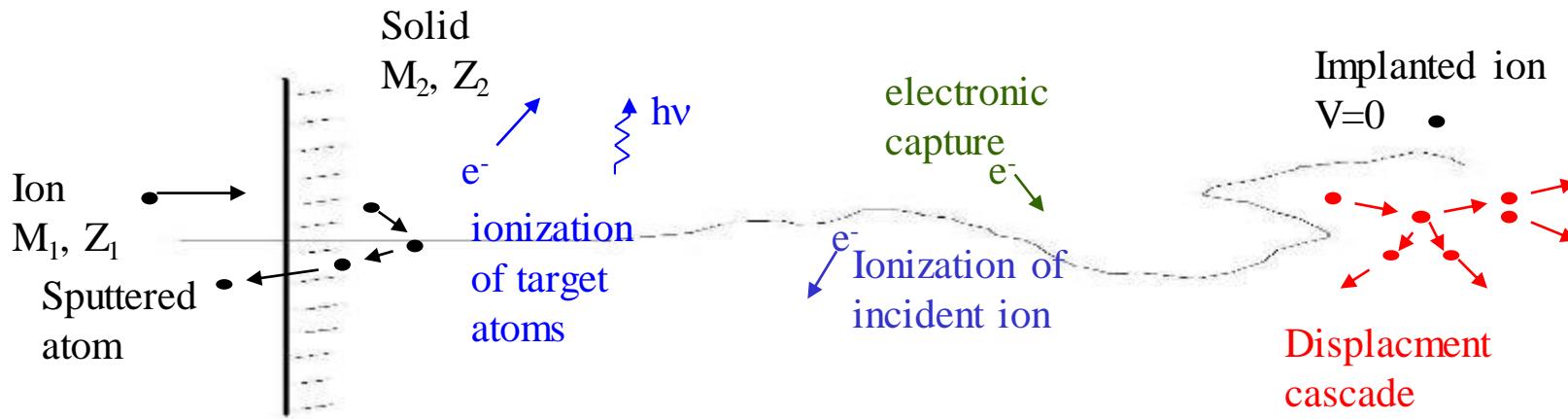
Actinides: mainly α decays

Fission products: mainly β decays

Most of alpha and beta decays

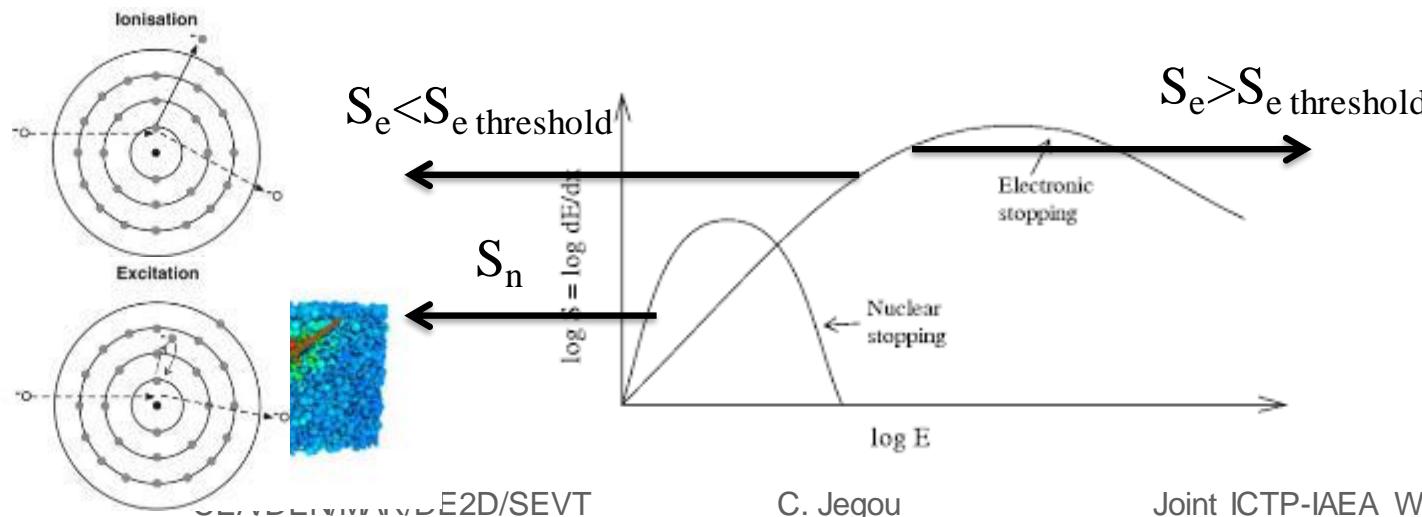
Interaction with matter

Due to the various decays: Emission of particles with high amount of energy

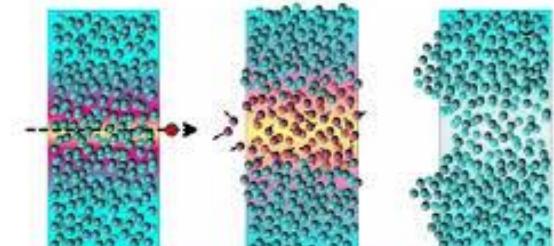


$Se = (dE/dx)_{elec}$ = Electronic energy loss due to collisions with electrons

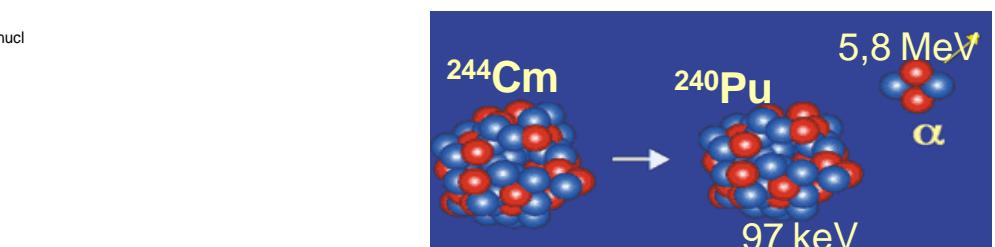
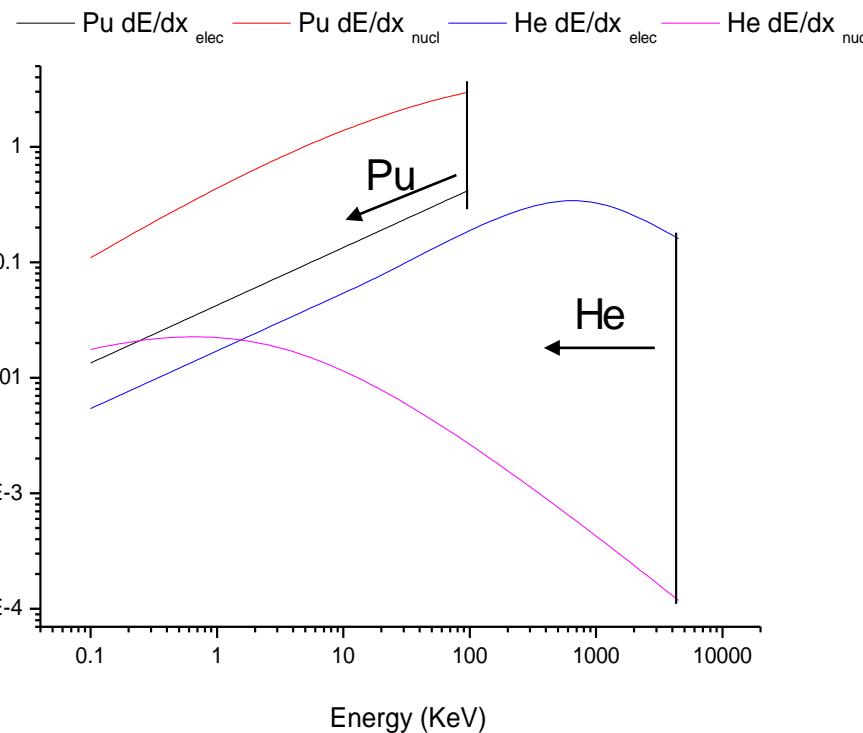
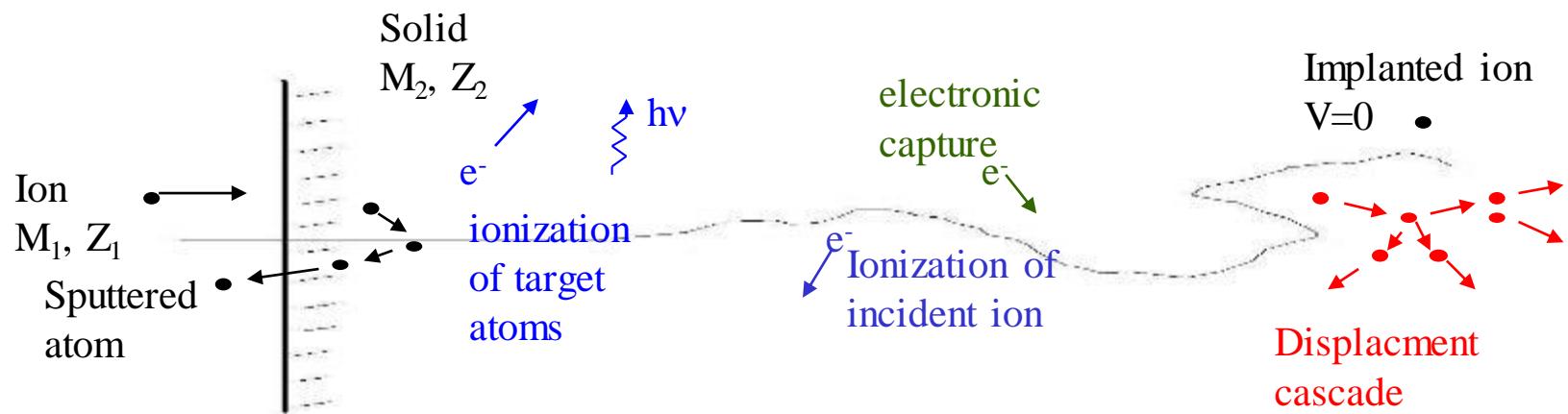
$Sn = (dE/dx)_{nucl}$ = Nuclear energy loss due to collisions with atoms



Nuclear Tracks in Solids: Principles & Applications (Fleischer, Price & Walker, 1975)



Interaction with matter



Recoil nuclei
(~100 keV)
30-40 nm

Mainly nuclear collisions
Ballistic damage
Displacement cascade

α (4-5 MeV)
20-30 μm

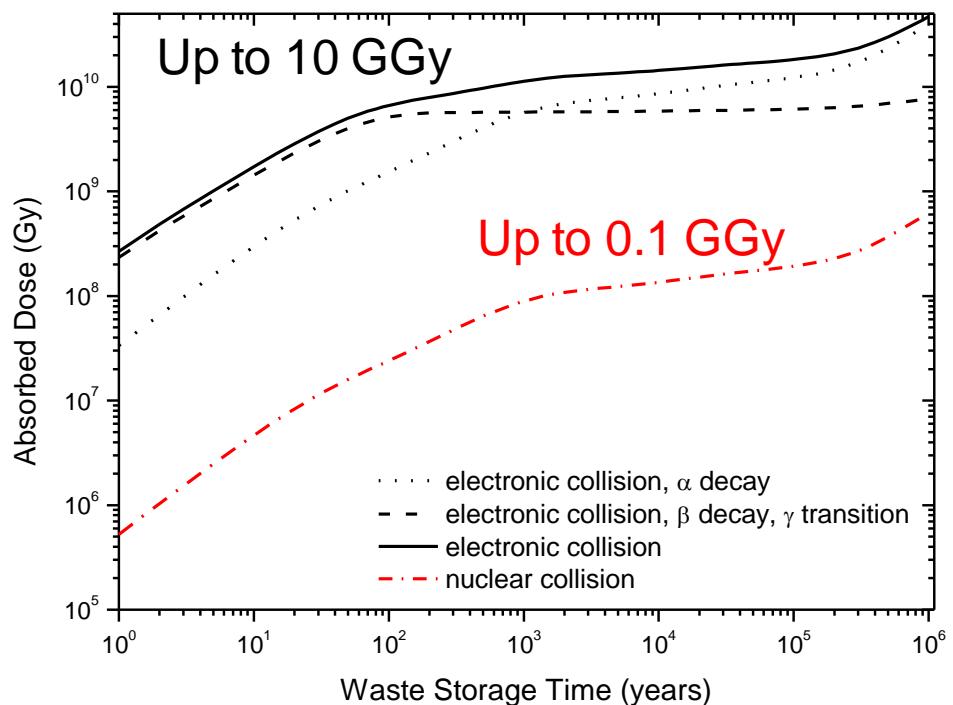
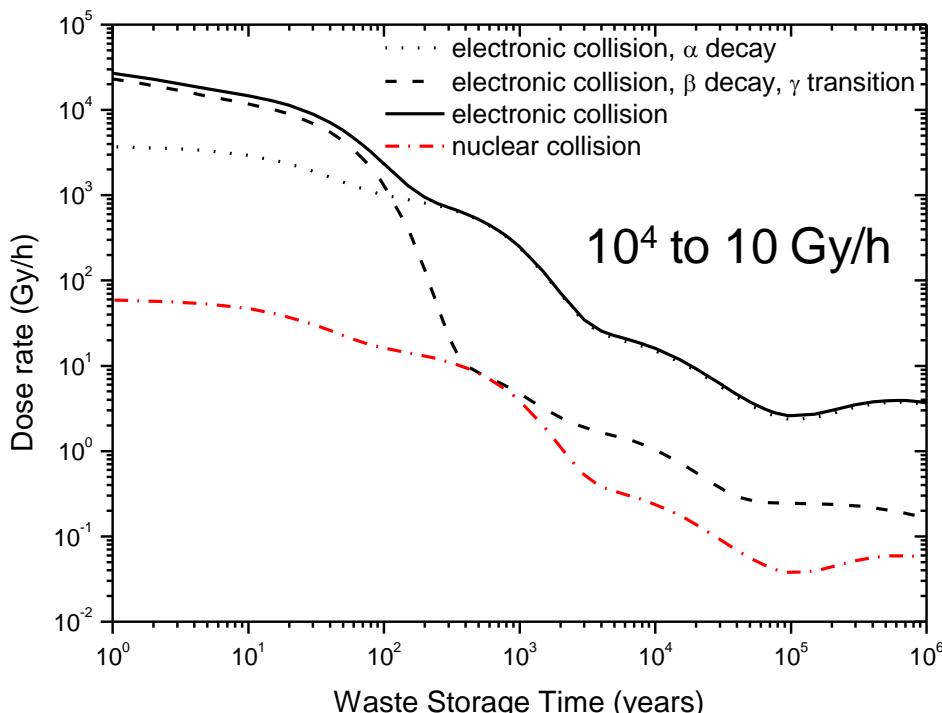
Mainly electronic collisions

Interaction with matter

Important parameters to consider:



- **Dose rate**: absorbed energy per unit of mass of material per unit of time (Gy/s)
- **Dose**: absorbed energy per unit of mass of material (Gy = J/kg)

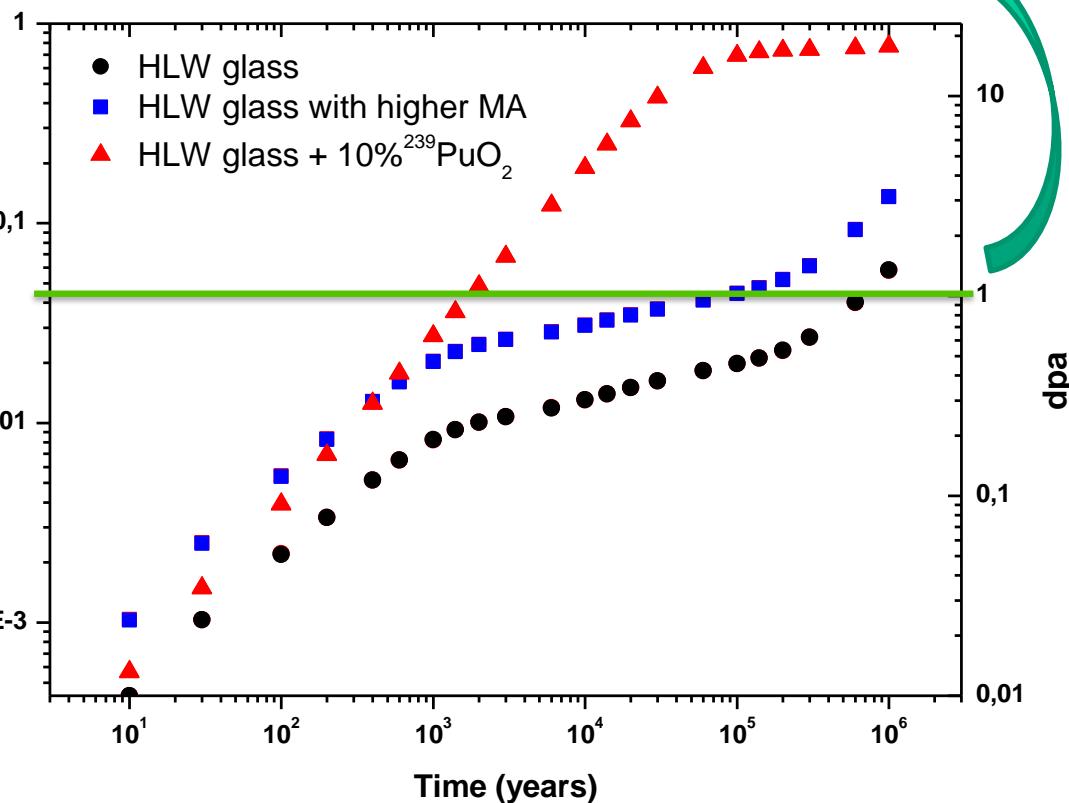


Interaction with matter

- Nuclear collisions, **dpa = displacements per atom**

$$\text{dpa} = \text{displacements per atom} = \frac{\text{Number of displaced atoms in volume from NRT equation}}{\text{Number of materials atoms in same volume}}$$

All the atoms have been displaced



$$N_d(T_d) = \begin{cases} 0 & , \quad T_d < E_d \\ 1 & , \quad E_d < T_d < 2E_d / 0.8 \\ \frac{0.8T_d}{2E_d} & , \quad 2E_d / 0.8 < T_d < \infty \end{cases}$$

$$T_D = F_{D,n} = E_0 - F_{D,e}$$

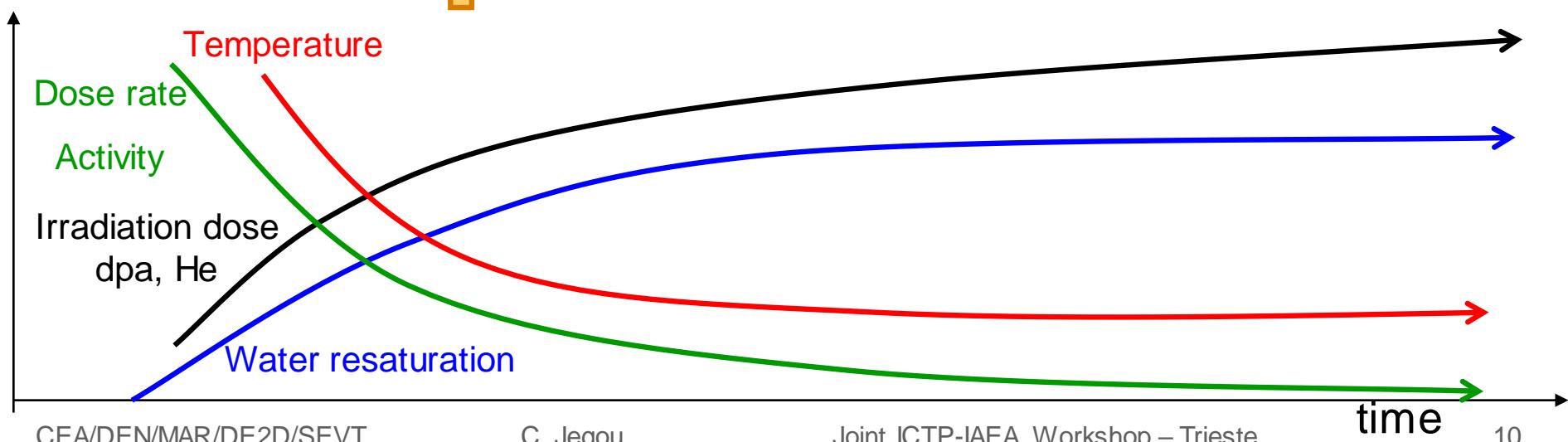
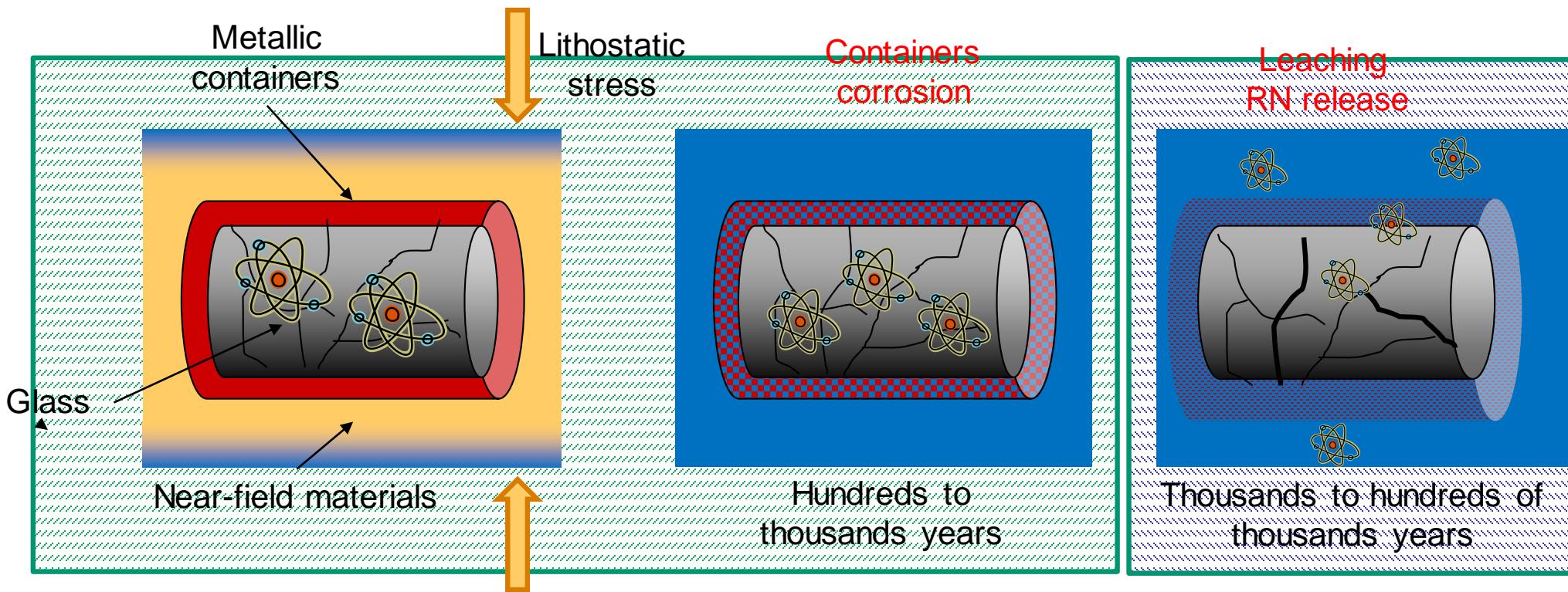
T_D =Energy available for damage production

E_0 =Energy of the particle

$F_{D,e}$ =Energy lost to electronic stopping

E_d =Threshold displacement energy

Glass Long Term Behavior – complex ageing scenario



Glass Long Term Behavior: main past studies?

