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The CERUS Project

and the Argentina's back-end strategy for the research reactor spent fuel

Ceramization of radioactive elements in sintered uranium (CERUS)

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Organization

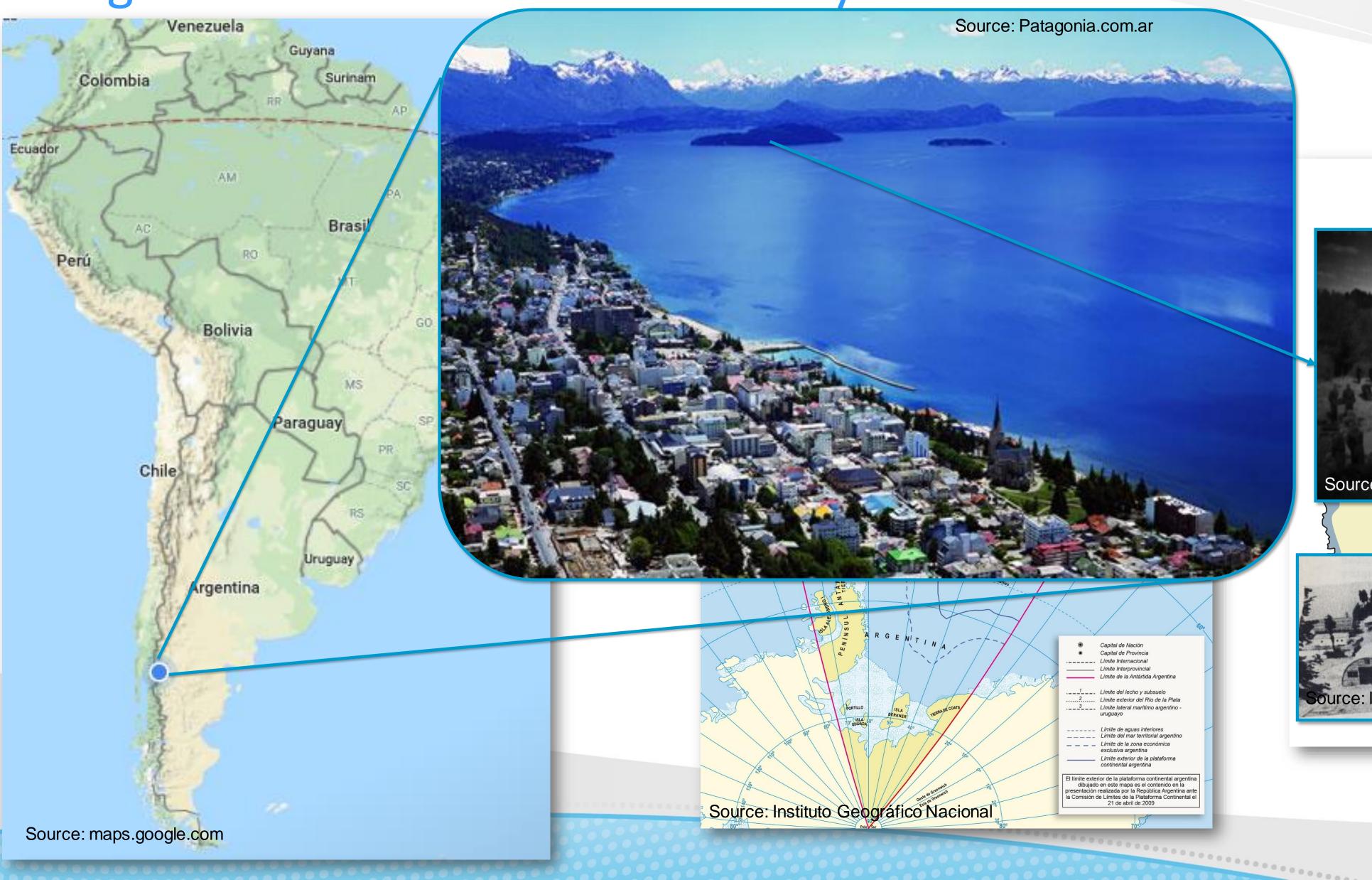


- Argentina and the nuclear industry
- Source of Spent Fuel and Radioactive Waste

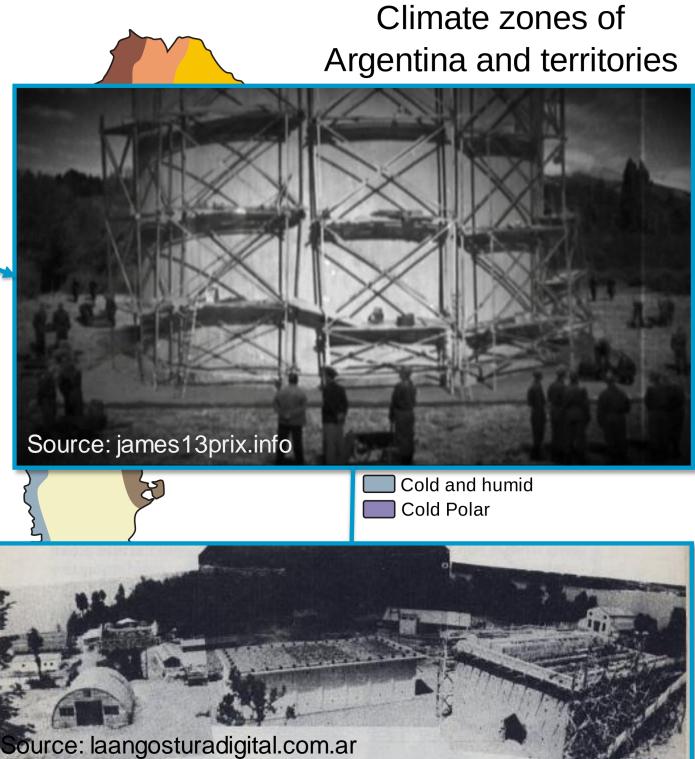
- Repositories of SF
- Cerus Project
- Conclusions and Outlook



Argentina and the nuclear industry