Main laboratory studies of alpha decay impact

USA (NLs)	70's-90's	$<3 \times 10^{18} \alpha/g$
UK (AERE)	70's-80's	$<3 \times 10^{18} \alpha/g$
France (CEA)	70's- 80's	$<3 \times 10^{18} \alpha/g$
EU (ITU)	70's-90's	$<5 \times 10^{18} \alpha/g$
JAPAN (JAERI)	90's	$<10^{19} \alpha/g$

**Macroscopic behavior in a limited level of dose
but no data on the glass structure !**

Need to improve the understanding of alpha decays effects

To predict long term behavior

To explore nuclear glass limits

To optimize the future glass or glass ceramics composition

Focus on the results of the research program started in 2001 at CEA

Methodology to simulate alpha decays effects

- Accelerate the time scale
- Dissociate the effects of self-irradiation (electronic / nuclear) and helium generation
- Evaluate the effects on the confinement properties
- Evaluate the effects on the glass structure

Propose some models to explain the glass behavior under alpha self-irradiation

1. Curium doped glasses

Atalante DHA, CEA



2. External irradiation with light and heavy ions

IPN Orsay Lyon, Ganil



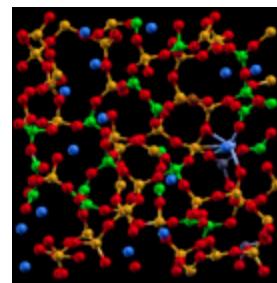
3. In pile irradiation : $^{10}\text{B}(\text{n},\alpha)^7\text{Li}$

OSIRIS, CEA



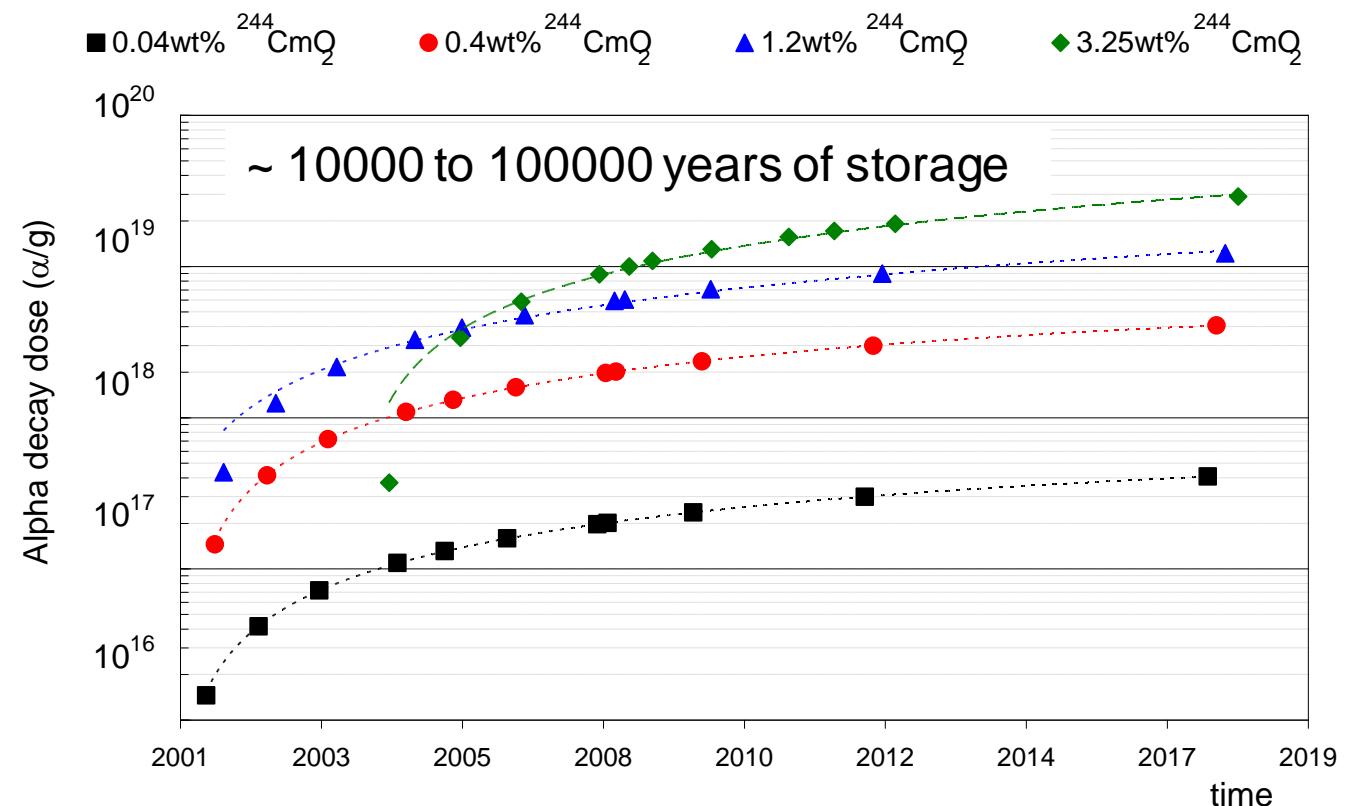
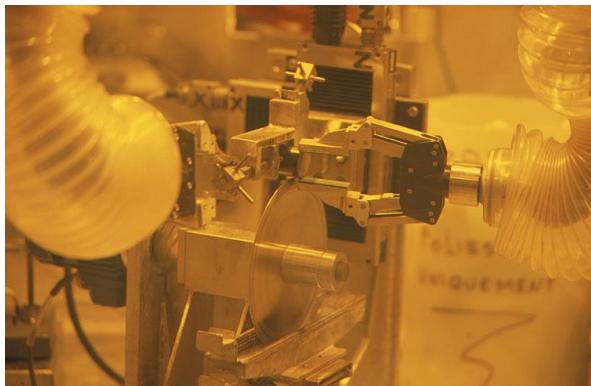
4. Molecular dynamic modeling of ballistic effects

DM, CEA



Methodology: Cm doping

- SON68 glasses doped with 0.04, 0.4, 1.2, 3.25wt% of $^{244}\text{CmO}_2$
- International Standard Glass (ISG) doped with 0.7wt% of $^{244}\text{CmO}_2$

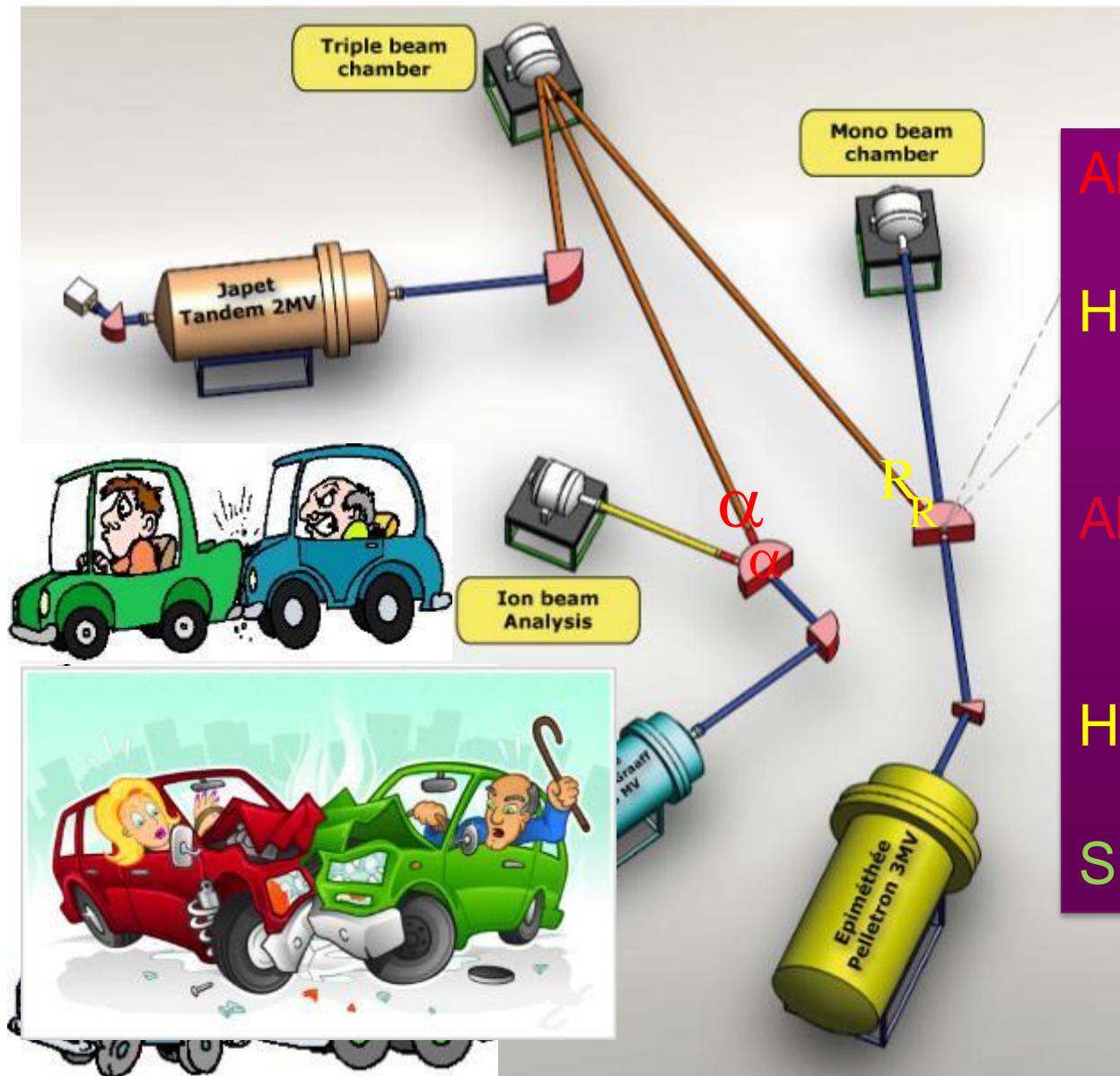


Mol%	SiO_2	Na_2O	B_2O_3	Al_2O_3	CaO	ZrO_2	Other oxides
ISG/CJ4	60.1	12.6	16.0	3.8	5.7	1.7	
R7T7	52.8	11.3	14.1	3.4	5.0	1.6	11.8

- Initial characterizations of the glasses (homogeneity, chemical composition)
- Periodical characterizations of the glass properties

Methodology: Ion beam irradiation experiment

Jannus Saclay, Orsay, Ganil



Alpha particles

Heavy ions (RN)

Alpha + Heavy ion

Heavy ion + Alpha

Simultaneous

Mono beam

Double beam

Methodology: In pile irradiation : $^{10}\text{B}(\text{n},\alpha)^7\text{Li}$



OSIRIS reactor, CEA SACLAY

Glass samples : polished disks
thickness 0.5 mm



Aluminum
sample holder
in contact with
cooling water

Neutron
detectors

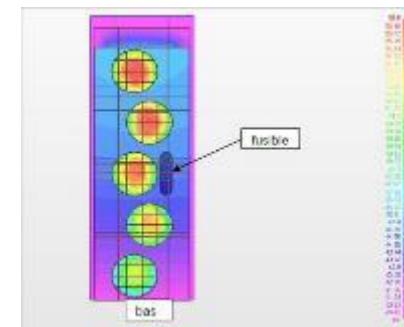


Glass



	D1	D2	D3	D4
E (MeV)	He(1.47) + Li(0.84)			
Fluence (neutron cm ⁻²)	5.9×10^{18}	1.2×10^{19}	3.5×10^{19}	5.2×10^{19}
Number of events (ion cm ⁻³)	3.5×10^{19}	7.0×10^{19}	2.1×10^{20}	3.1×10^{20}
dE/dx _{nucl} (keV nm ⁻¹)	dE/dx(He) <0.03 dE/dx(Li) <0.06			
dE/dx _{elec} (keV nm ⁻¹)	dE/dx(He) <0.33 dE/dx(Li) <0.56			
E_{nucl} (GGy)	0.06	0.13	0.39	0.57
E_{elec} (GGy)	5.16	10.45	30.69	45.71
Dpa	0.27	0.54	1.6	2.38

Mol%	SiO ₂	Na ₂ O	B ₂ O ₃	Al ₂ O ₃	CaO	ZrO ₂	Other oxides
CJ1	67.7	14.2	18.1				
SON68	52.8	11.3	14.1	3.4	5.0	1.6	11.8



Thermal modeling and fuses
observations after irradiation:

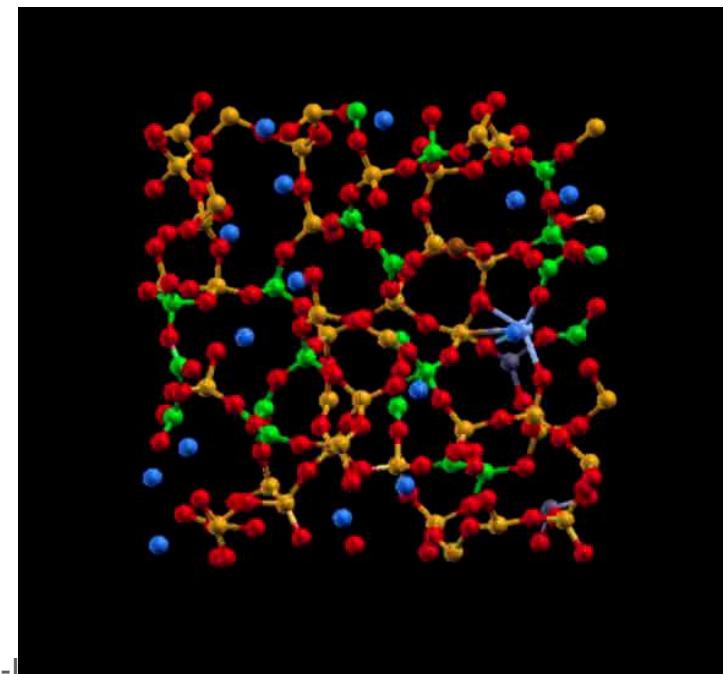
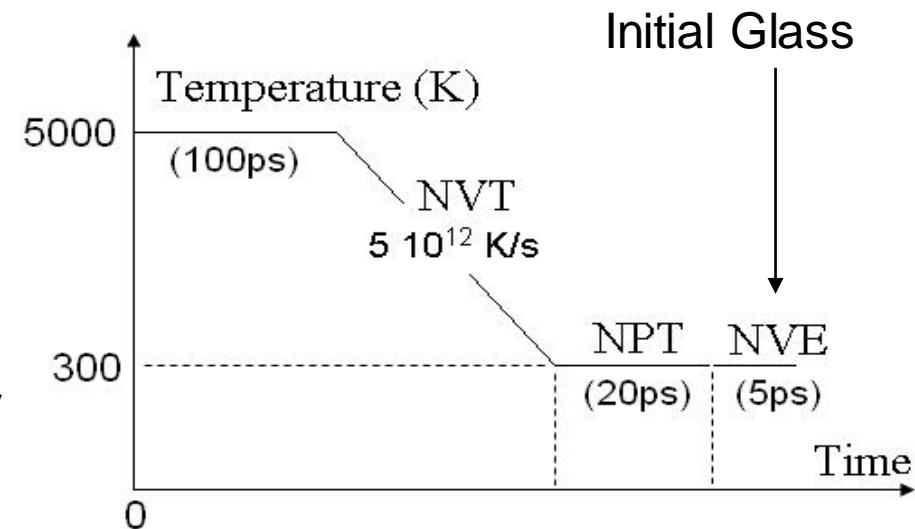
T<70°C

Methodology: Molecular dynamic modeling

- Simplified borosilicate glasses (CJ1, CJ7)

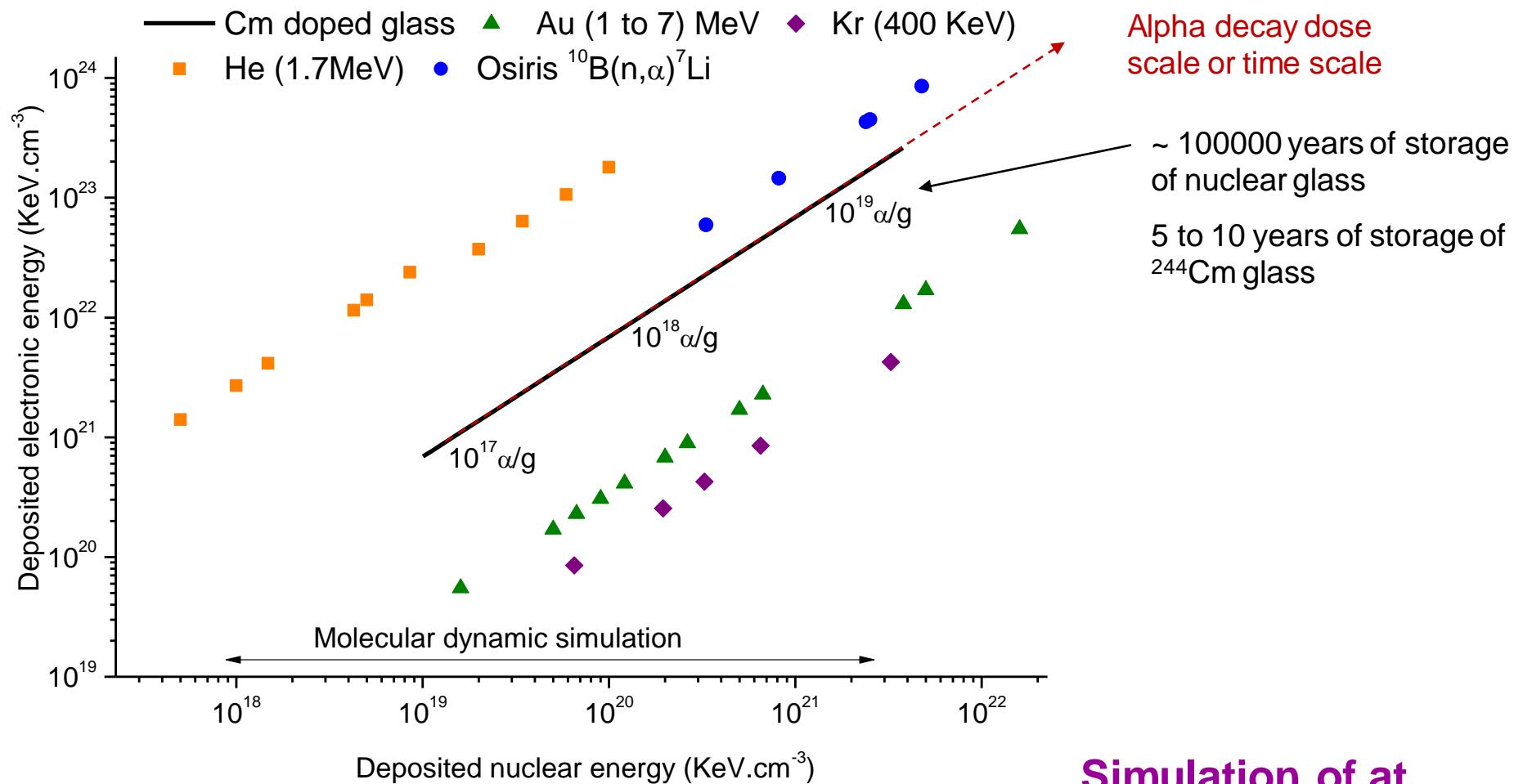
$$\phi(r_{ij}) = \frac{q_i q_j}{r_{ij}} + B_{ij} \exp\left(-\frac{r_{ij}}{\rho_{ij}}\right) - \frac{C_{ij}}{r_{ij}^6}$$

- Accumulation of displacement cascades caused by uranium atoms of energies from 700eV to 70keV
- Characterization of the structural modifications induced by displacement cascades (SRO and MRO)



Mol%	SiO ₂	Na ₂ O	B ₂ O ₃	Al ₂ O ₃	CaO	ZrO ₂	Other oxides
CJ1	67.7	14.2	18.1				
CJ7	63.8	13.4	17.0	4.1		1.8	
<i>SON68</i>	52.8	11.3	14.1	3.4	5.0	1.6	11.8

Methodology: Materials and irradiation conditions



Light ions irradiations (He) : mainly electronic interactions

Heavy ions irradiations (Kr, Au) : mainly nuclear interactions

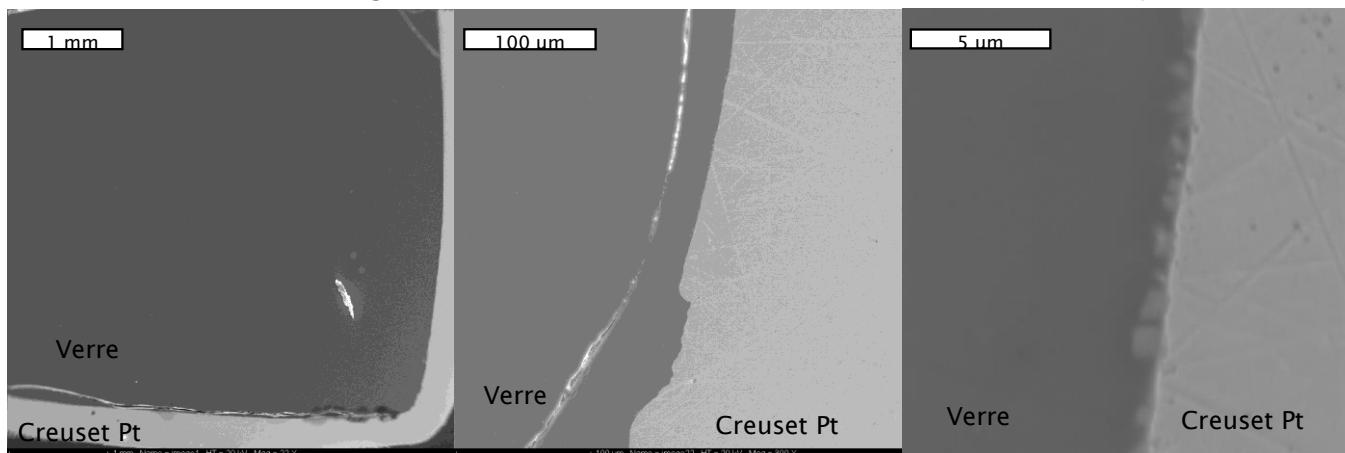
Doped glasses and OSIRIS irradiation : electronic and nuclear interactions

Molecular Dynamics : only nuclear interactions

Simulation of at least 100000 years of disposal by various methods !

Stability of the metastable glassy state ? Effect on the glass microstructure

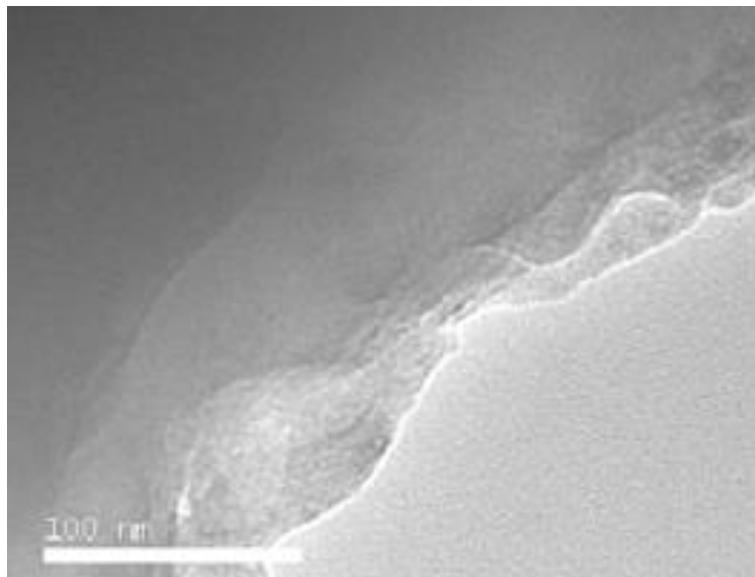
^{244}Cm SON 68 glass : SEM (CEA Marcoule), alpha decay dose $2 \times 10^{19} \alpha/\text{g}$



(Around 100000 years of storage)

S. Peugeot et al. JNM 44 (2014)

^{244}Cm SON 68 glass : TEM (ITU Karlsruhe), alpha decay dose $8 \times 10^{18} \alpha/\text{g}$



Homogeneous microstructure,
without bubbles, phase
separation or crystallization
Stability of the glassy state



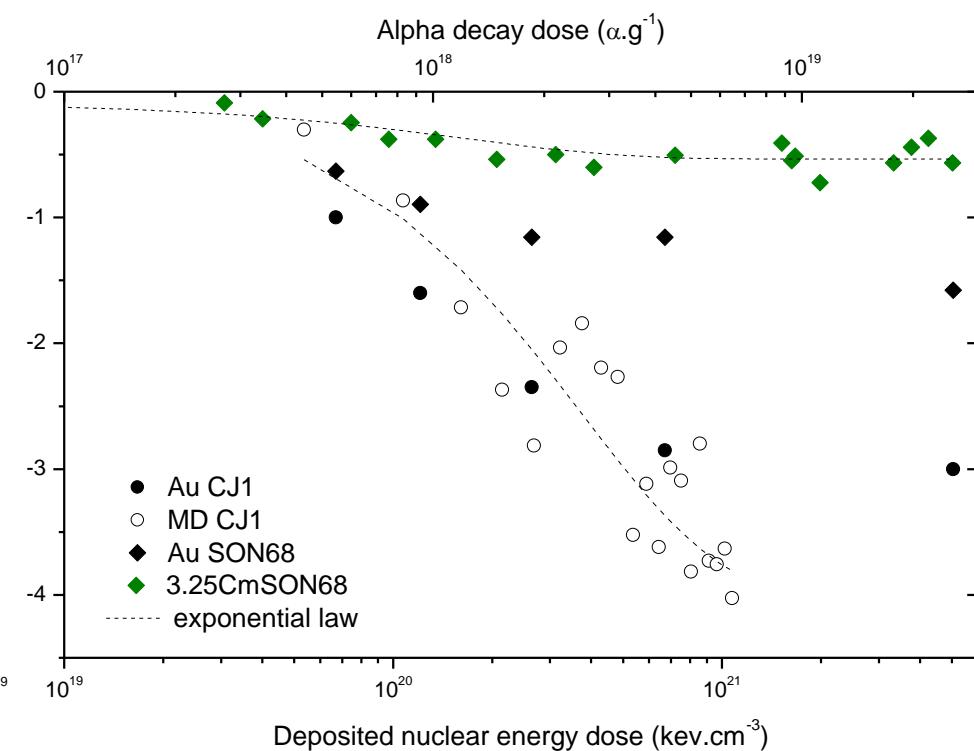
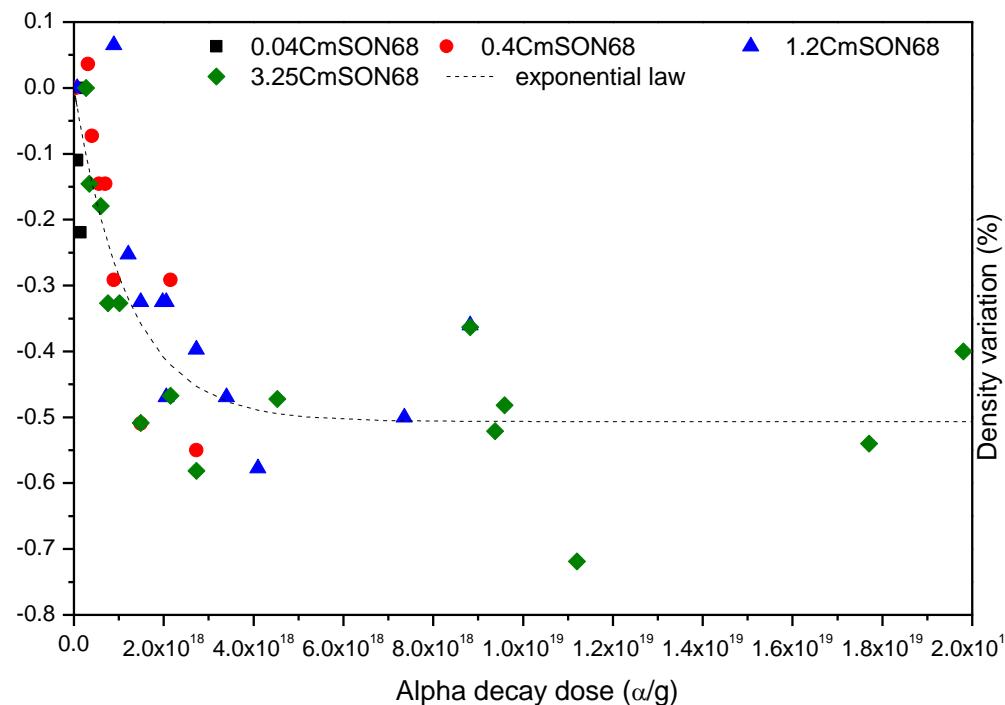
Slight decrease of the glass density (0.5%)

No effect of the dose rate

Stabilization of the evolution at around $4 \times 10^{18} \alpha/g$

Evolution according to an exponential law (direct impact model)

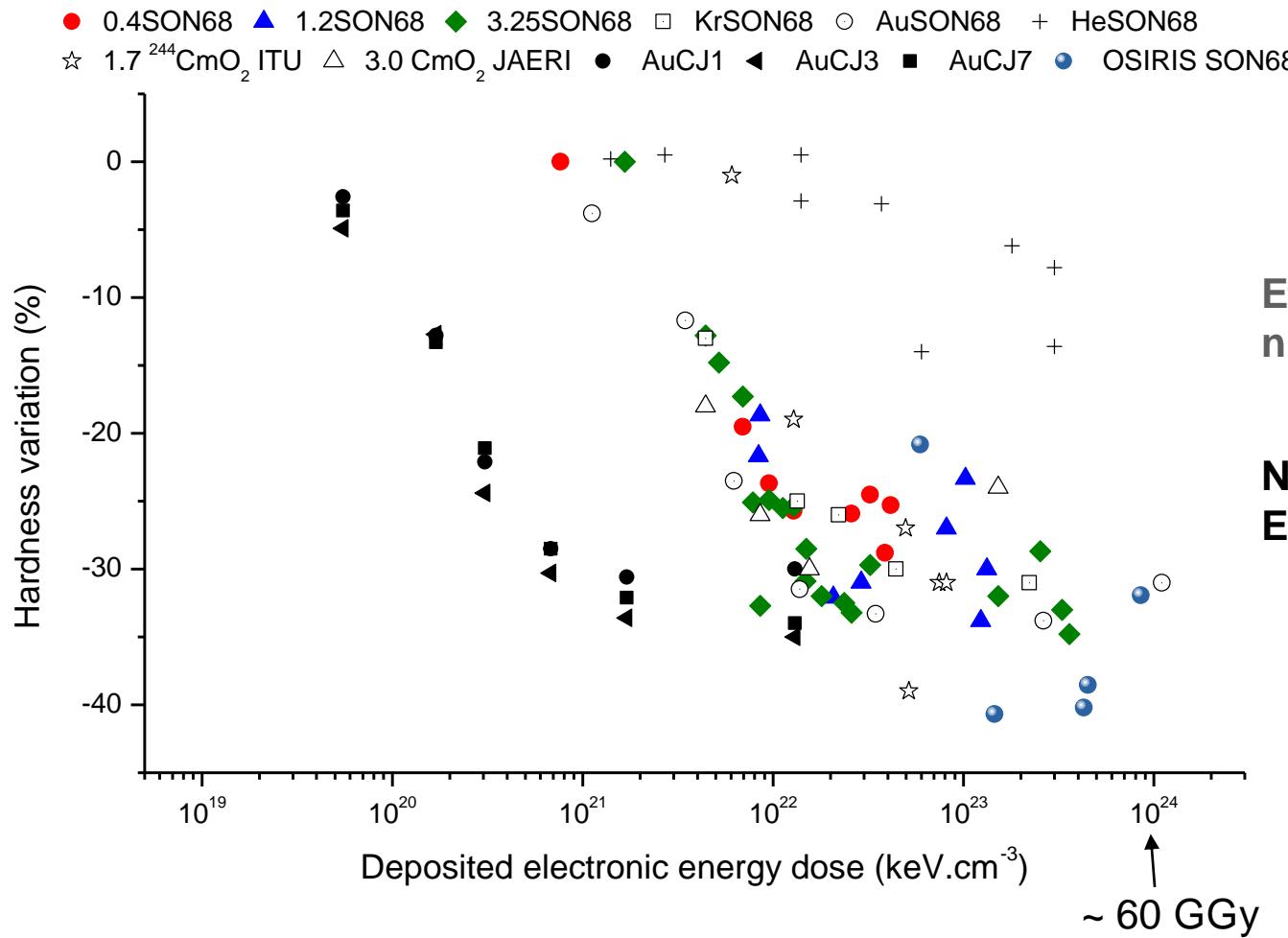
- ✓ Variations correctly simulated by external irradiations with heavy ions and MD simulation
- ✓ Swelling level is lower under α decays irradiation (0.5% compared to 1.2% Au irradiation)



Mechanical properties: example of hardness

Decrease of hardness on curium doped glasses and ions irradiated glasses

He induced lower changes



Effect of electronic or nuclear interactions?

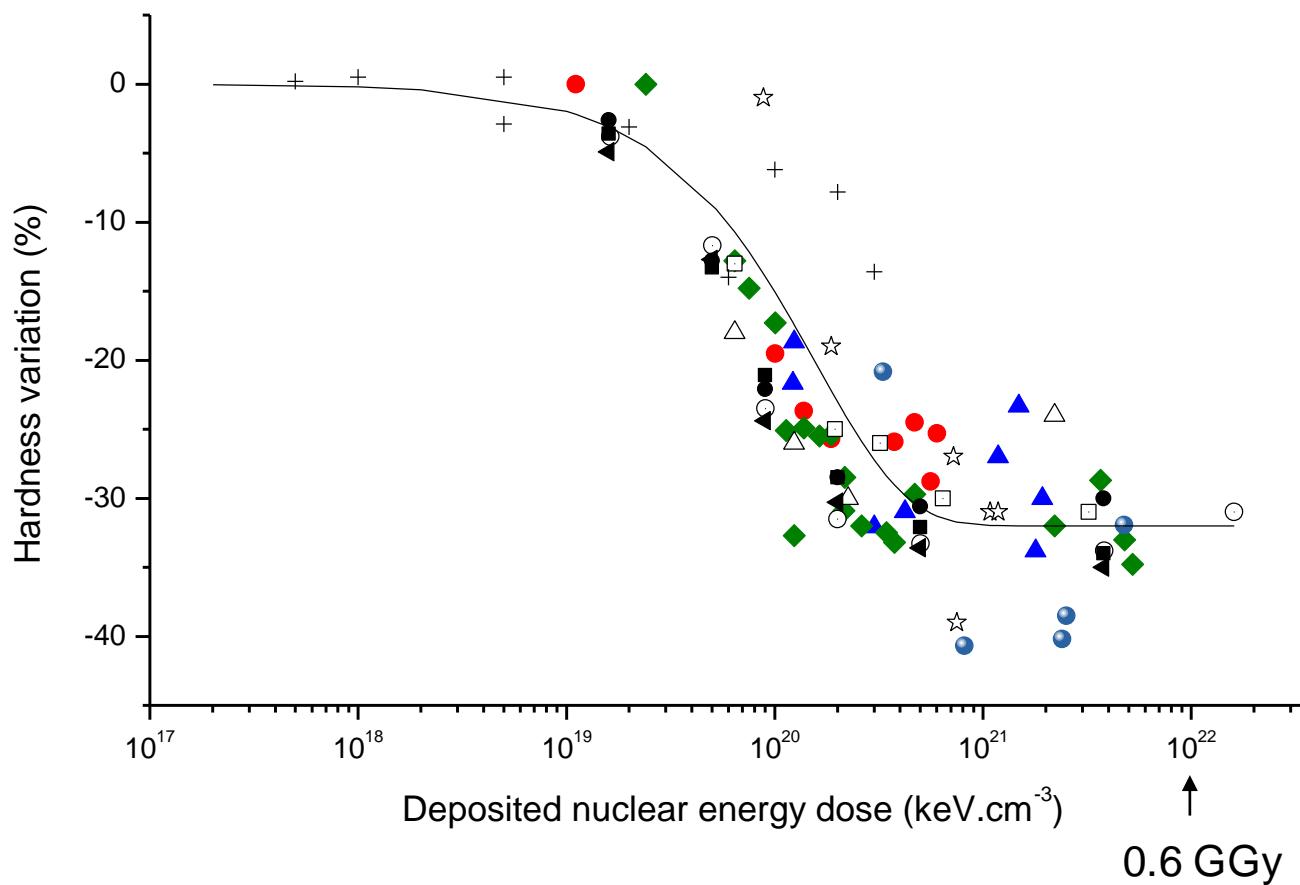
No agreement versus Electronic dose

Mechanical properties: example of hardness

Decrease of hardness on curium doped glasses and heavy ions irradiated glasses

He induced lower changes

- 0.4SON68 ▲ 1.2SON68 ♦ 3.25SON68 □ KrSON68 ○ AuSON68 + HeSON68
- ☆ 1.7 $^{244}\text{CmO}_2$ ITU △ 3.0 CmO_2 JAERI ● AuCJ1 ◀ AuCJ3 ■ AuCJ7 ● OSIRIS SON68



Effect of electronic or nuclear interactions?

Quite good agreement between doped glasses and heavy ions irradiated glasses

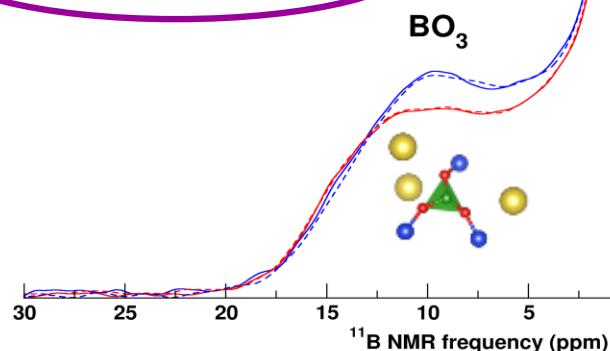


Effect induced by nuclear interactions, rôle of RN !

^{244}Cm ISG glass

— CJ₄ Cm (04/13) $4 \times 10^{18} \alpha/\text{g}$
 — CJ₄ Cm annealed (04/13)

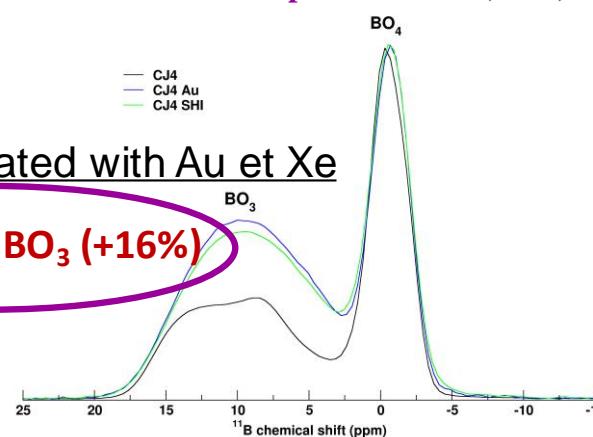
Increase of BO₃ (+7%)



T. Charpentier et al. Scientific Reports 6:25499 (2016)

ISG irradiated with Au et Xe

Increase of BO₃ (+16%)



C. Mendoza et al. NIMB 325 (2014) 54-65



EURACTⁿMR
International Platform for NMR Research and Training

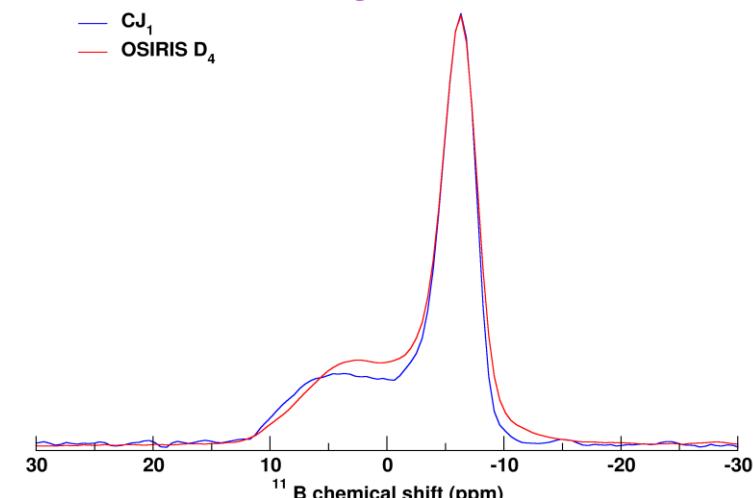
RMN at ITU

11B MQMAS

CJ1 irradiated in OSIRIS reactor

S. Peuget et al, NIMB 327 (2014) 22-28

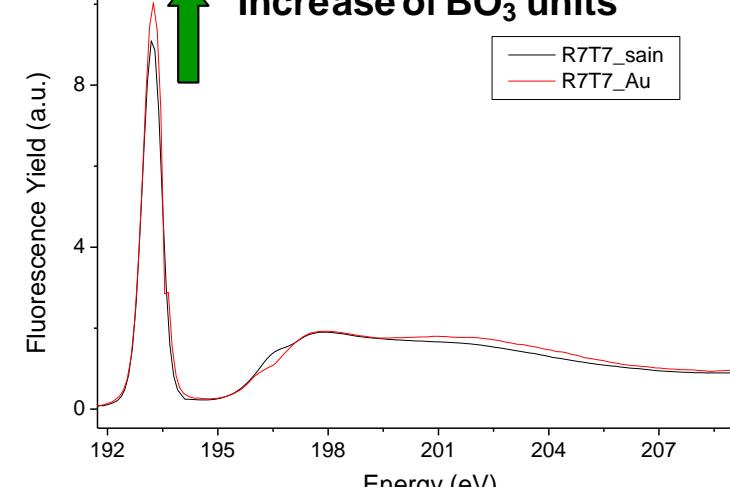
— CJ₁
 — OSIRIS D₄



Xanes B K edge: R7T7 irradiated with Au

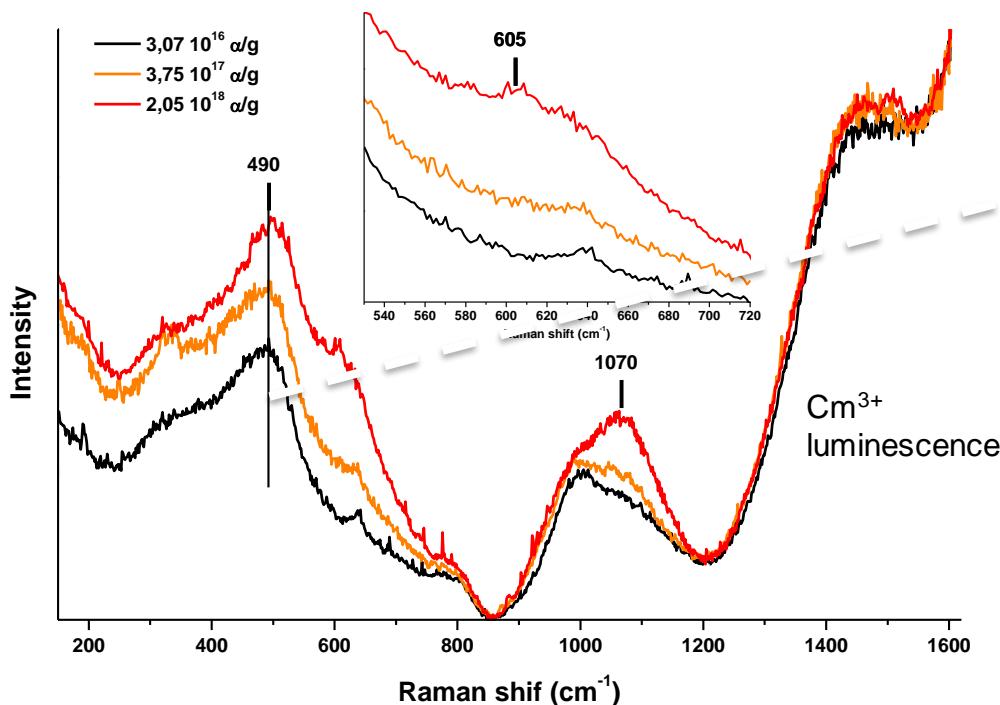
Increase of BO₃ units

— R7T7_sain
 — R7T7_Au



G. Bureau, thesis, (2008)

Raman spectroscopy on Cm doped ISG (Atalante, DHA)



C. Mendoza et al. Proc. Chem. 7 (2012) 581



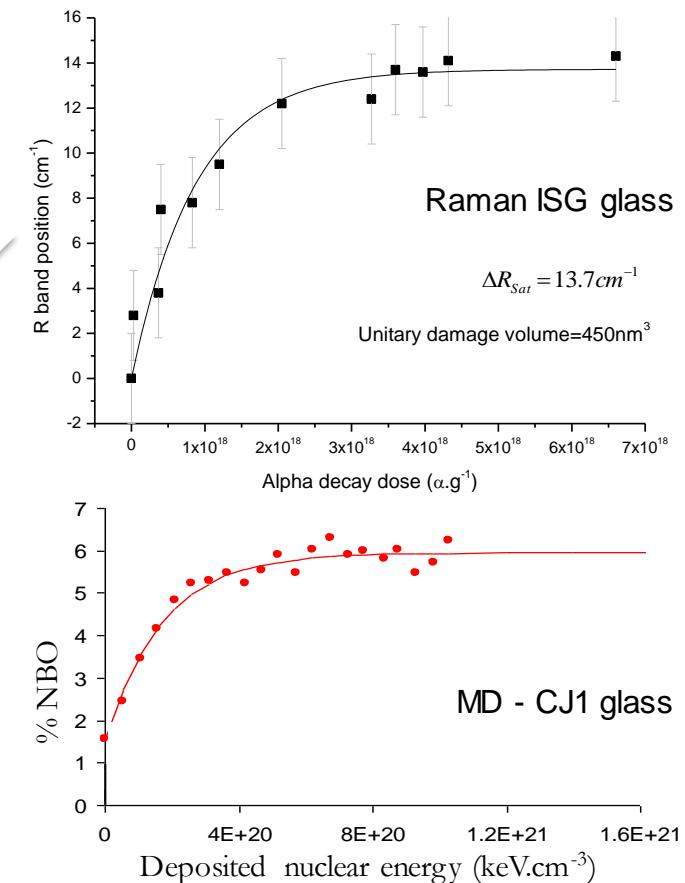
- Increase of Q3 contribution in ISG glass : more NBO

- Slight shift of the vibration band around 500cm^{-1}

Decrease of the mean angle between silica tetrahedra

- New D2 band on ISG Cm doped glass: 3 members silica rings

- Stabilization of the silicon local environment after around $4 \times 10^{18} \alpha/\text{g}$

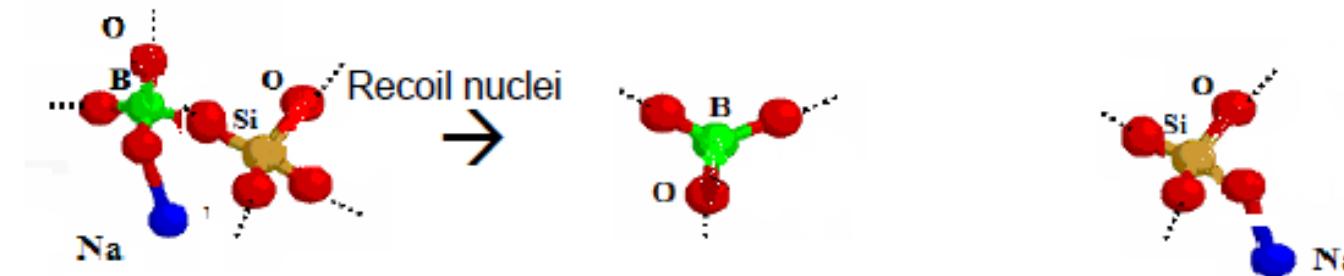


J.-M. Delaye et al, J. Non-Cryst. Solids 357 (2011) 2763

Effects on the glass structure? Summary

Modification of the Short Range Order

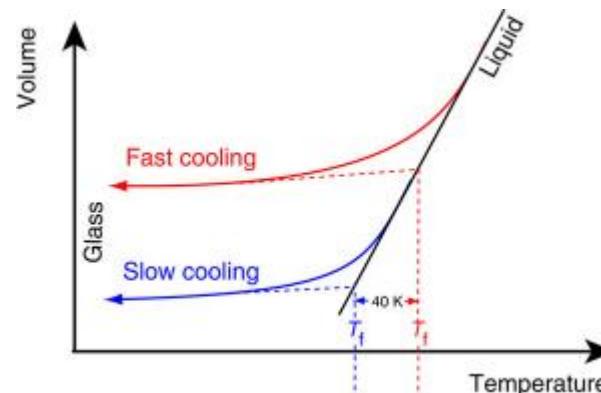
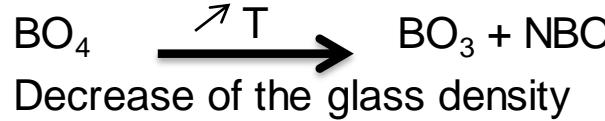
Increase of trigonal boron, increase of NBO



Modification of the Medium Range Order

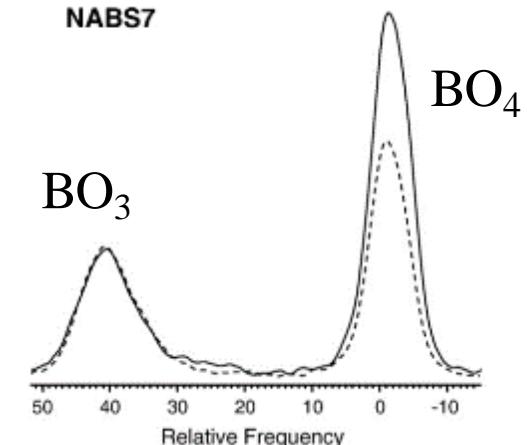
Ring statistic modification, increase of glass disorder and Si/B mixing

Effects similar to those induced by thermal quenching of a molten glass



¹¹B NMR on quenched and annealed glass
NABS7

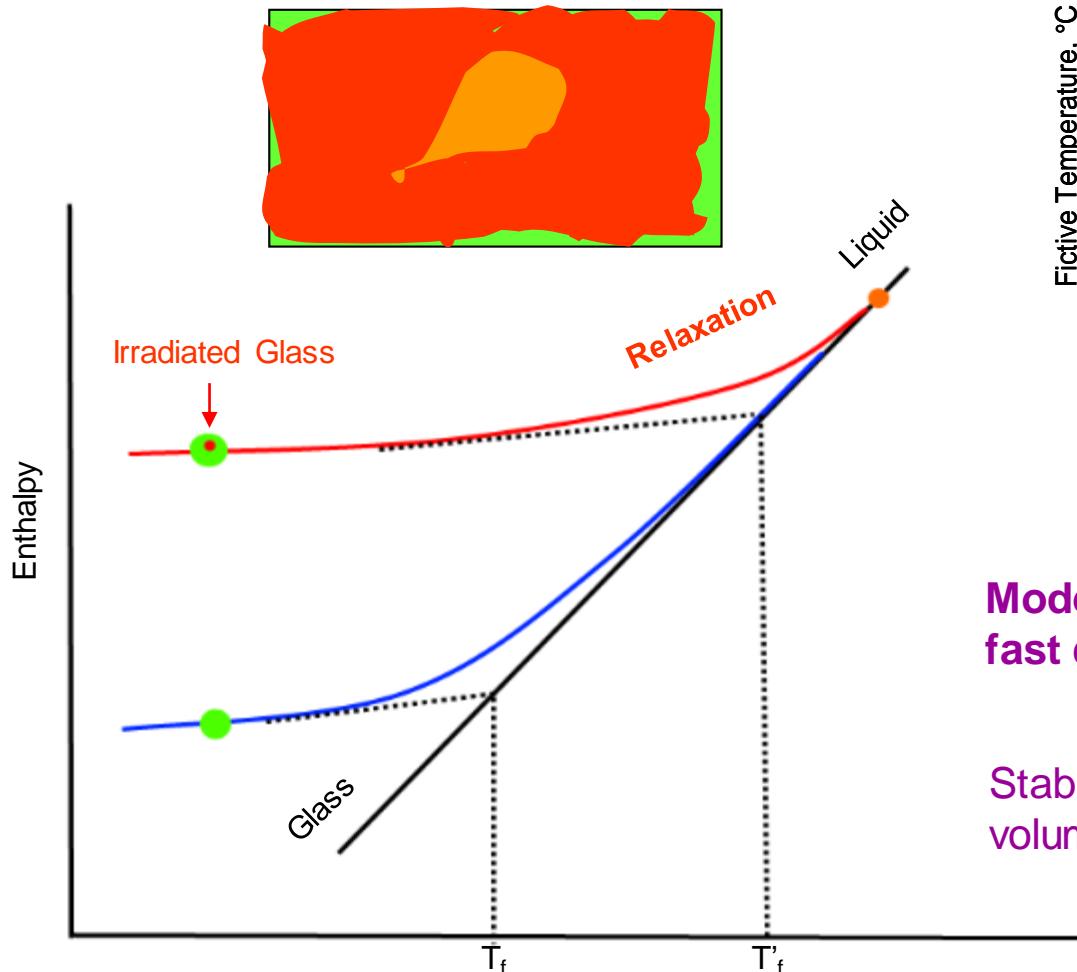
Wu and Stebbins JNCS 356 (2010)



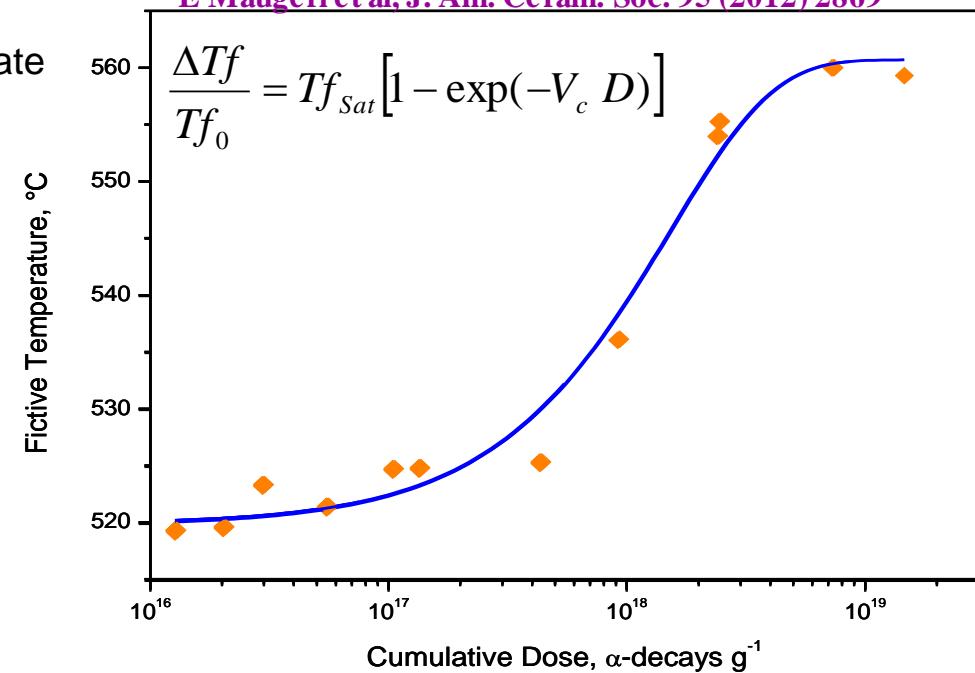
Understanding of glass behavior under alpha decays

1. Balistic step : disordered state
2. Relaxation step : very important quenching rate

Irradiated zone has a higher fictive temperature



E Mauger et al, J. Am. Ceram. Soc. 95 (2012) 2869



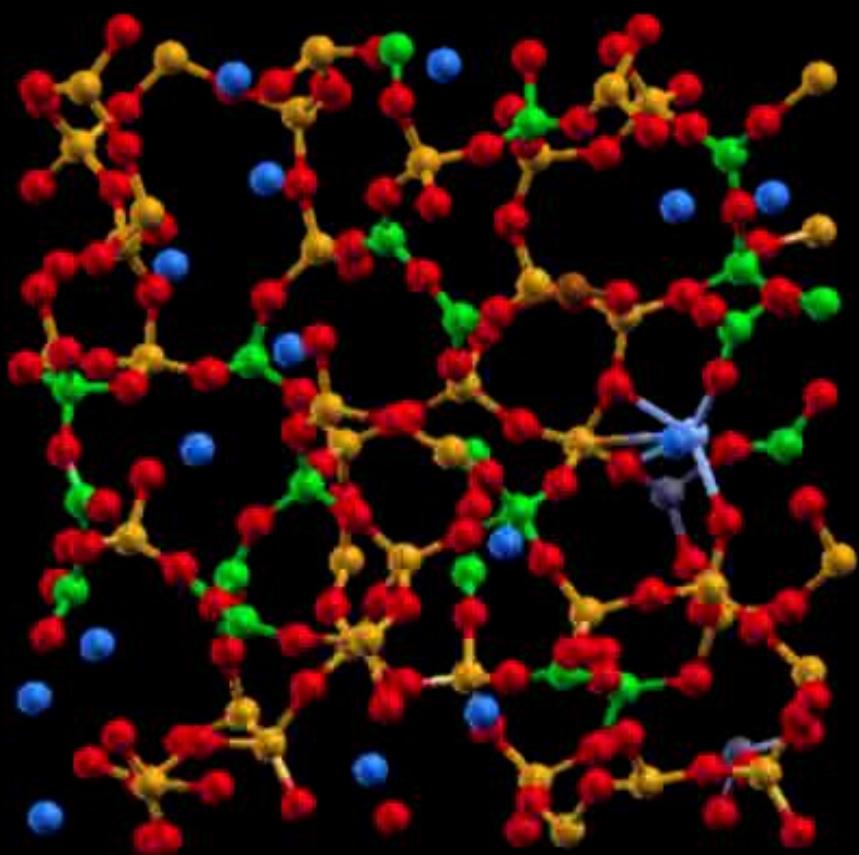
Model of accumulation of ballistic disordering fast quenching events: “supervitrification”

Stabilization of a new glass structure when all the volume has been damaged once

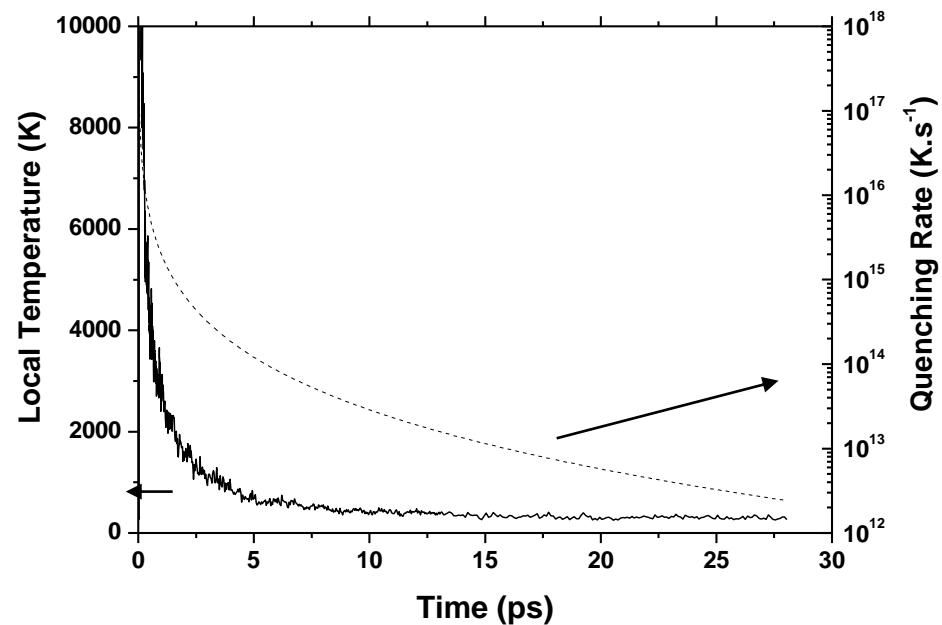
Effects on the glass structure? Ballistic damage

What happen in the displacement cascade induced by a recoil nuclei?

JM Delaye, PRB 61 (2000) 14481



1. Ballistic phase
2. Thermal phase



Golden = Si
Green = B
Blue = Na
Red = O

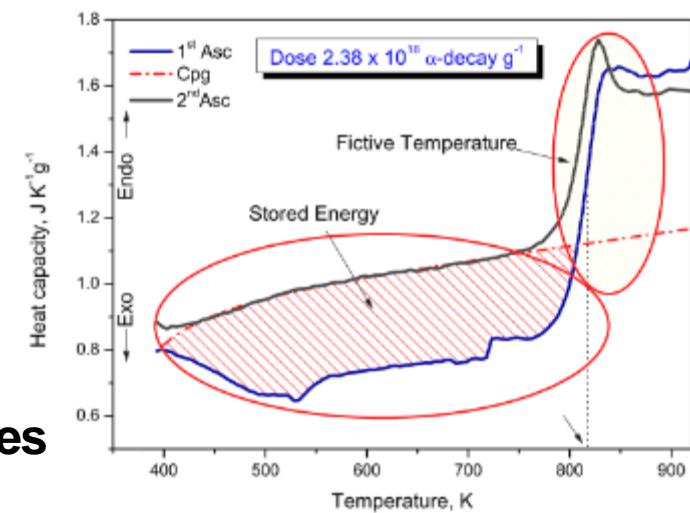
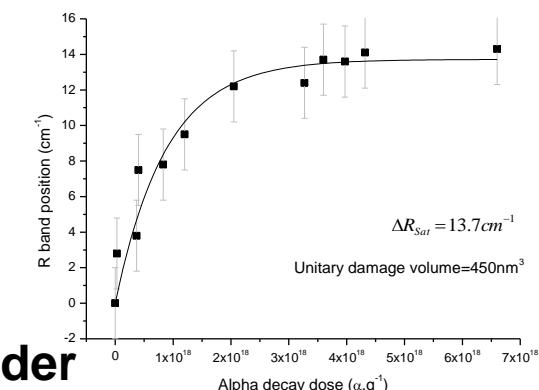
Very high quenching rate of the disordered state induced by the displacement cascade

Supervitrification

Characteristics of the irradiated glassy state? α irradiation

Main effects observed under alpha decay irradiation:

- ✓ A saturation effect with dose, a new glass structure is reached after around $4 \times 10^{18} \text{ } \alpha/\text{g}$ (Nuclear dose $\sim 30 \text{ MGy}$)
- ✓ No effect of the dose rate in the relevant range
- ✓ Changes at both Short Range Order and Medium Range Order
 - ✓ Changes in boron coordination number and glass polymerization index
 - ✓ Changes in ring statistic, angle distribution
- ✓ A higher fictive temperature after irradiation
- ✓ Stored energy of $\sim 100 \text{ J/g}$
- ✓ Complex glasses are less modified than simple glasses



Waste mechanical degradation?

Can irradiation induce a cracking of the material?

- Due to important swelling under irradiation?
- Degradation of the mechanical properties?
- Due to bubble formation (He bubbles generated by alpha decays)



Waste mechanical degradation?

Can irradiation induce a cracking of the material?

- Due to important swelling under irradiation?
- Degradation of the mechanical properties?
- Due to bubble formation (He bubbles generated by alpha decays)

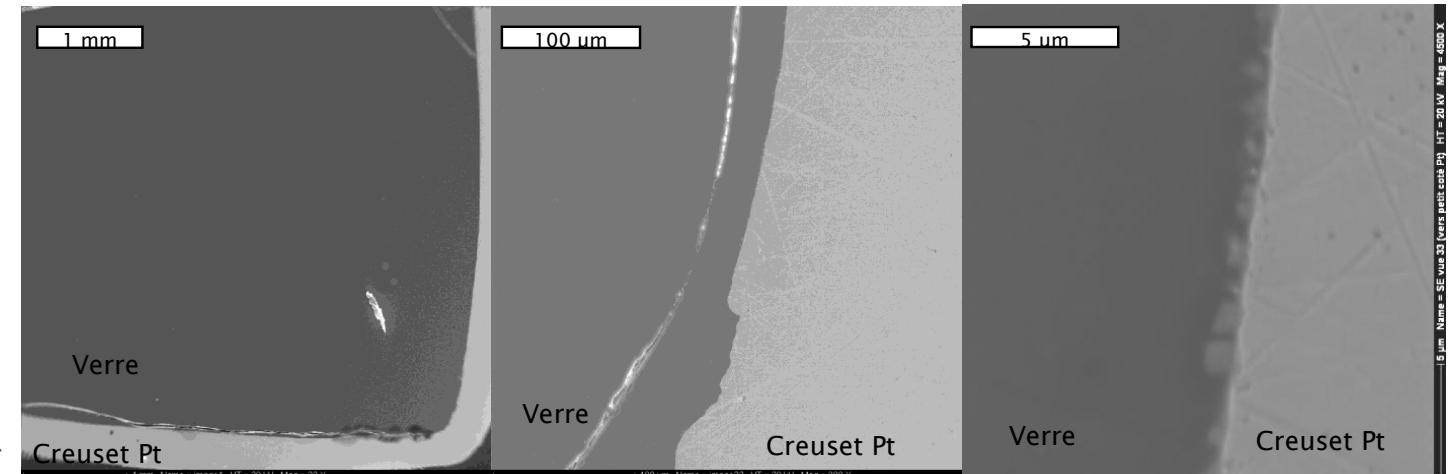
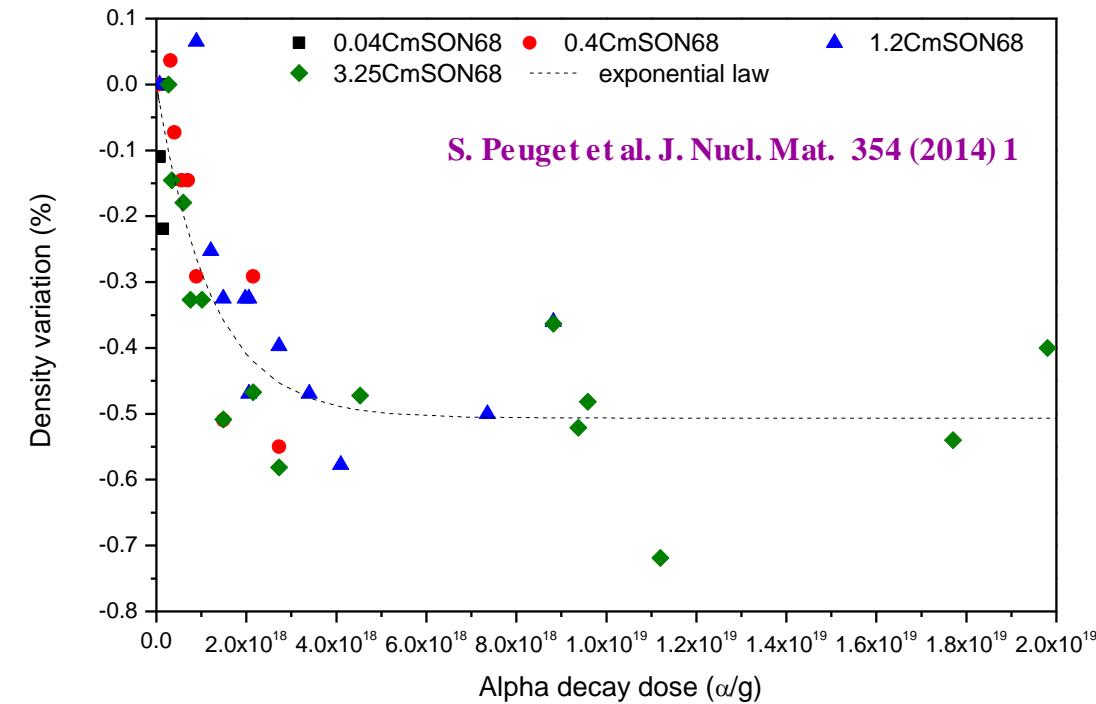


WASTE MECHANICAL DEGRADATION?

Important swelling of Homogeneous glass?

Slight variation of the glass density

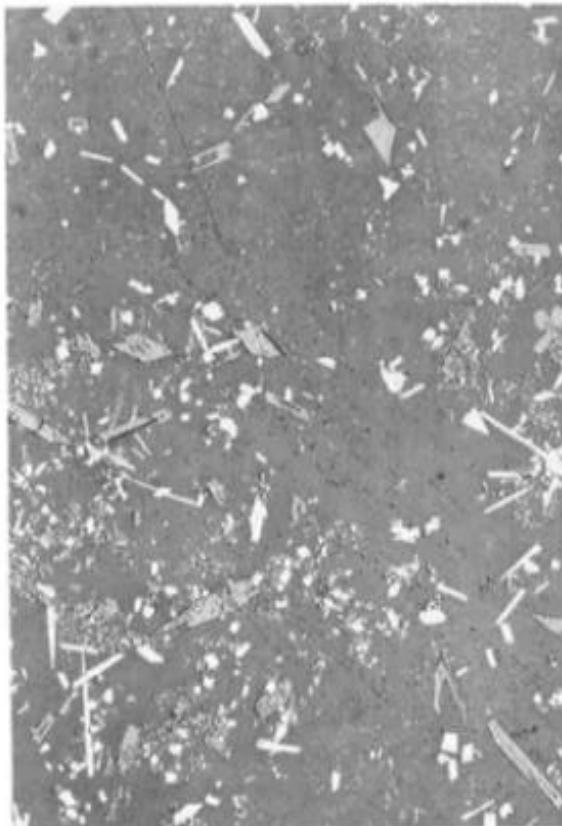
Low swelling level, no microcraking



WASTE MECHANICAL DEGRADATION?

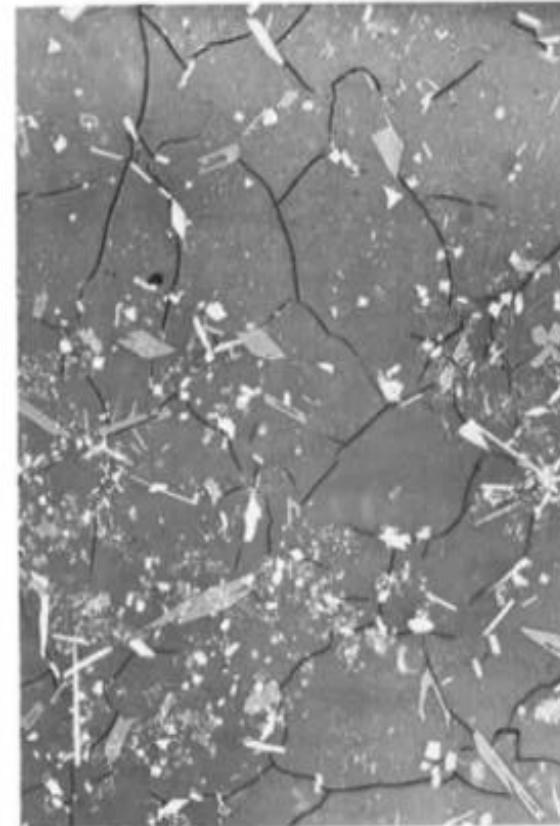
Waste form degradation ? GCM?

Microcracking observed on some GCM



0.02×10^{24} ALPHA DECAY/m³

100 µm



0.8×10^{24} ALPHA DECAY/m³

W. J. Weber and F. P. Roberts, Nuclear Technology, vol. 60., 178-198.

Amorphization of the
crystalline phases: high
swelling level of crystalline
phase

To go further in GCM
development:
Need to understand and master
the origin of radiation induced
cracking
Evaluation of the impact of
type of phase, density and size
of crystalline phases

Waste mechanical degradation?

Can irradiation induce a cracking of the material?

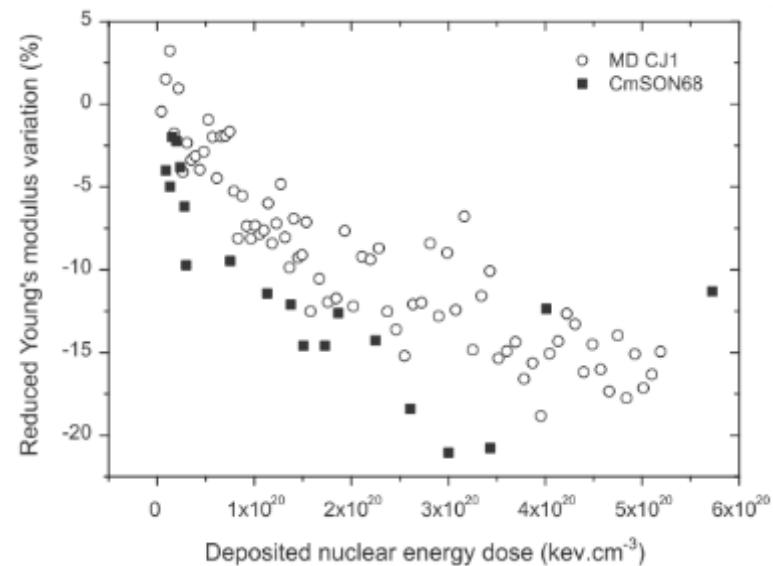
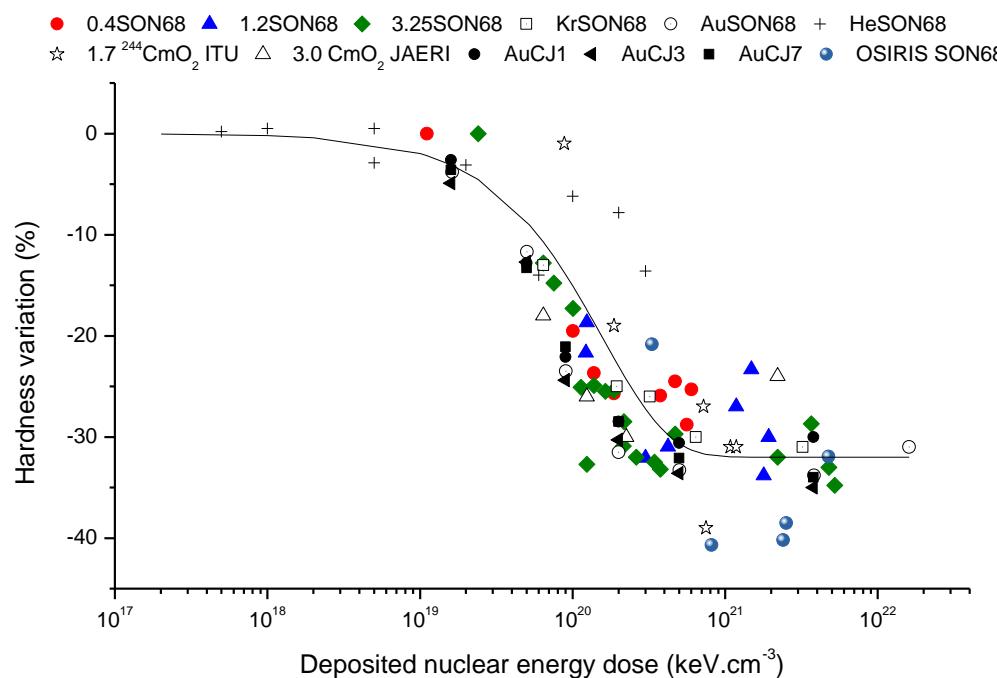
- Due to important swelling under irradiation?
- Degradation of the mechanical properties?
- Due to bubble formation (He bubbles generated by alpha decays)



WASTE MECHANICAL DEGRADATION?

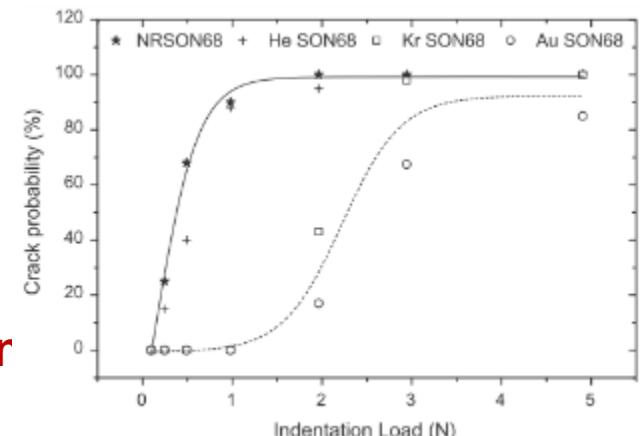
Degradation of the mechanical properties?

Decrease of Hardness, Young Modulus, increase of fracture toughness



No significant degradation of the mechanical properties
Even slightly better, fracture toughness increase ...

Origin associated to structural changes under irradiation



Waste mechanical degradation?

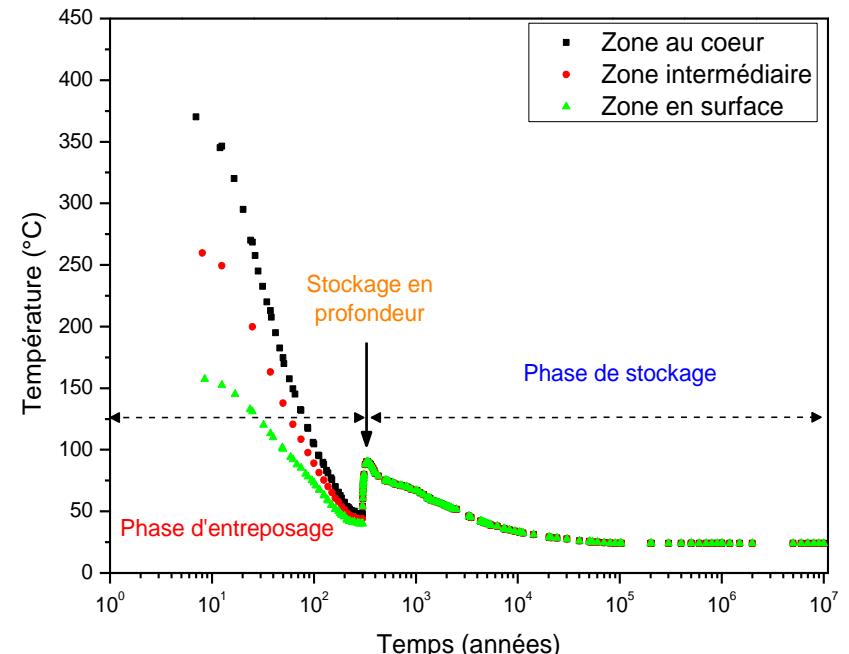
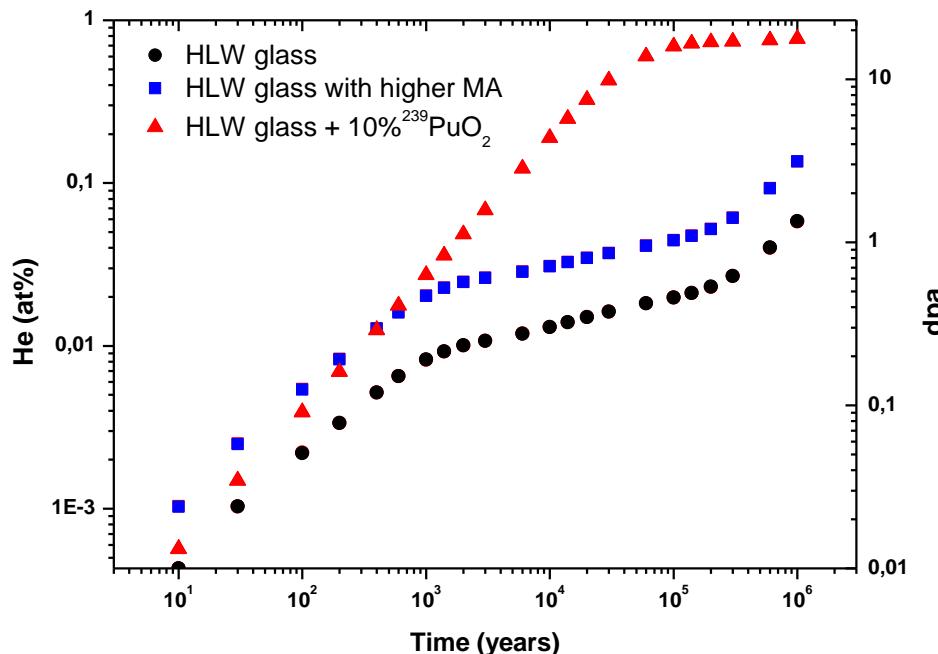
Can irradiation induce a cracking of the material?

- Due to important swelling under irradiation?
- Degradation of the mechanical properties?
- Due to bubble formation (He bubbles generated by alpha decays)



WASTE MECHANICAL DEGRADATION?

Is there any risk of formation of pressurized He bubbles in a nuclear glass?



- Helium incorporation mechanism in the glassy network ?
- Solubility limit ? Helium bubble formation?
- Helium diffusion mechanism?
- Impact of radiation damage on these mechanisms?

WASTE MECHANICAL DEGRADATION?

Irradiation in OSIRIS reactor ($^{10}\text{B}(\text{n},\alpha)^7\text{Li}$)

$[\text{He}]_{\text{max}} : 2,2 \times 10^{20} \text{ at./cm}^3$
 $\text{dpa} : \sim 1-2$



CEA Saclay
OSIRIS

He - METHODOLOGY

He Infusion (P, T)

Equilibrium gas/solid

$[\text{He}]_{\text{max}} : 3,5 \times 10^{18} \text{ at./cm}^3$
 $\text{dpa} : 0$



CEA Marcoule



CEA/DEN/MAR/DE2D/SEVT

$^3\text{He}^+$ implantation :

$[\text{He}]_{\text{max}} : 4,3 \times 10^{21} \text{ at./cm}^3$ (local)
 $\text{dpa} : 11$ (local)

CEMHTI Orléans, LEEL Saclay
NRA $d(^3\text{He}, p)\alpha$



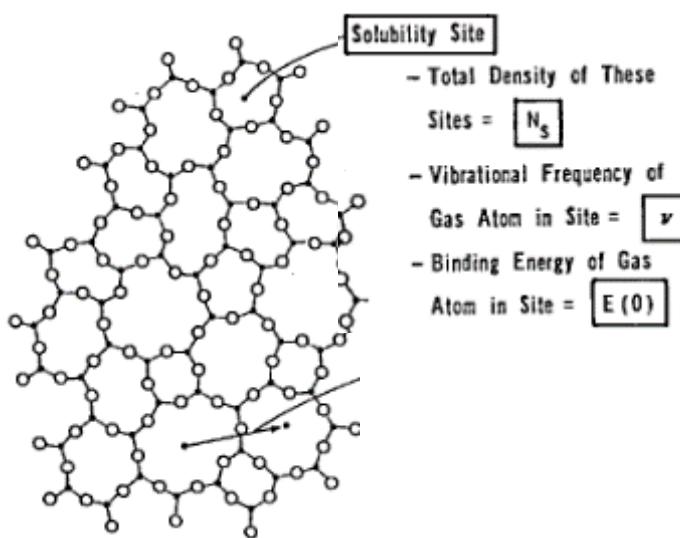
Jannus Orsay, MIAMI Huddersfield
in-situ TEM



C. Jegou

Glass LTB: He incorporation in nuclear glass

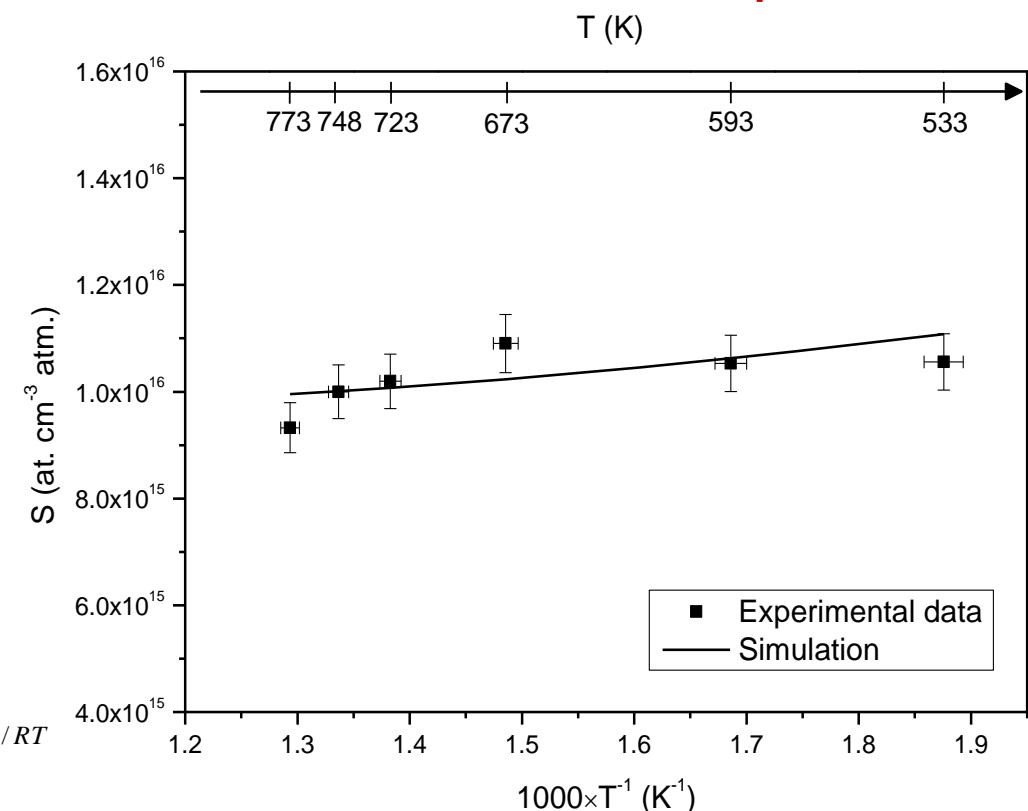
Incorporation of He in the glass free volume



$$S = \frac{C}{p} = \left(\frac{h^2}{2\pi m k_B T} \right)^{3/2} \frac{1}{k_B T} N_s \left(\frac{e^{-h\nu/2k_B T}}{1 - e^{-h\nu/k_B T}} \right)^3 e^{-E(0)/RT}$$

Shackelford J. Appl. Phys. 43 (1972)

He infusion experiments



T. Fares, J. Am. Cer. Soc. 95 (2012) 3854

Density of solubility sites accessible to helium in R7T7 glass: $N_s \sim 3 \times 10^{21}$ sites.cm⁻³

Ns~3 at% to confirm by high pressure infusion experiments at JRC-ITU

Glass LTB: He incorporation in nuclear glass

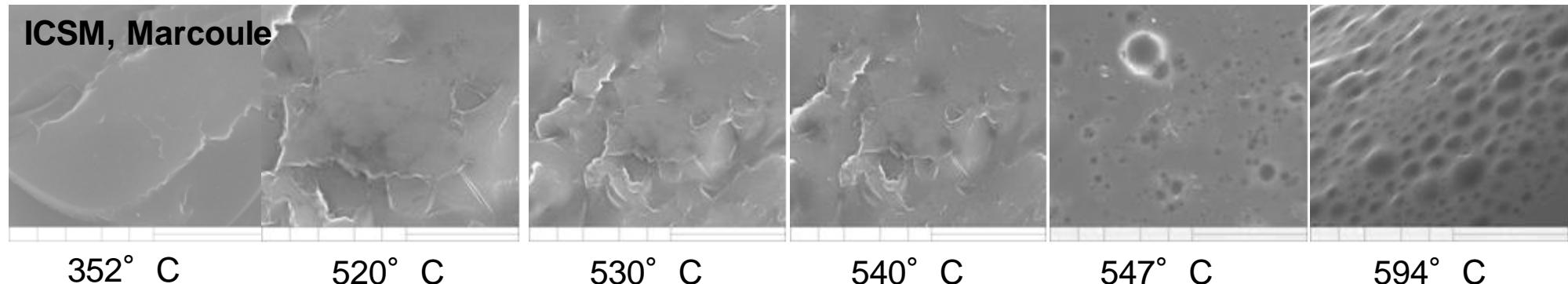
(Ns=3 at%)

Physical state of He ?

Homogeneous generation with equilibrium gas/solide

In-situ ESEM during thermal treatment on He infused nuclear glass ([He]=0.001at%)

ICSM, Marcoule



352° C

520° C

530° C

540° C

547° C

594° C

No damage, dpa=0

T_g

T (° C)

$T < T_g$:

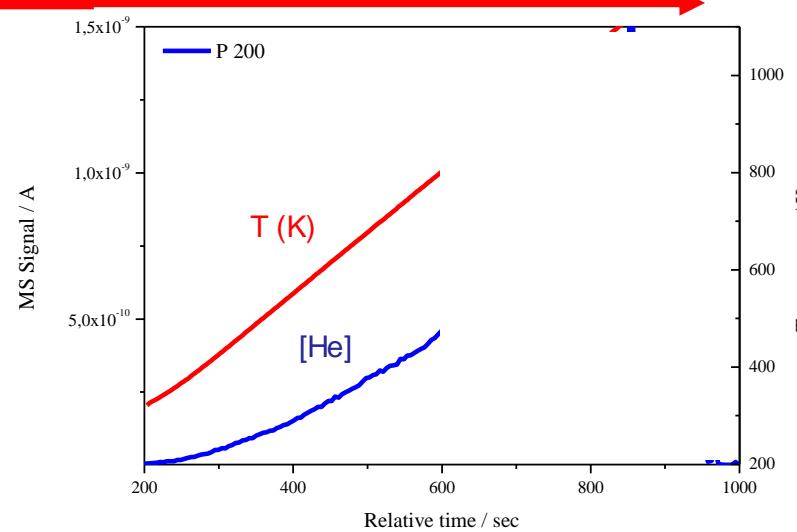
No evolution of microstructure
Helium inside the free volume

$T \sim T_g$:

Dark zones
➤ nucleation of He bubbles ($1 \mu\text{m} < \phi < 10 \mu\text{m}$)

$T > T_g$:

Formation of bubbles
➤ migration of bubbles
➤ release by bursts



➤ $T < T_g$: no glass deformation

→ No bubbles ($\phi < 50 \text{ nm}$)

➤ $T > T_g$: glass deformation is possible

→ Bubble formation ($\phi > 50 \text{ nm}$)

He release
ITU QGames
Workshop – Trieste

Glass LTB: He incorporation in nuclear glass

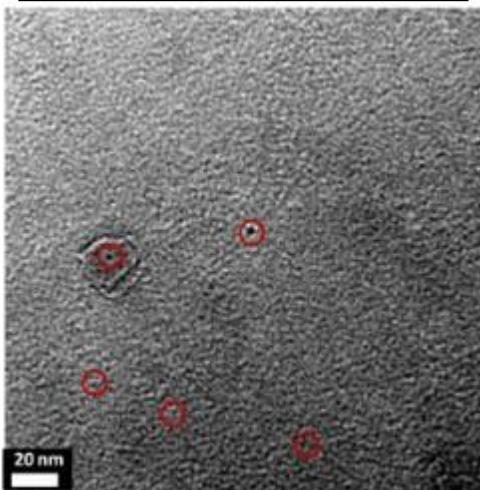
(Ns=3 at%)

Physical state of He ?

Heterogeneous generation at -130°C with damage

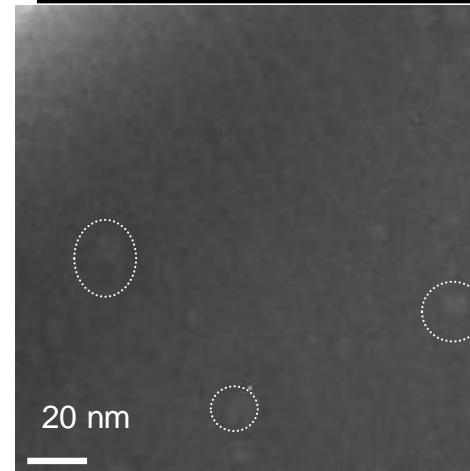
In-situ TEM during He implantation: Jannus Orsay, MIAMI Huddersfield

10 keV, Jannus

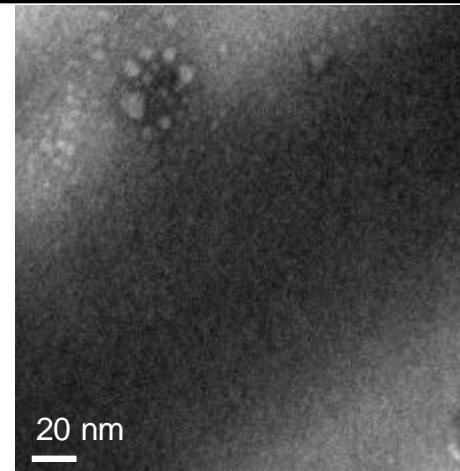


$\sim 10^{15}$ (0.1 at%, 0.1dpa)

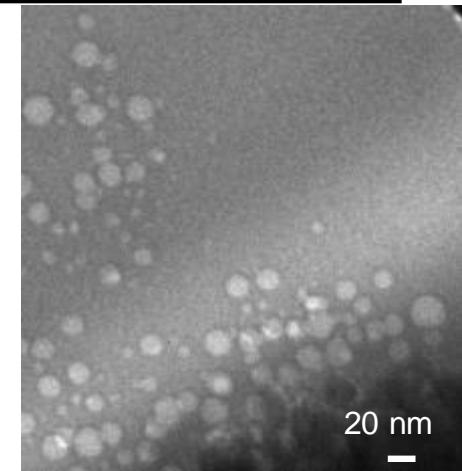
6 keV, MIAMI



$\sim 2 \times 10^{16}$ (2 at%, ~2dpa)



$\sim 4 \times 10^{16}$ (4 at%)



$\sim 9 \times 10^{16}$ (9 at%)

➤ [He] < Ns:

First bubbles observed at ~0,1 at%

Weak evolution of size and density

Ns (~3 at%)

➤ [He] > Ns:

Increase of the bubble size

F (He.cm⁻²)

Increase of the bubble density

Glass LTB: He incorporation in nuclear glass

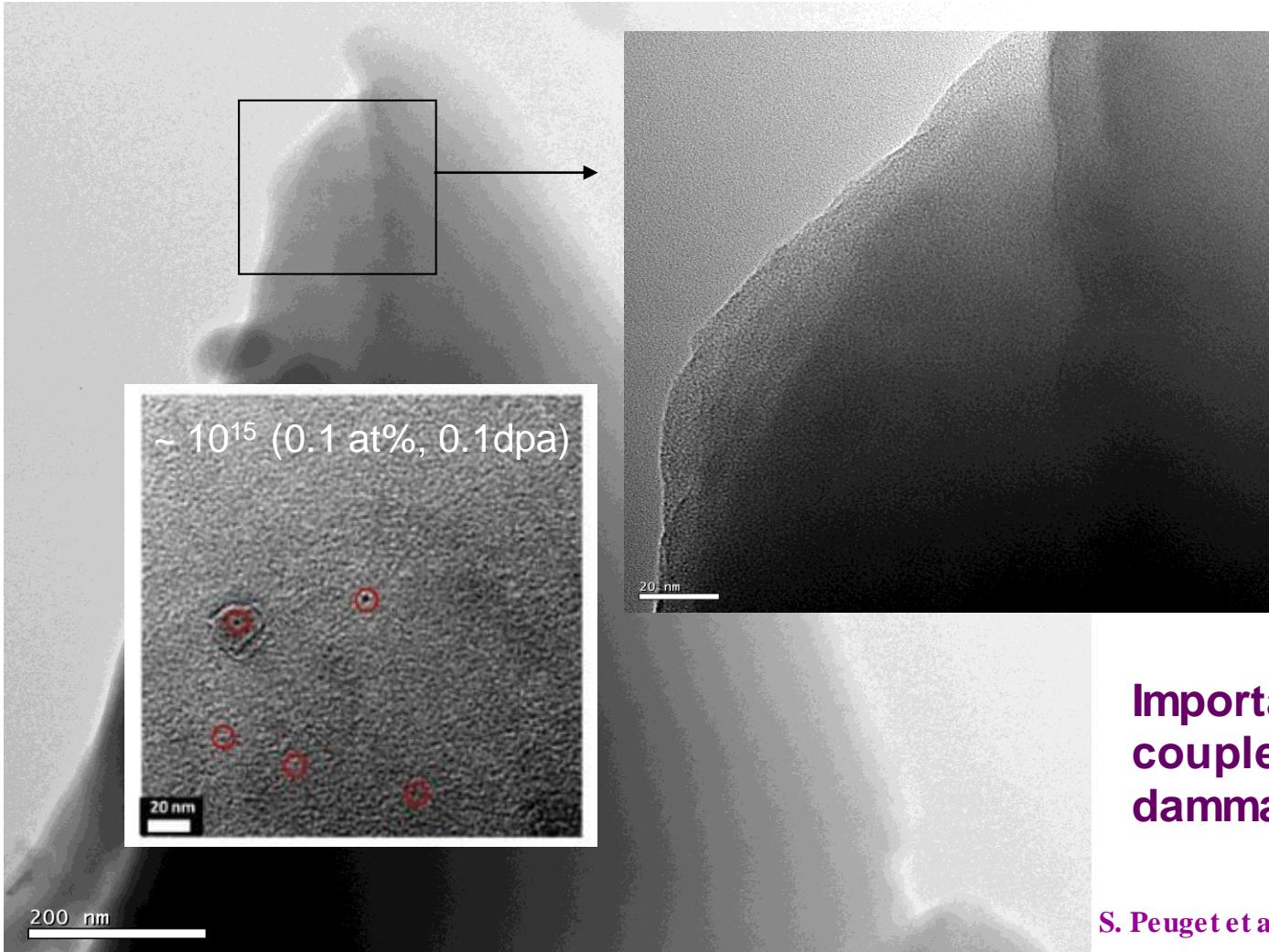
(Ns=3 at%)

Physical state of He ?

Homogeneous generation at room temperature

^{244}Cm and OSIRIS irradiated glass $^{10}\text{B}(\text{n},\alpha)^7\text{Li}$: $\sim 2 \times 10^{20}$ He/cm³ (0.2 at%, ~1dpa)

(~ 10⁶ ans de stockage)

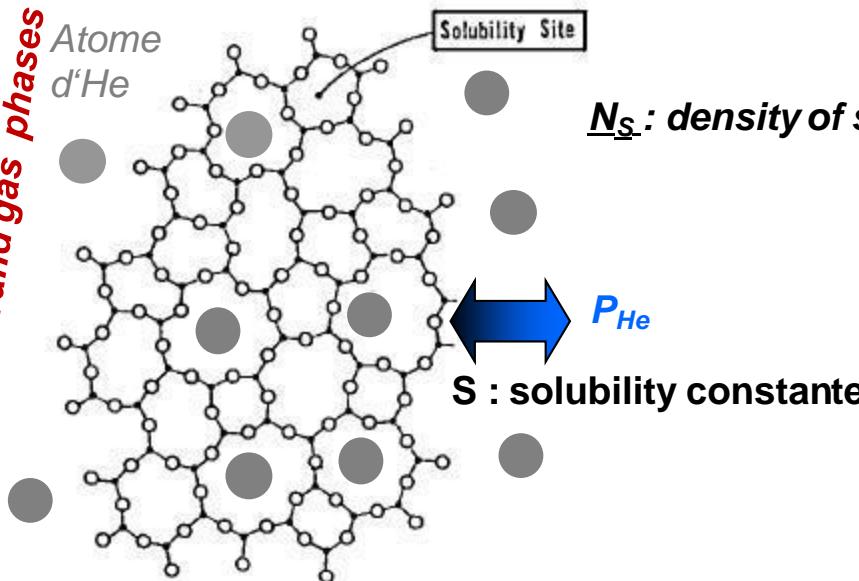


S. Peugeot et al. NIMB 327 (2014) 22-28

TEM (ITU Karlsruhe)

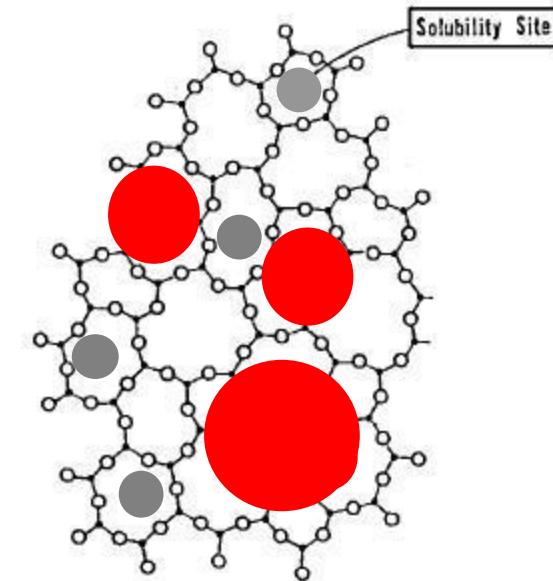
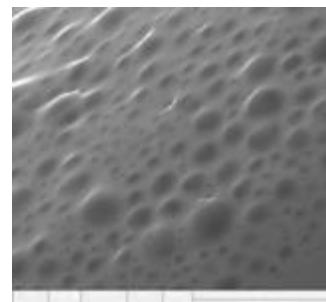
Glass LTB: He incorporation in nuclear glass, summary

Solubilisation: equilibrium between solid and gas phases



- Helium solubilized in the glass free volume
- Bubble formation for $T > T_g$ ($\sim 550^\circ\text{C}$)

594°C



Incorporation (desintegration, implantation) : out of equilibrium

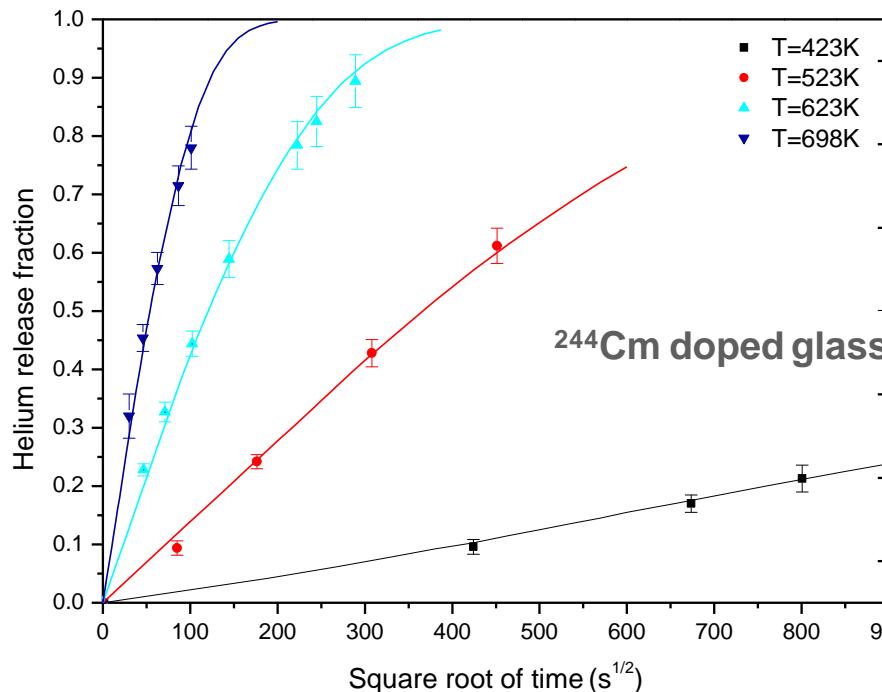
1. $[\text{He}] < N_s$
 - He solubilized in the free volume
 - Weak probability for He bubble formation
 - Importance of temperature and damage?

Disposal conditions: in progress

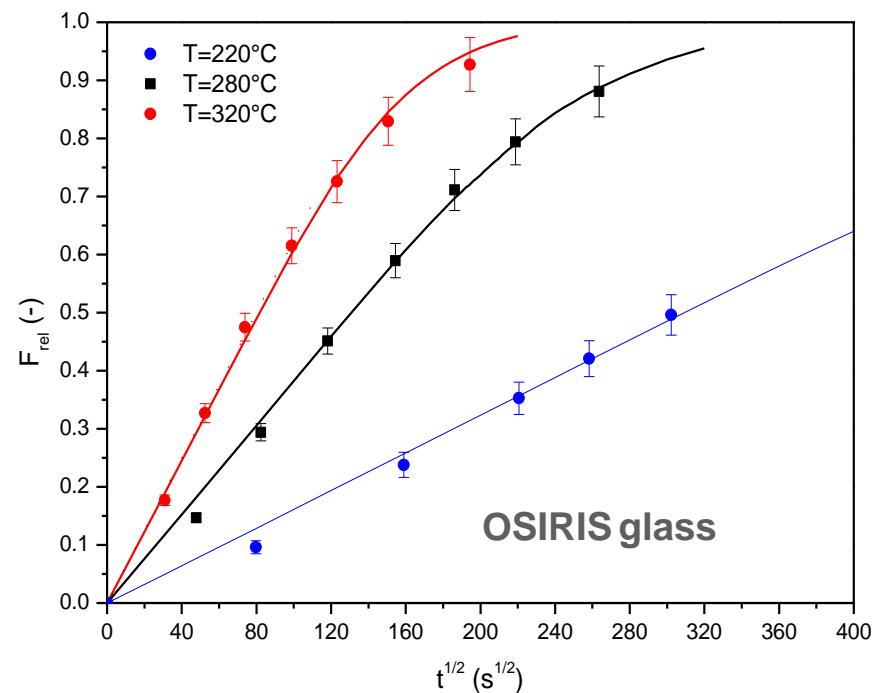
2. $[\text{He}] > N_s$
 - Bubble formation
 - Stress state ?

Glass LTB: He diffusion in nuclear glass

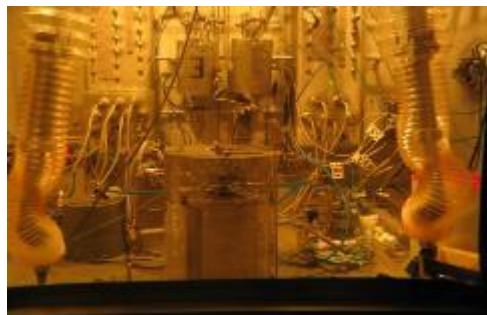
Helium release experiment: determination of D_{He}



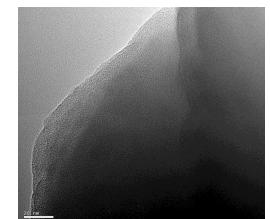
T. Fares, J. Nucl. Mater. 416 (2011) 236



T. Fares, Thèse Univ. Montpellier II (2011)

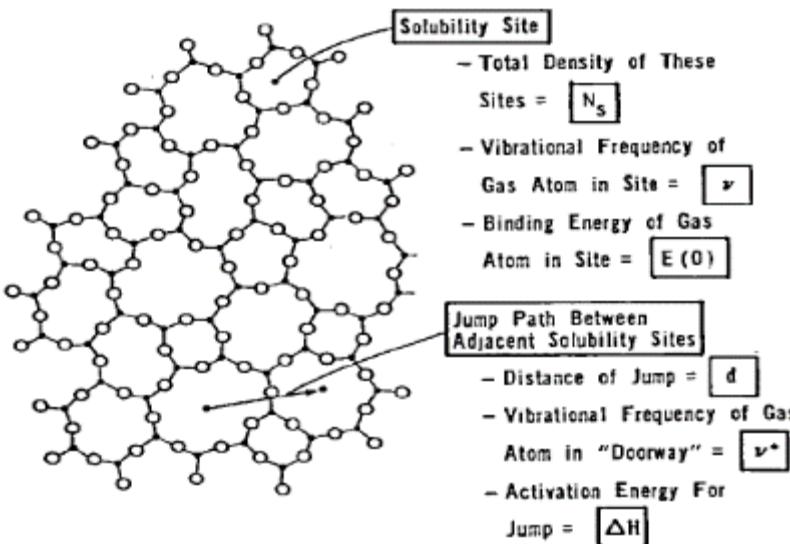


$$\begin{cases} \frac{\partial C(x, y, z, t)}{\partial t} = A(x, y, z, t) + D\Delta C(x, y, z, t) \\ A(x, y, z, t) = -\frac{dN_{(^{244}Cm)}}{dt} = A_0(x, y, z)\exp(-\lambda_{(^{244}Cm})t) \end{cases}$$



**Only a solubilized helium population is needed to fit the data
No need to introduce helium bubbles in the model ...**

Glass LTB: He diffusion in nuclear glass



All data are in quite good agreement

$E_a \sim 0.6$ eV

Diffusion through the glass free volume

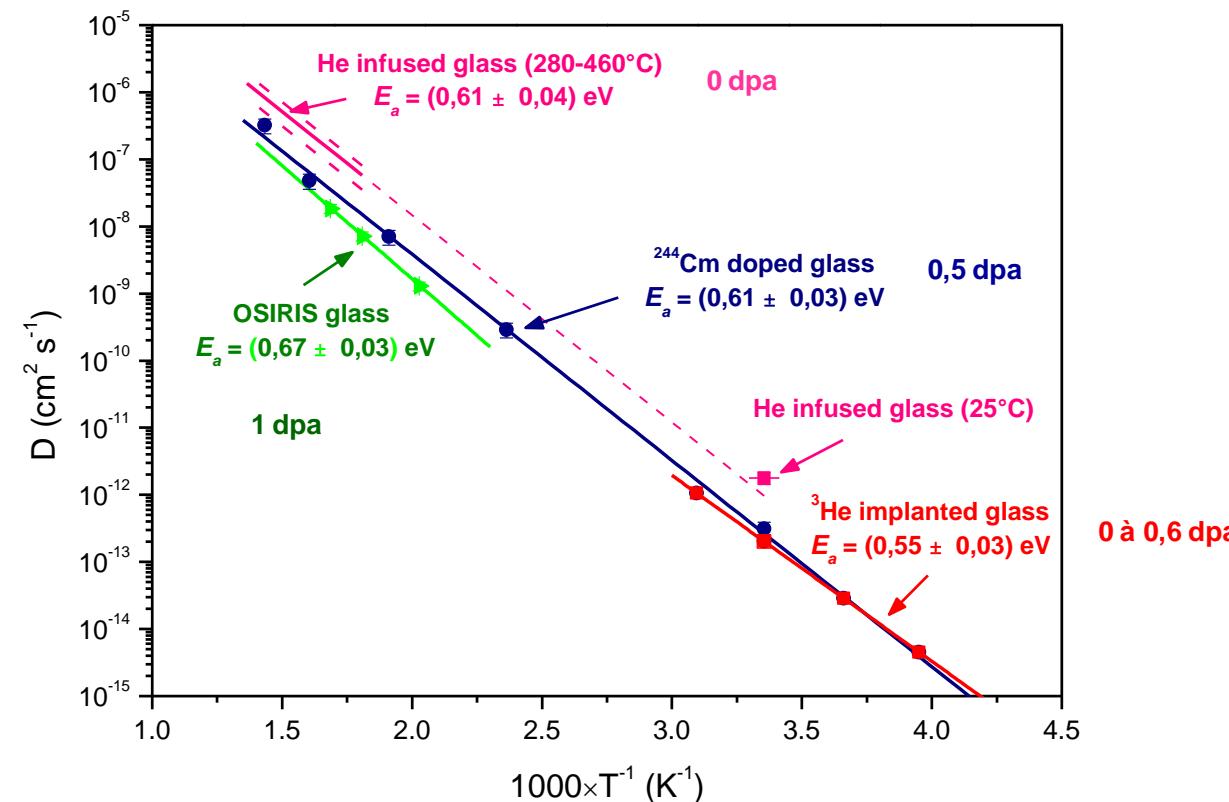
No significant effect of the glass damage on D_{He}

$$D = \frac{1}{6} \cdot d^2 \cdot \frac{\nu^3}{\nu^{*2}} \cdot e^{-\Delta H/K_B T} = D_0 \cdot e^{-\Delta H/K_B T}$$

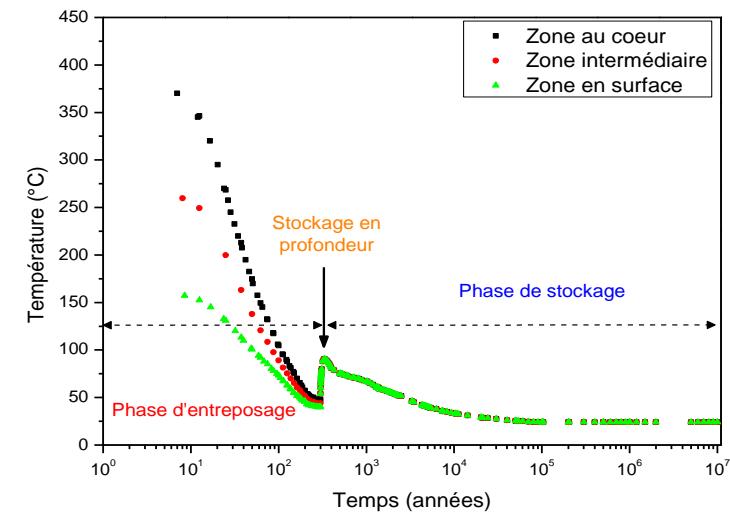
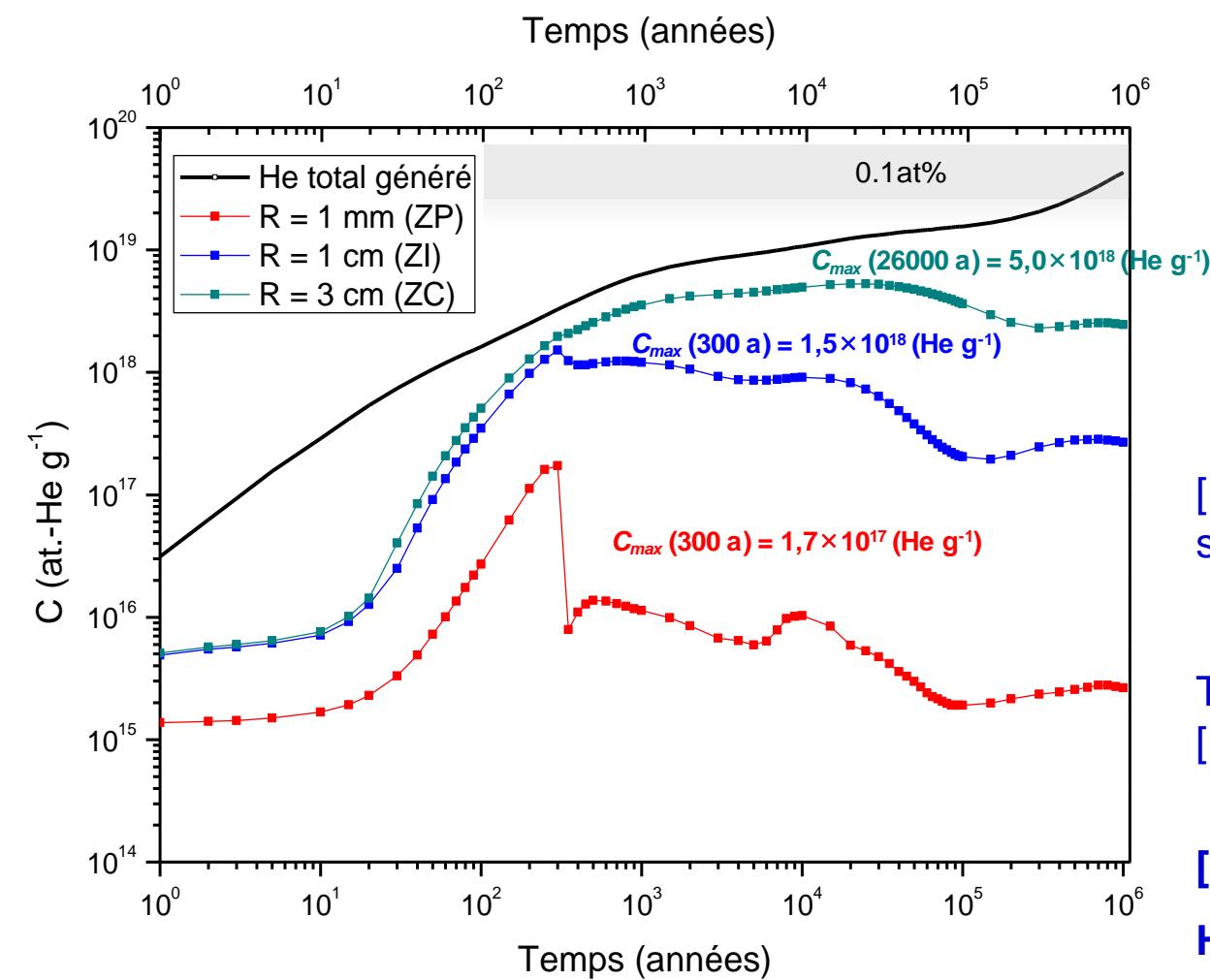
J. Shackelford J. Appl. Phys. 43 (1972)

T. Fares, Thèse Univ. Montpellier II (2011)

F. Chamssedine J. Nucl. Mater. 400 (2010) 175



Glass LTB: modeling of He migration in a canister



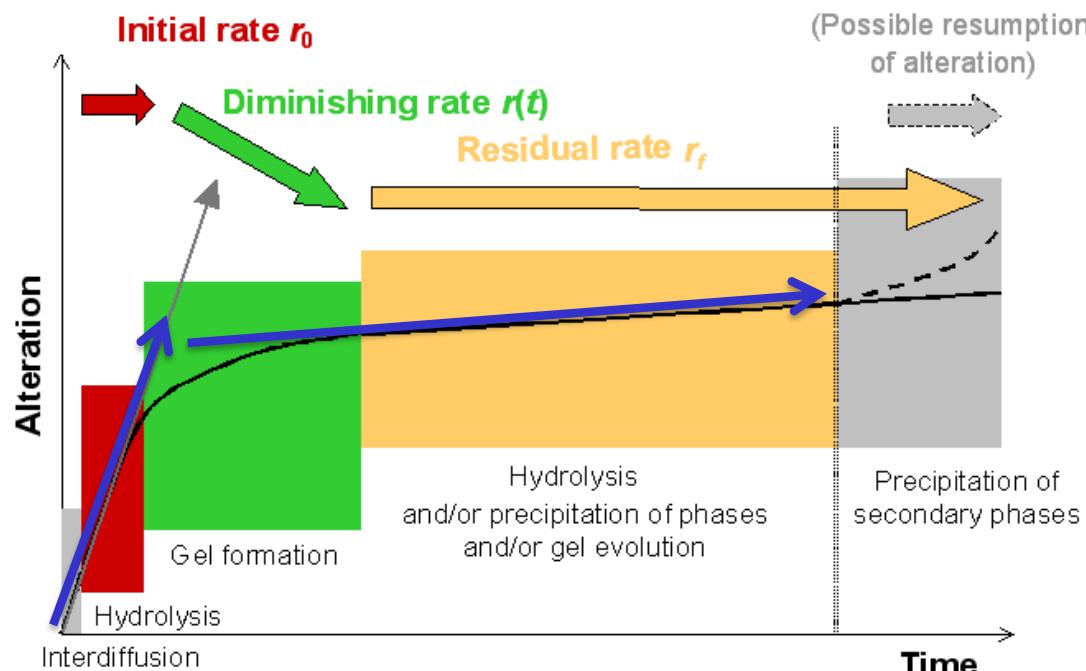
$[He] = f(t)$ is strongly dependent of bloc size

Thermal diffusion can strongly reduce $[He]_{\max}$

$$[He]_{\max} < Ns / 100$$

He bubble formation is improbable

EFFECTS OF RADIATION ON THE LEACHING BEHAVIOR?



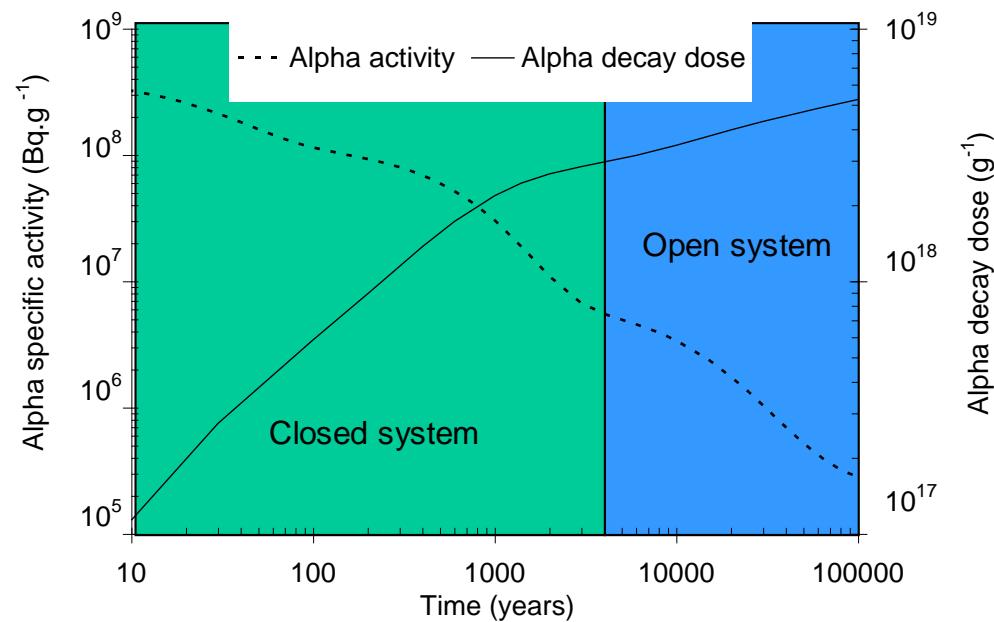
Two main steps to study:

1. Impact on r_0
2. Impact on residual rate, r_r

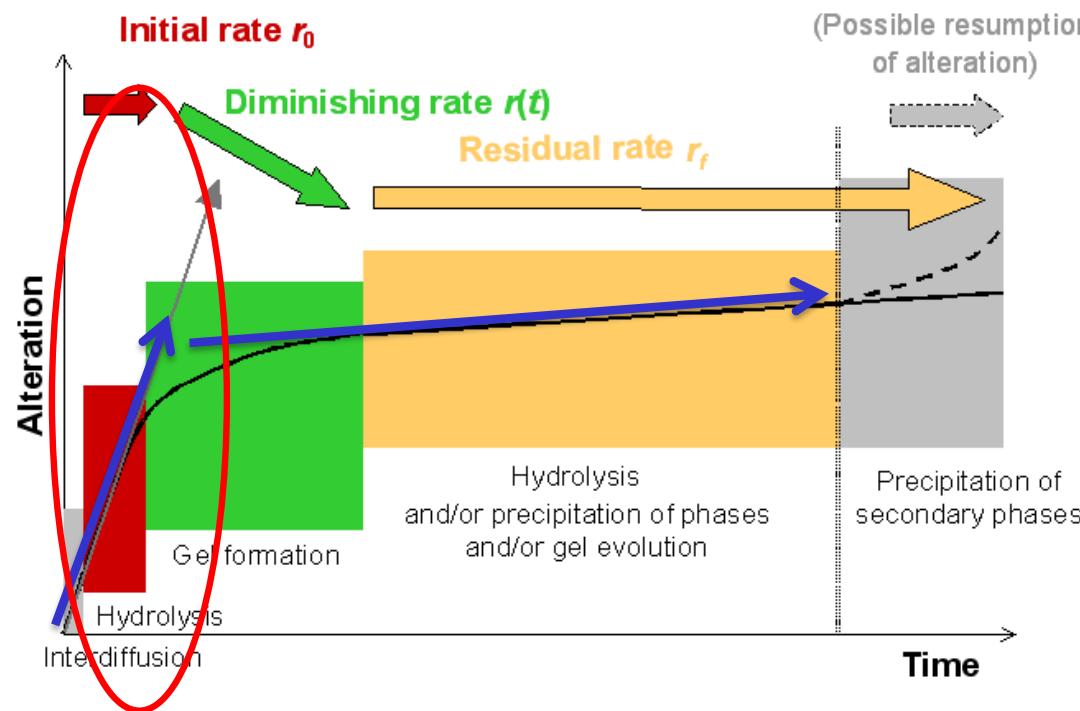
Two parameters to study:

1. Dose rate
2. Dose

Experiments on radioactive and externally irradiated SON68 glasses



EFFECTS OF RADIATION ON THE LEACHING BEHAVIOR?



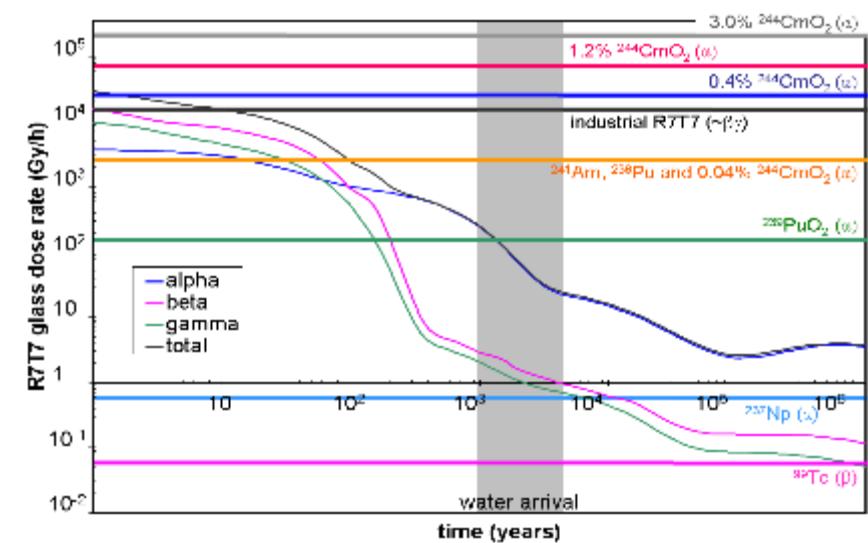
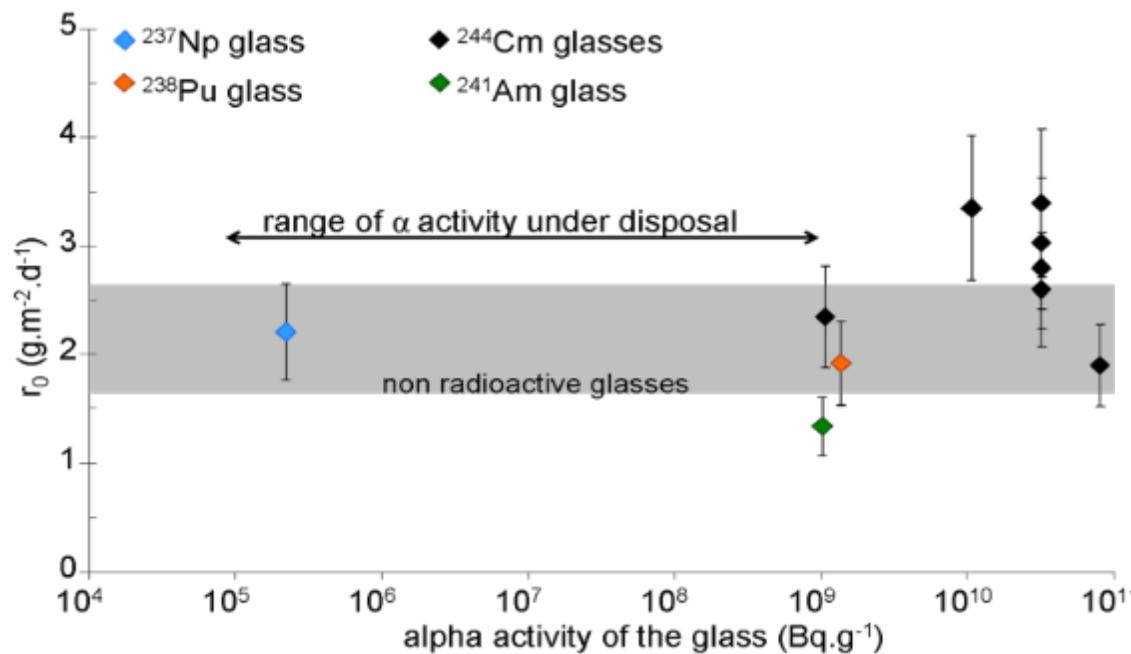
1. Impact on r_0

Two parameters to study:

1. Dose rate
2. Dose

ALPHA DOSE RATE on initial rate R₀?

Soxhlet-mode dynamic leach tests or short static tests (low SA/V ratio) on different α doped glasses^{1,2}



No significant impact of α activity on R_0

¹S. Peugeot et al., JNM 362 (2007)

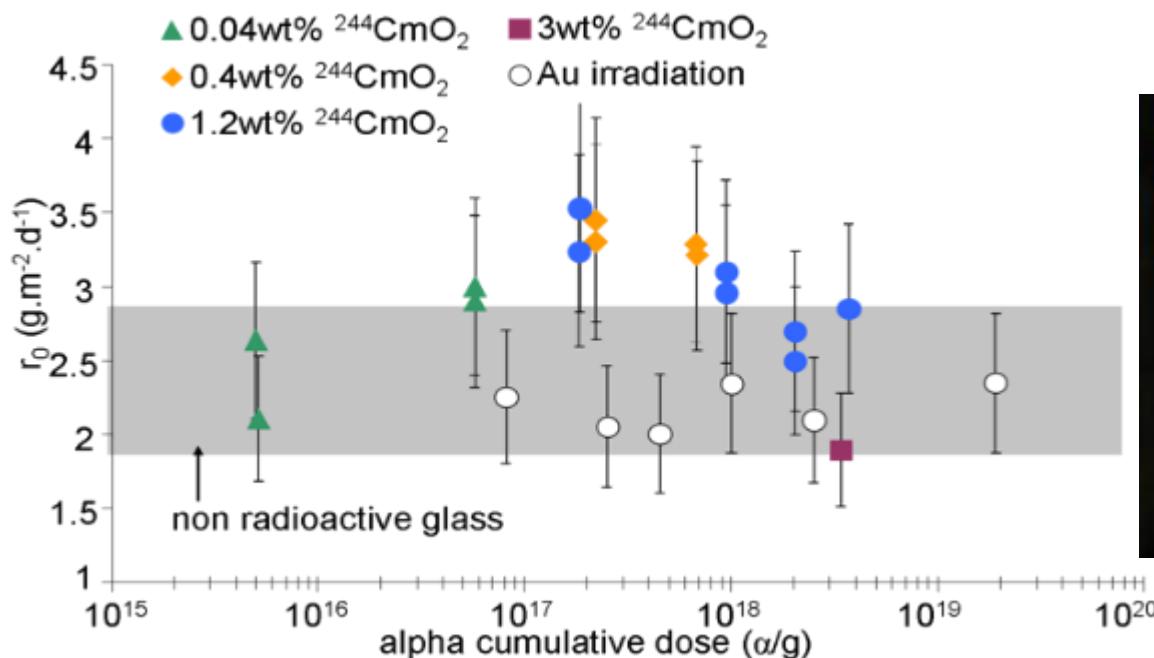
²T. Advocat et al., JNM 298 (2001)

ALPHA CUMULATIVE DOSE on initial rate R_0 ?

Soxhlet-mode dynamic leach tests (100°C, 1 month) on¹:

- ^{244}Cm -doped glasses
- non radioactive glasses previously irradiated by Au ions

R_0 determined from solution analysis



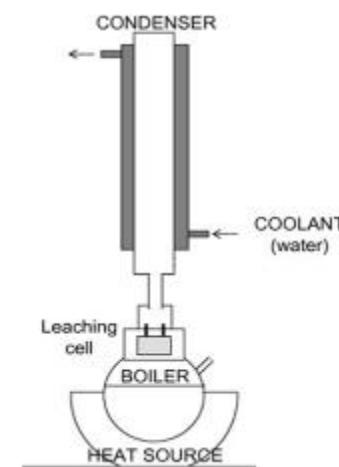
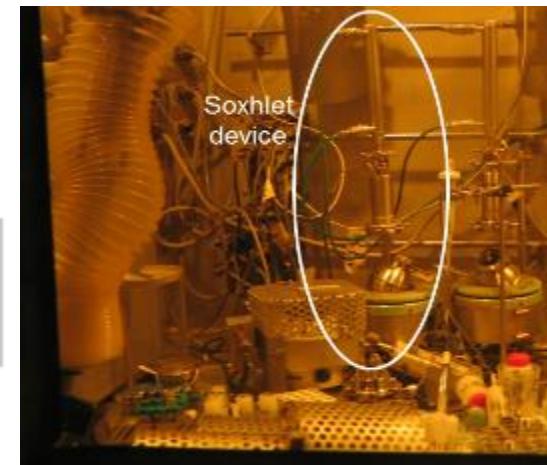
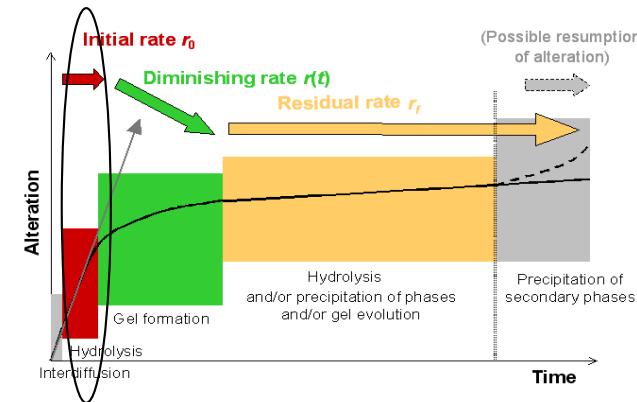
No significant ballistic impact of α cumulative dose on R_0

In agreement with data from literature: no impact² or less than factor two³

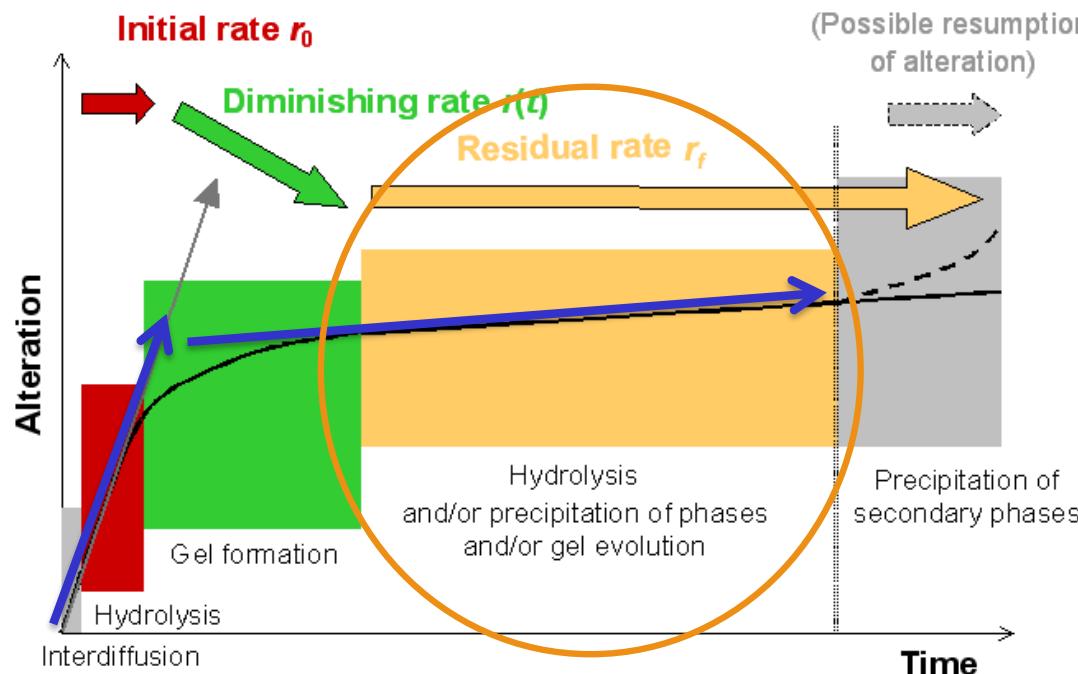
¹S. Peugeot et al., JNM 362 (2007)

²D.M. Wellman et al., JNM 340 (2005)

³W.G. Burns et al., JNM 107 (1982)



EFFECTS OF RADIATION ON THE LEACHING BEHAVIOR?



2. Impact on residual rate, r_r

Two parameters to study:

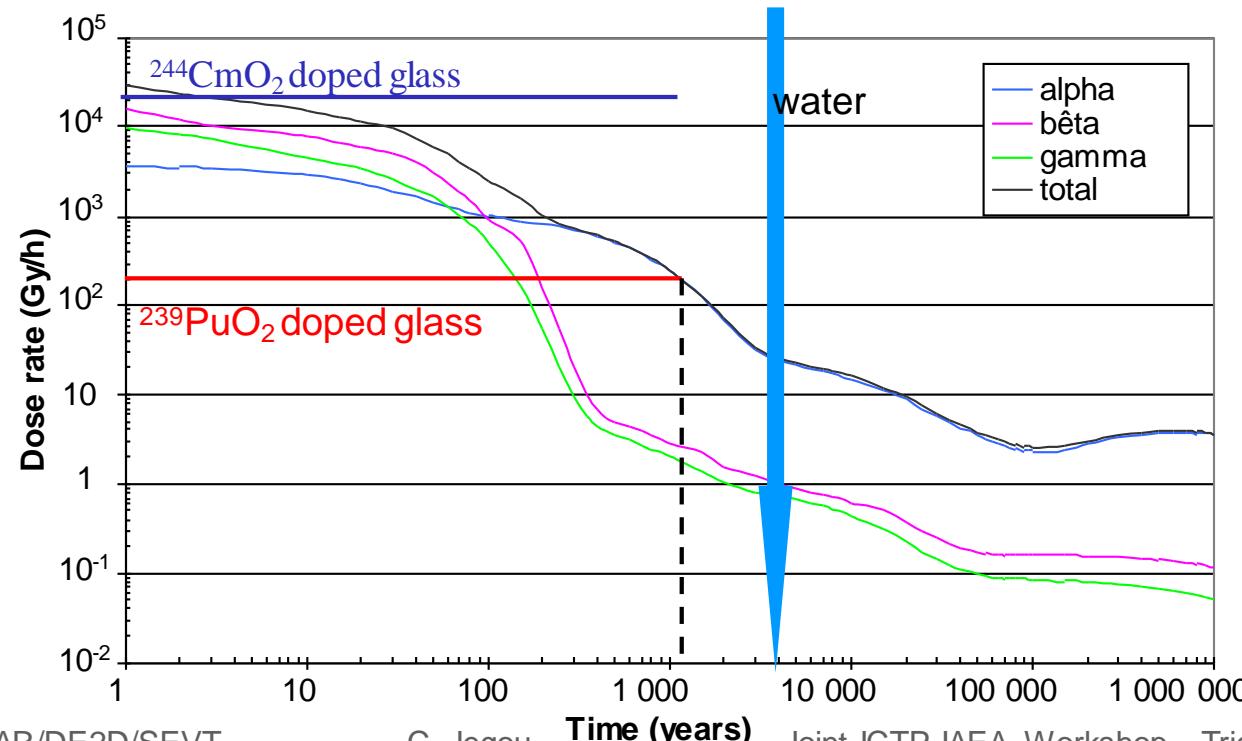
- 1. Dose rate**
- 2. Dose**

ALPHA DOSE RATE on residual rate ?

✓ Glass characteristics:

- 2 SON68 doped glasses

α-doped glasses	Dose rate (Gy/h)	Dose cumulated before leach exp
0.85 wt% of $^{239}\text{PuO}_2$	150	$\sim 3.7 \times 10^{16} \alpha/\text{g}$
0.4 wt% of $^{244}\text{CmO}_2$	23,500	$\sim 3 \times 10^{18} \alpha/\text{g}$



EXPERIMENTAL

- ✓ Glass samples ground & sieved → 63 – 125 µm fraction powder

$$S_{\text{BET}} \sim 645 \text{ cm}^2 \cdot \text{g}^{-1}$$

- ✓ SON68 reference glass powder → reference experiment

- ✓ Glass alteration

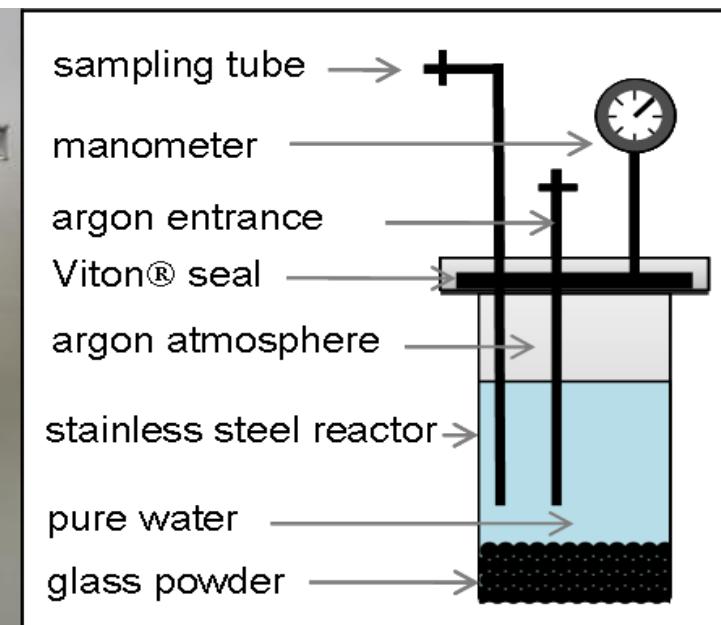
- **~10 g of glass powder**
- **~300 mL of pure water**

} S/V = 20-25 cm⁻¹

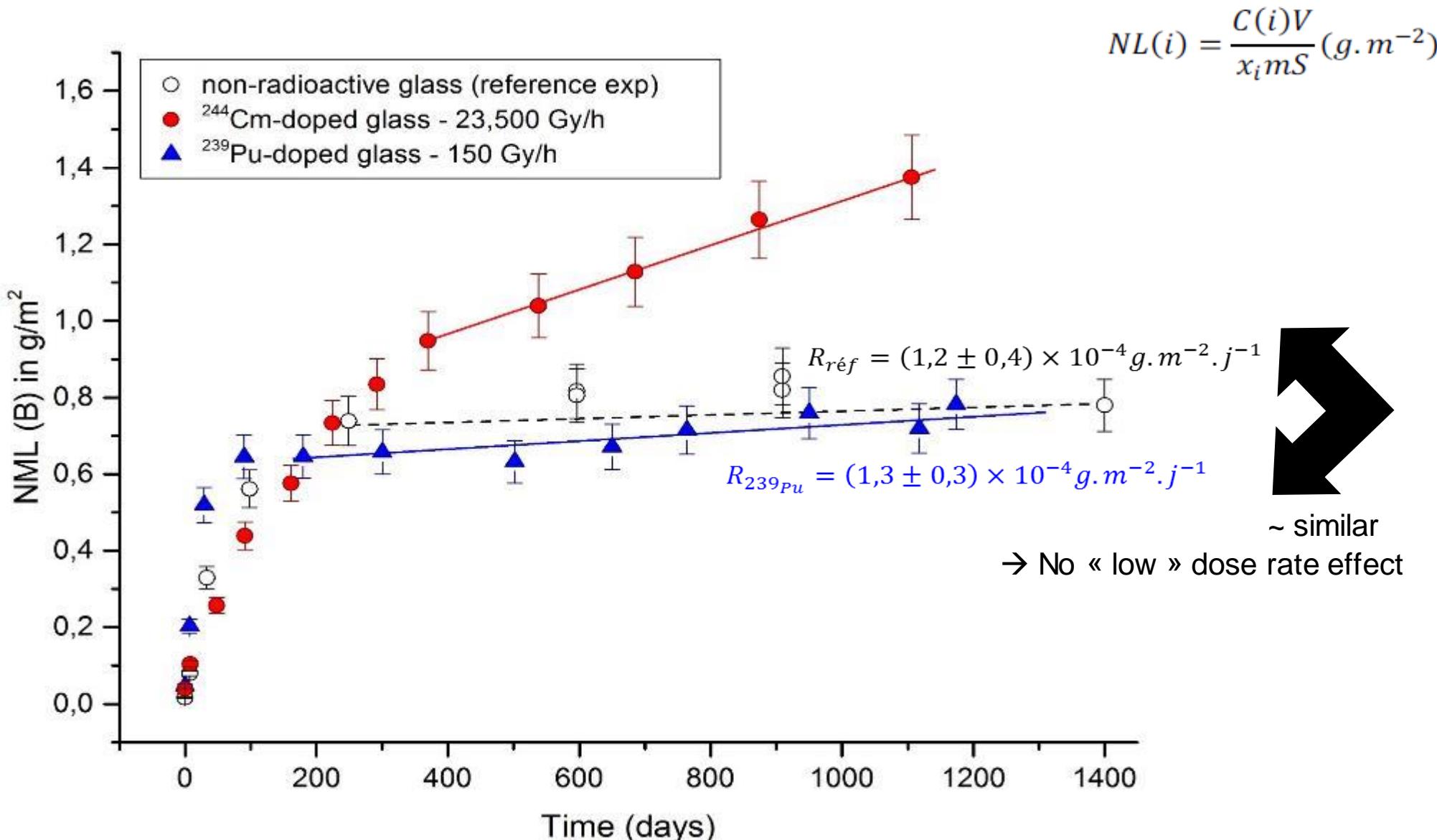
- **4 Bar Ar overpressure**
- **90°C**
- **> 3 years**

- ✓ Analyses:

- **Regular leachate samples**
(ICP-AES, ionic chromatography, pH, Eh, radiochemistry)
- **Solid analyses after leach test**
(SEM, TEM, EDX)

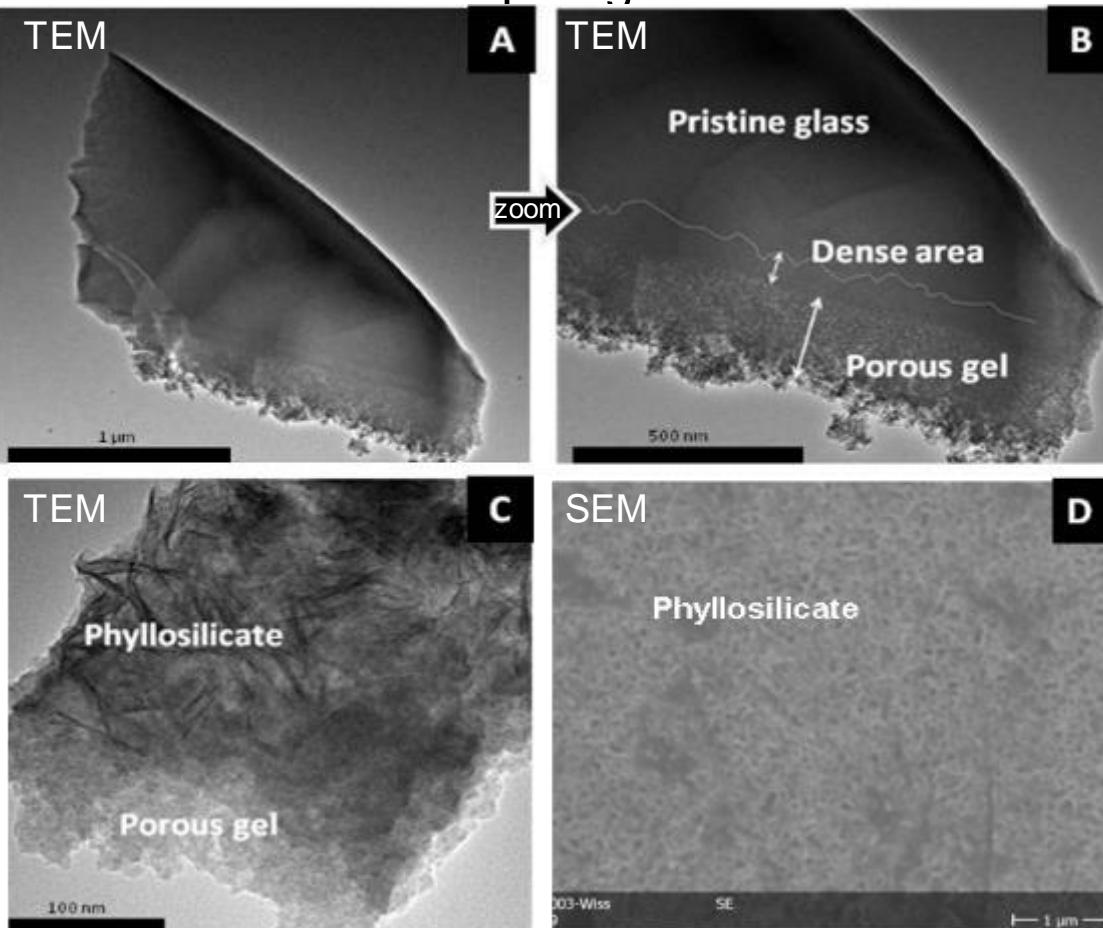


RESULTS: tracer evolution (B, Na, Li)



RESULTS: solid analyses

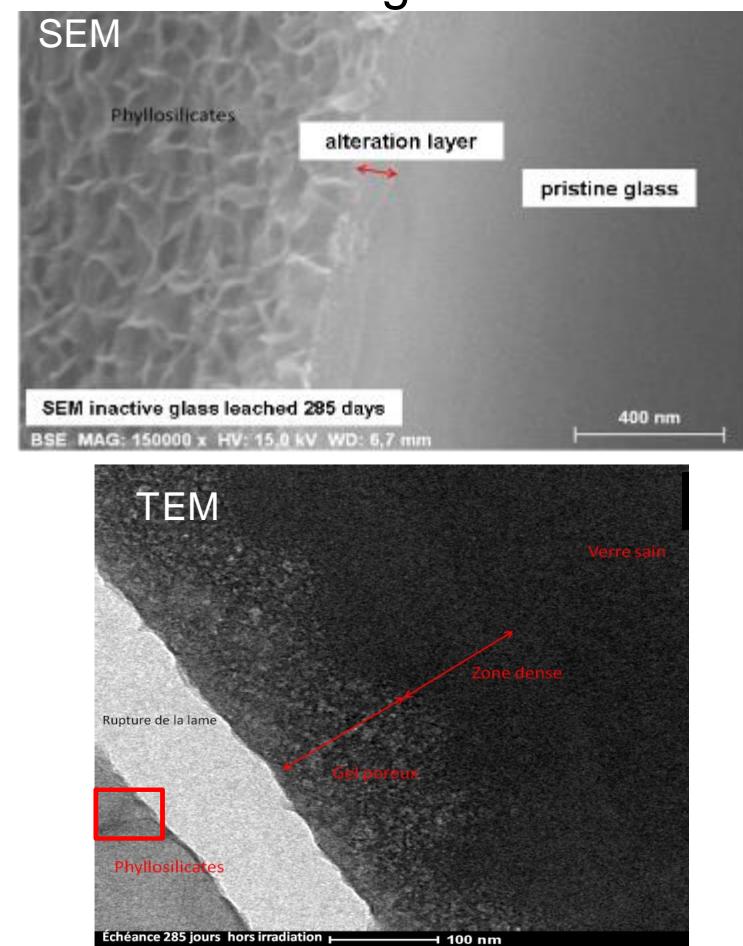
Pu-doped glass



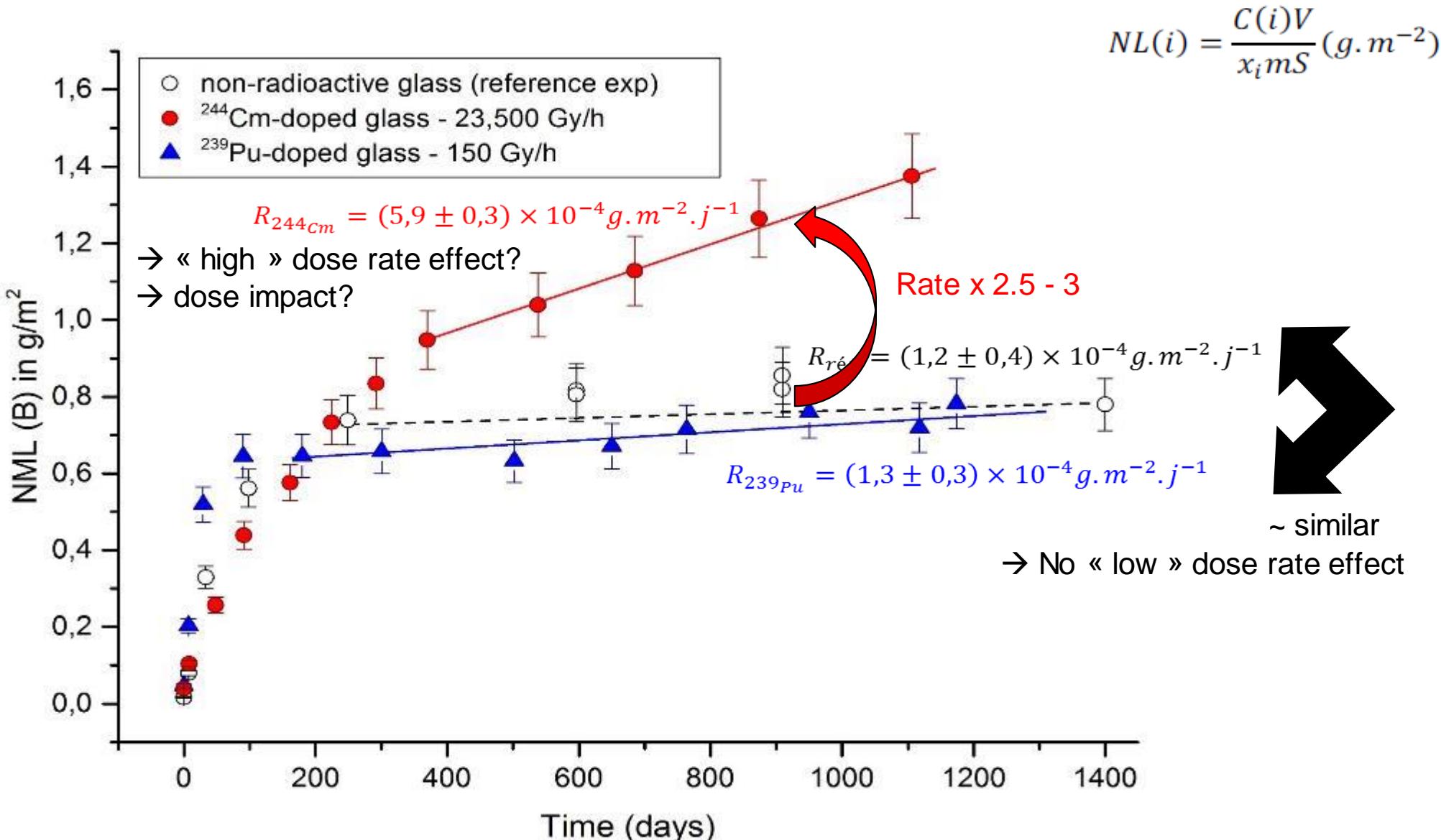
Alteration layer formed under radiation for ^{239}Pu -doped glass:

- Similar to non radioactive one
- Thickness : similar to those calculated from solution releases (300 nm)

Inactive glass



RESULTS: tracer evolution (B, Na, Li)



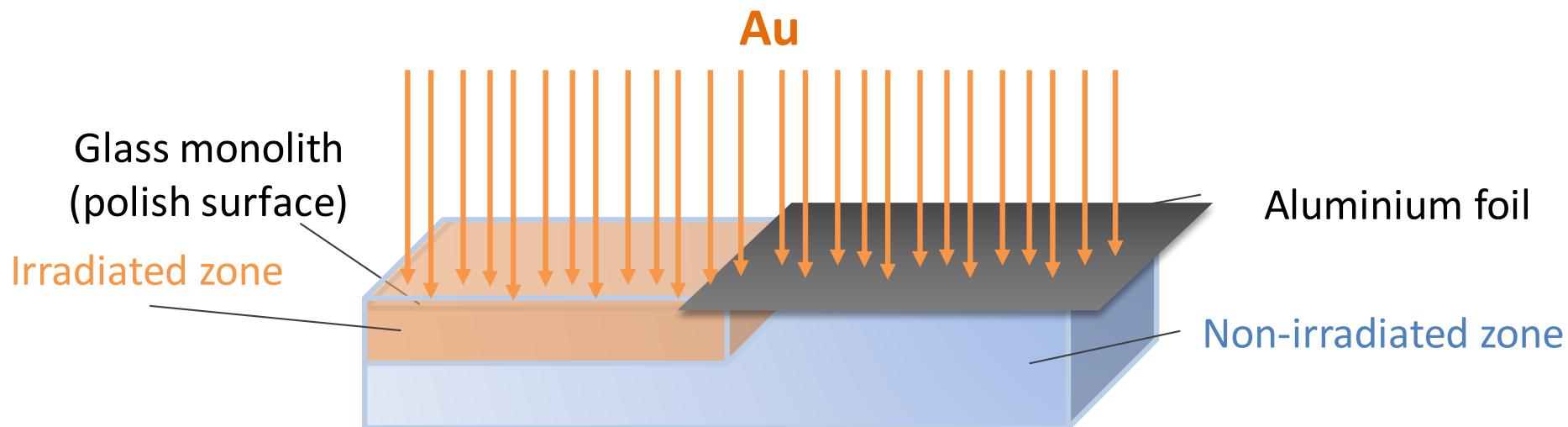
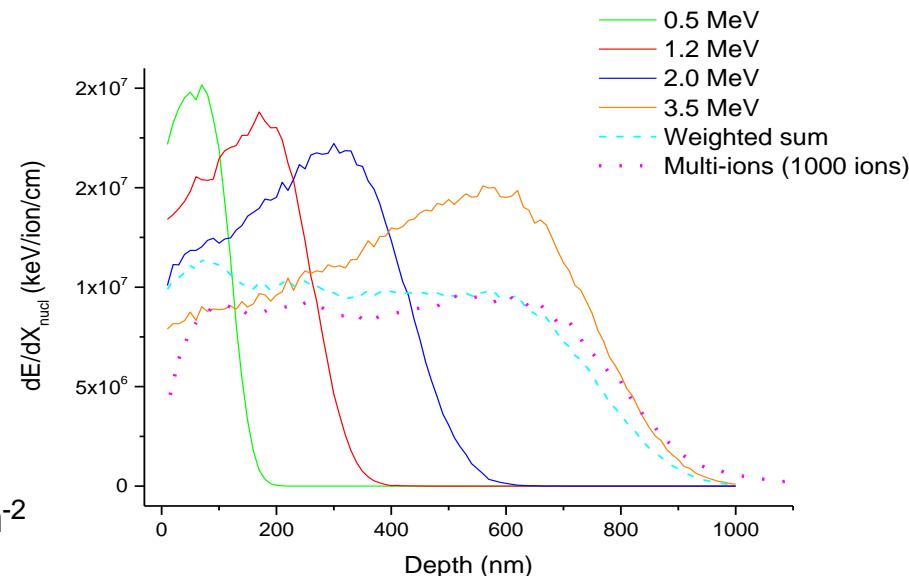
Glasses and irradiation conditions

Simple glass composition

- ISG (International Simple Glass)

Multiple-energy gold ion irradiation

- 0.5 – 3.5 MeV → ≈ constant ballistic damage
- Energy deposition < ion track formation
- Wide range of fluences: $1.9 \times 10^{12} \rightarrow 5.5 \times 10^{14}$ ions.cm $^{-2}$



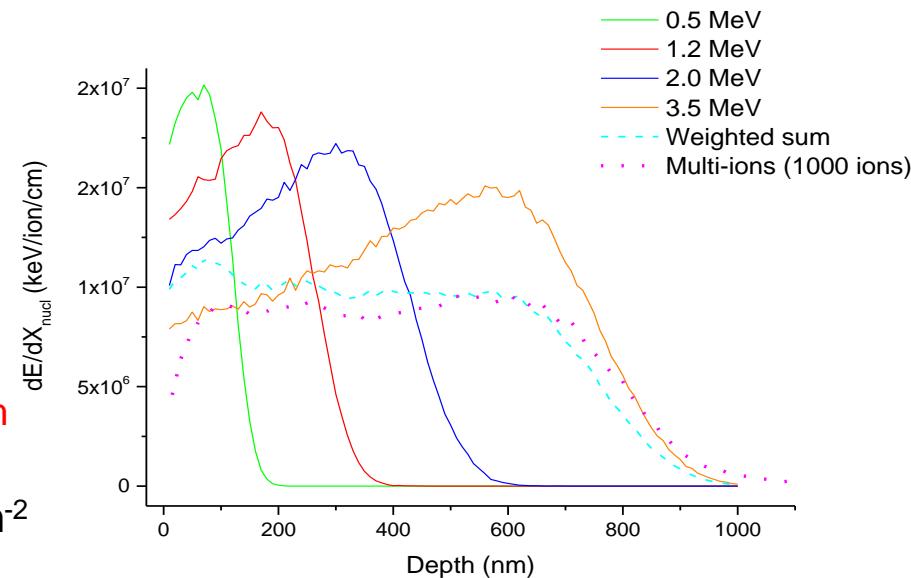
Glasses and irradiation conditions

Simple glass composition

- ISG (International Simple Glass)

Multiple-energy gold ion irradiation

- 0.5 – 3.5 MeV → ≈ constant ballistic damage 1000 nm
- Energy deposition < ion track formation
- Wide range of fluences: $1.9 \times 10^{12} \rightarrow 5.5 \times 10^{14}$ ions.cm $^{-2}$
- Ballistic dose: 0.7 → 215 MGy

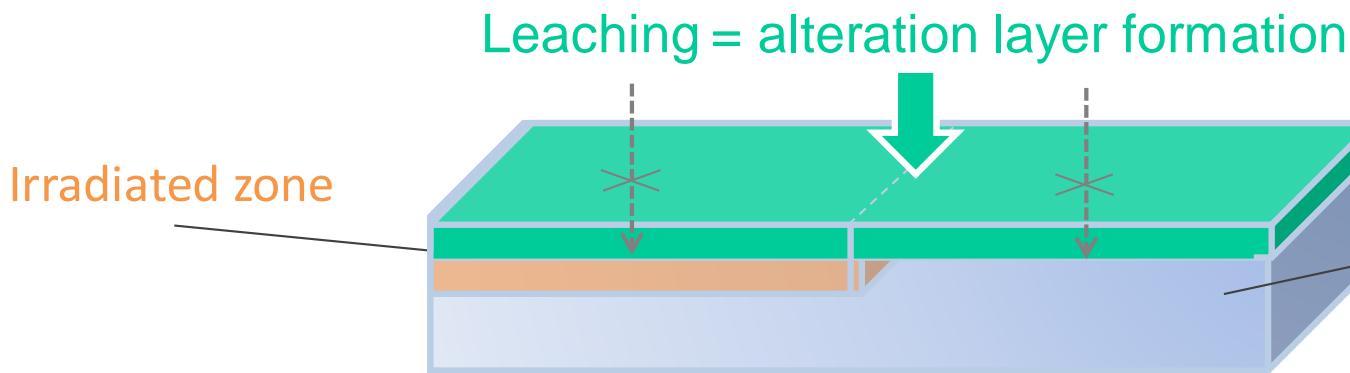


Simple glass composition

- ISG (International Simple Glass)

Multiple-energy gold ion irradiation

- 0.5 – 3.5 MeV → ≈ constant ballistic damage
- Energy deposition < ion track formation
- Wide range of fluences:
- Ballistic dose: 0.7 → 215 MGy

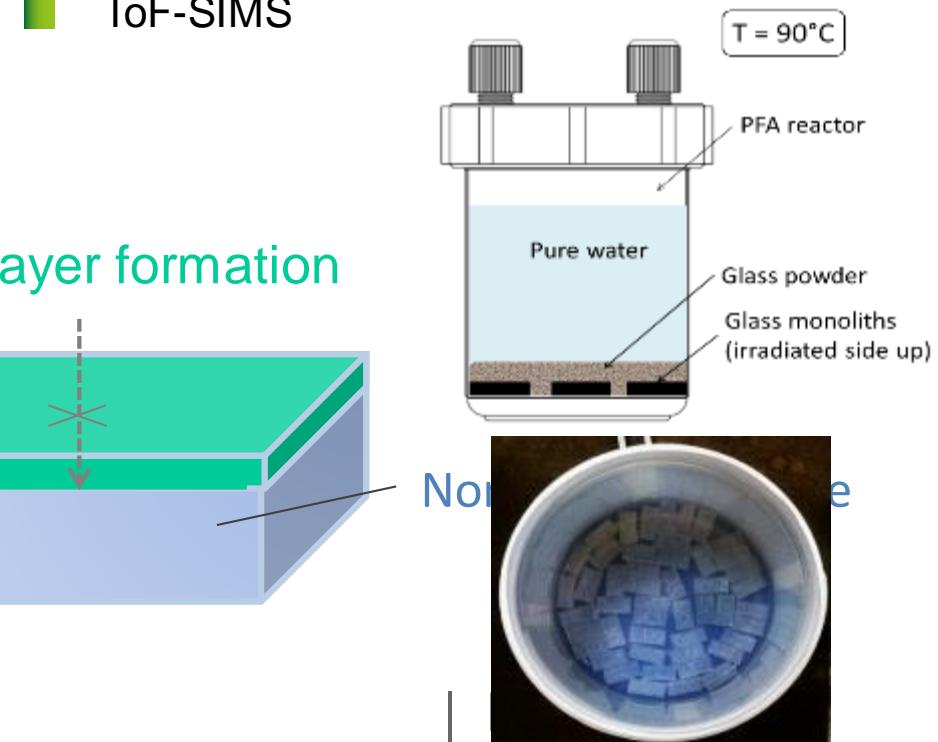


Alteration protocol

- Savillex, 200 cm⁻¹, 90°C
- Glass monoliths sampled regularly

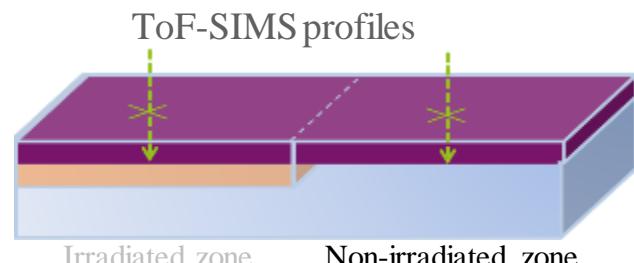
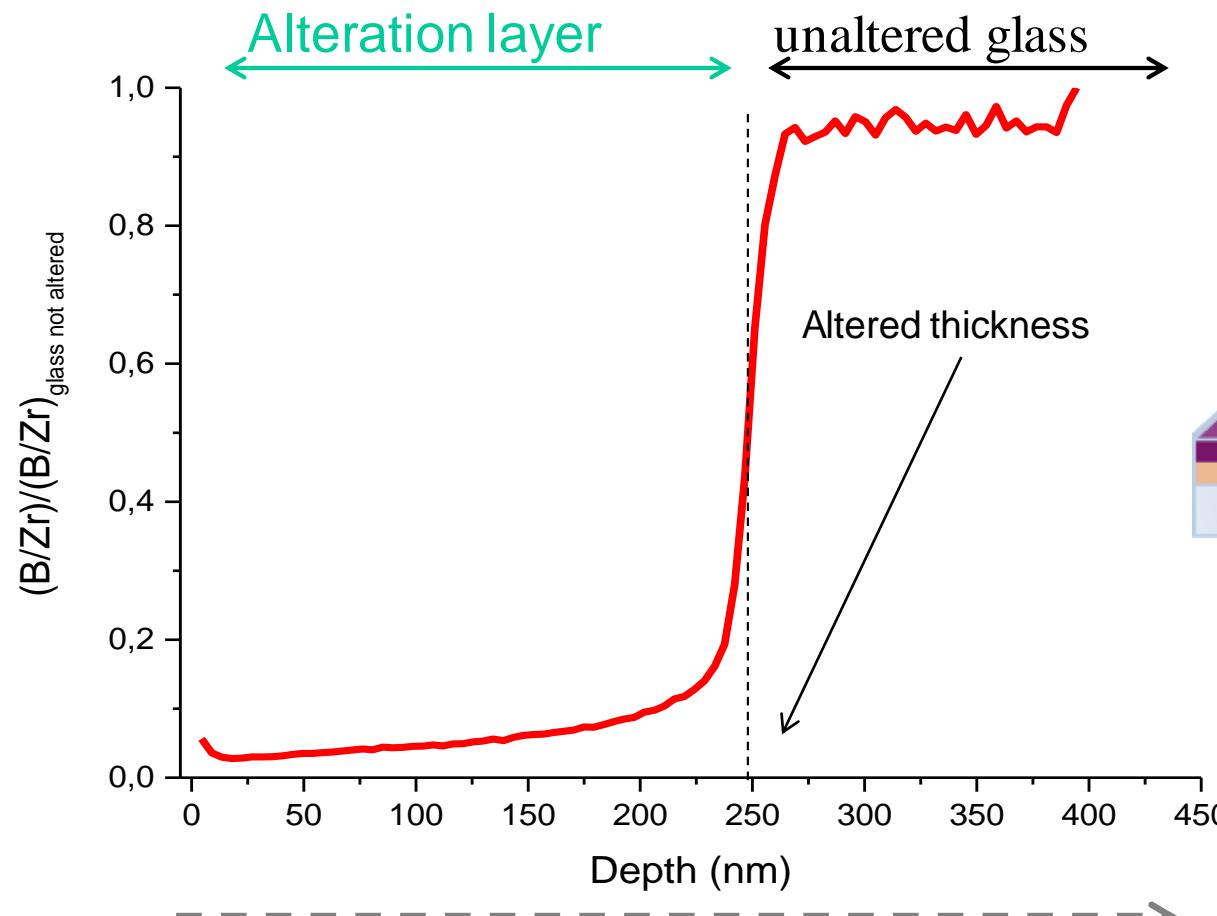
Altered layer characterization

- ToF-SIMS



TOF-SIMS PROFILES

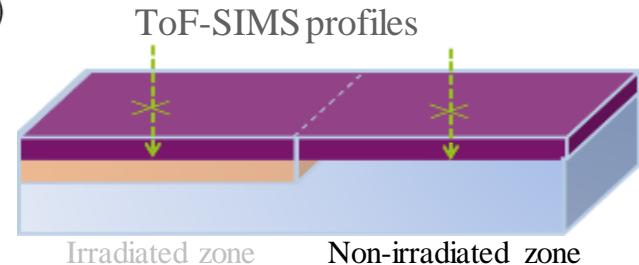
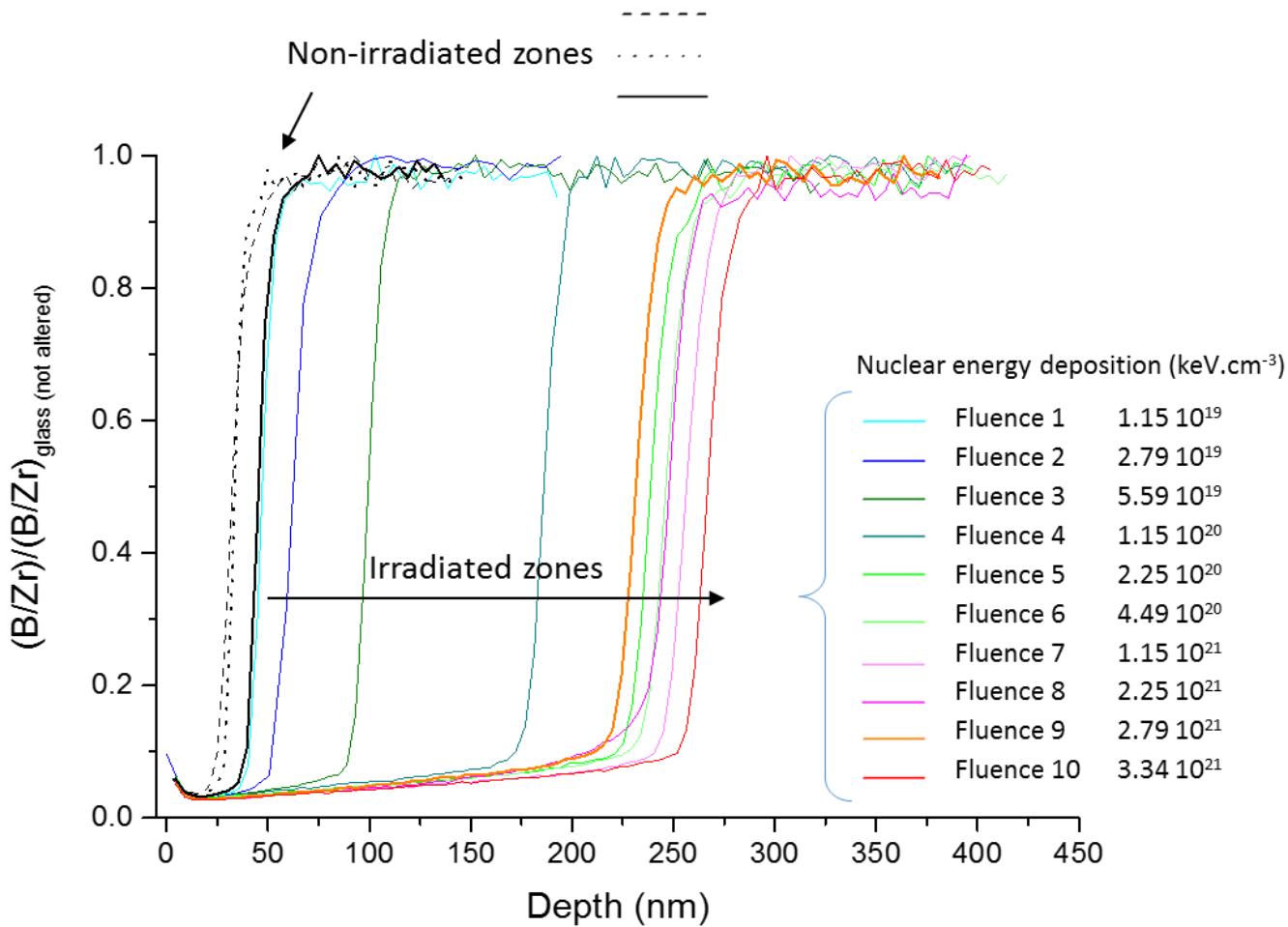
■ Alteration layer thickness determination from boron profile



ISG, 13 days of alteration, dose = 145 MGy

TOF-SIMS PROFILES

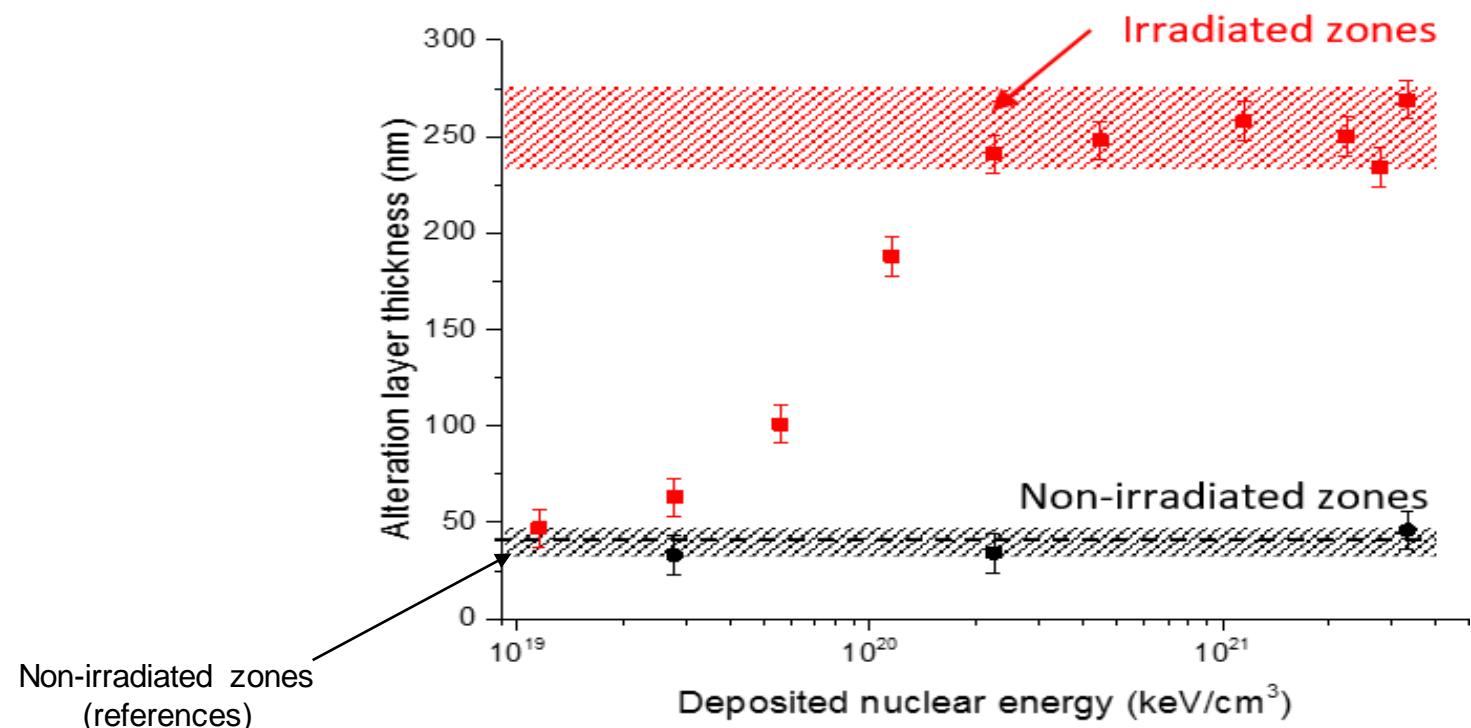
■ Alteration layer thickness determination from boron profile



TOF-SIMS PROFILES

Altered thickness vs fluence

a) ISG glass samples, altered for 13 days

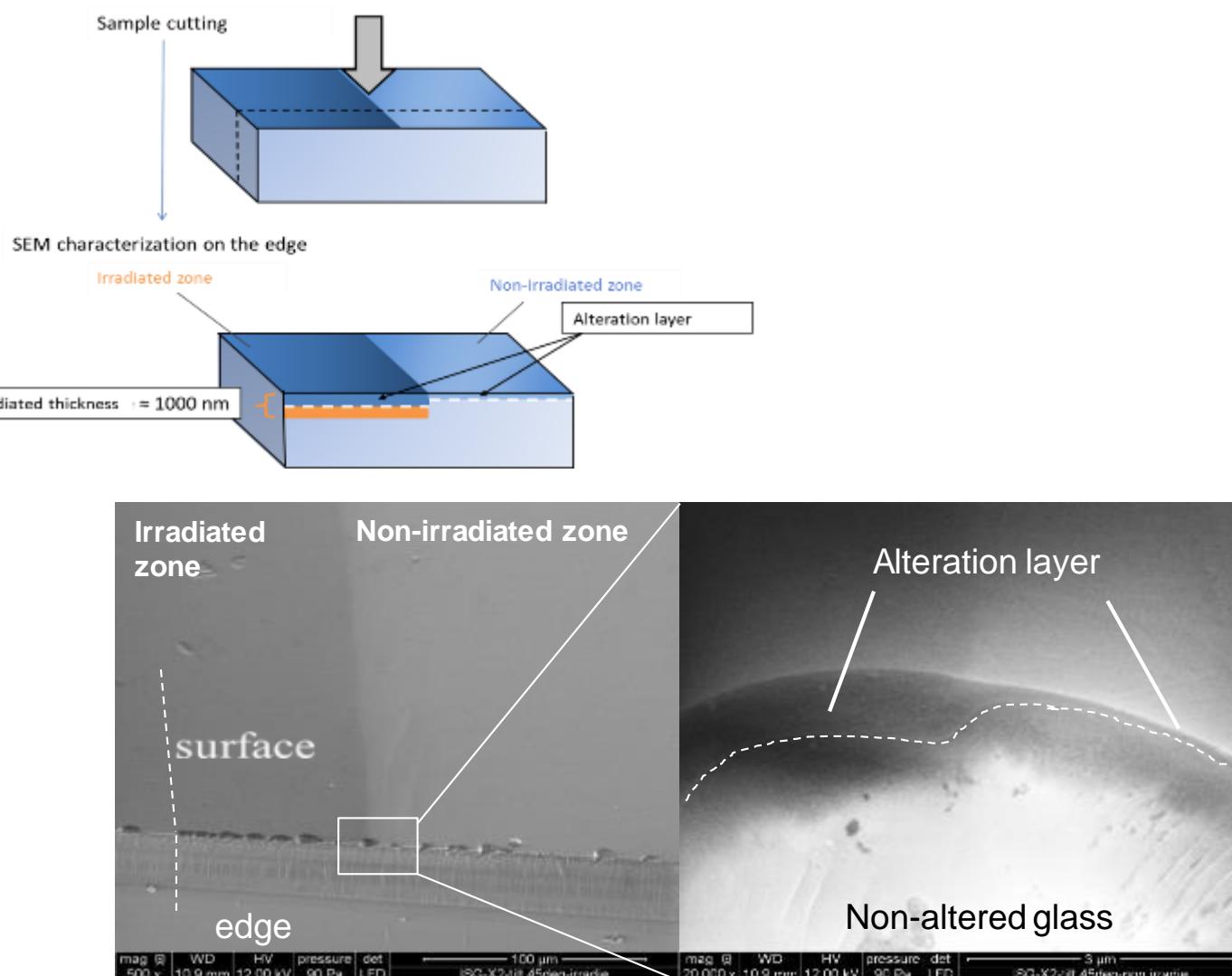


- All non-irradiated zones in agreement
- Increase of alteration layer thickness vs nuclear dose
- « plateau » observed after $\approx 2\text{-}4 \cdot 10^{20}$ keV_{bal}/cm³

On ²⁴⁴Cm-doped glass → higher Rr probably due to alpha cumulative dose

SEM & TEM CHARACTERIZATIONS

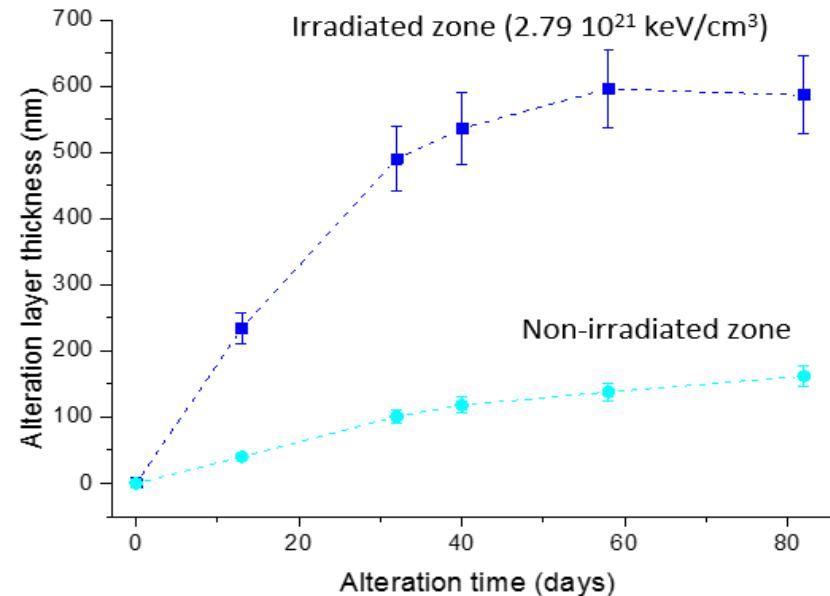
- SEM: ISG monolith altered 13 days (fluence = 3.34×10^{21} keV/cm³)



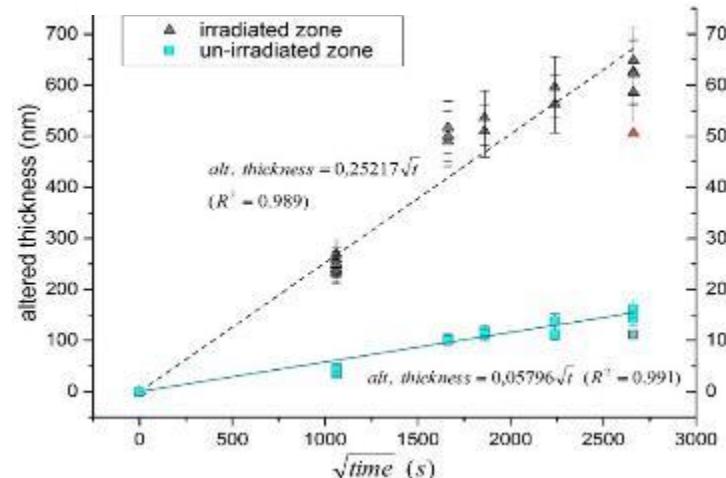
TOF-SIMS PROFILES

■ Alteration kinetics: 1 dose, thickness evolution versus time

a) ISG glass (fluence 9)



Diffusive mechanism



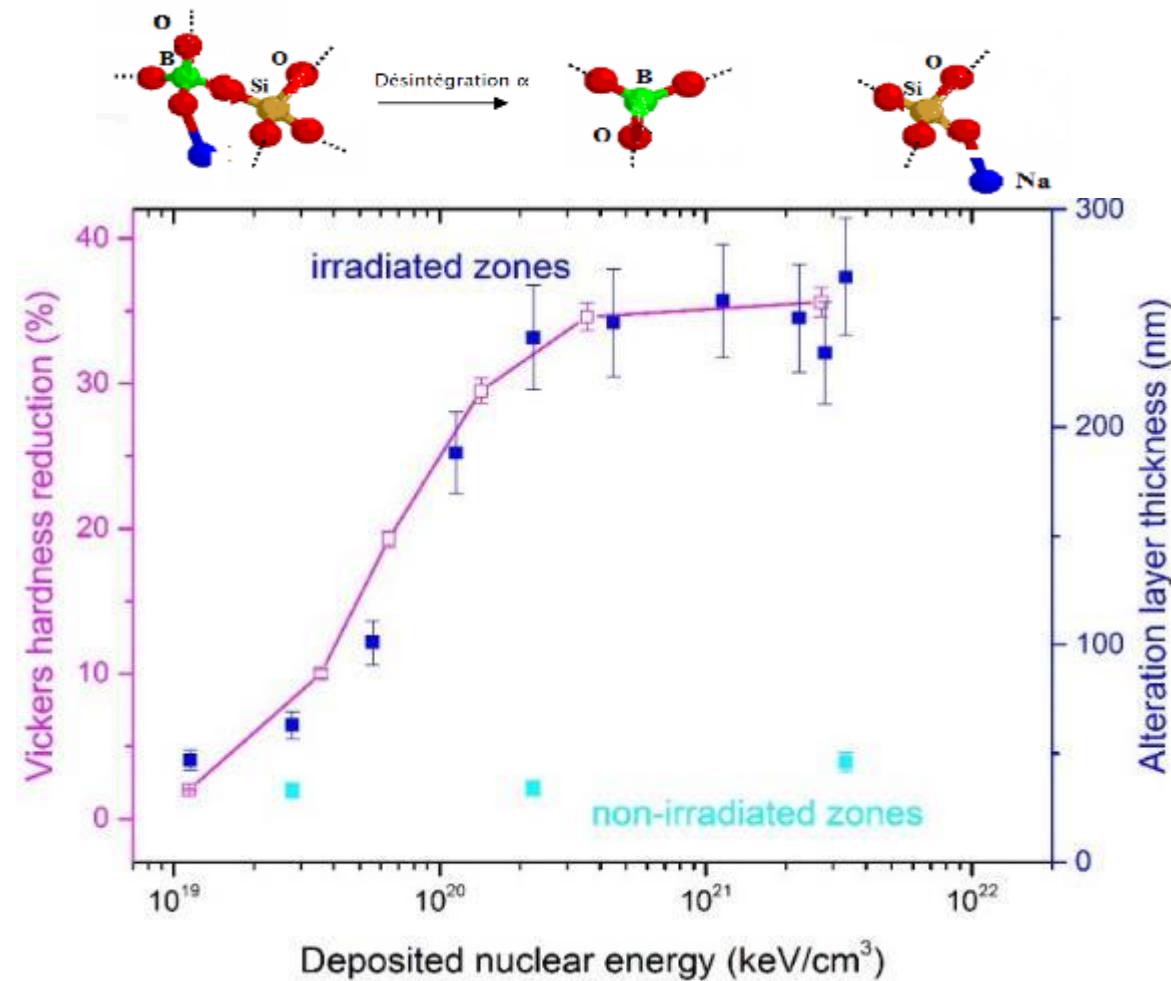
$$T_{alteration\ layer} = 2 \times \sqrt{\frac{D_{app} \times t}{\pi}}$$

Apparent diffusion coefficient increases
Increase of alteration rate $\times \sqrt{20} \approx 4.5$

$D_{app} (m^2/s)$	
Irradiated zone ($\geq 2.10^{20} \text{ keV/cm}^3$)	5.0×10^{-20}
Non irradiated zone	$2.6 \times 10^{-21} \times 20$

DISCUSSION

■ Comparison with modifications of glass structure and properties

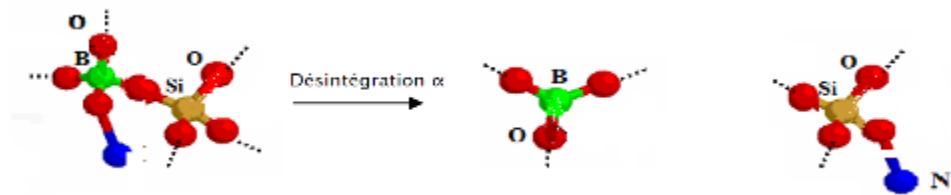


Vickers hardness reduction & altered thickness: same tendency

→ Structural / properties & chemical durability similarly affected by the same cause

DISCUSSION

■ Comparison with modifications of glass structure and properties



- Increase of free volumes → increase of water or alkali migration?
- Increase of intern energy → increase of glass reactivity?

Higher increase of alterability on glasses submitted to gold irradiation (x4.5) than on Cm-doped glass (x2.7)

- Glass composition effect: complex compositions less sensitive to irradiation than simple glasses
- Recovery effect of α particles in real alpha decay

CONCLUSIONS & PROSPECTS

Initial alteration rate: no significant effect of alpha irradiation on complex glasses

Residual alteration rate: increase of altered thickness on damaged glasses (x 4.5 max) and plateau reached for doses $> 2\text{-}4 \cdot 10^{20}$ keV_{bal}/cm³ (few 10^{18} α /g)



Chemical durability & glass structure / properties similarly affected by irradiation

- Mechanisms: water access and/or increase of local reactivity
- **Long term chemical durability of glass sensitive to its initial structure**

« simplified » system vs ²⁴⁴Cm-doped glass, also taking into account:

- Glass composition: simple glass more sensitive than complex glasses
- Recovery effect of α particles in real alpha decay → « Dual Beam » irradiations

To increase mechanistic understanding :

- To explore very initial steps of alteration (water penetration in damaged glasses)
- Atomistic modeling: create a damaged glass, explore water diffusion...
- To study properties of alteration layer formed from damaged glasses

DE LA RECHERCHE À L'INDUSTRIE



Funded by **CEA** and **AREVA NC**

With the support of



THANKS TO



J.M. Delaye, M. Tribet, A.H Mir, E.A. Maugeri, C. Mendoza,
R. Caraballo, O Bouty, C. Jégou
DEN/DTCD/SECM, CEA Marcoule, France



T. Charpentier, M. Moksura
DSM/IRAMIS, CEA Saclay, France



I. Monnet, M. Toulemonde, S. Bouffard, Ganil, Caen, France



J. DeBonfils, G. Panczer, D. DeLigny
LPCM – University Claude Bernard, Lyon



G. Calas, L. Galoisy
IMPMC - University Pierre et Marie Curie, France



G. Henderson
University of Toronto, Department of Geology, Toronto, Canada

T. Wiss, A. Jenssen, J.Y Colle, J. Somers, L. Martel, C. Selfslag,
D. Staicu, A. Zappia
EC JRC-ITU, Karlsruhe, Germany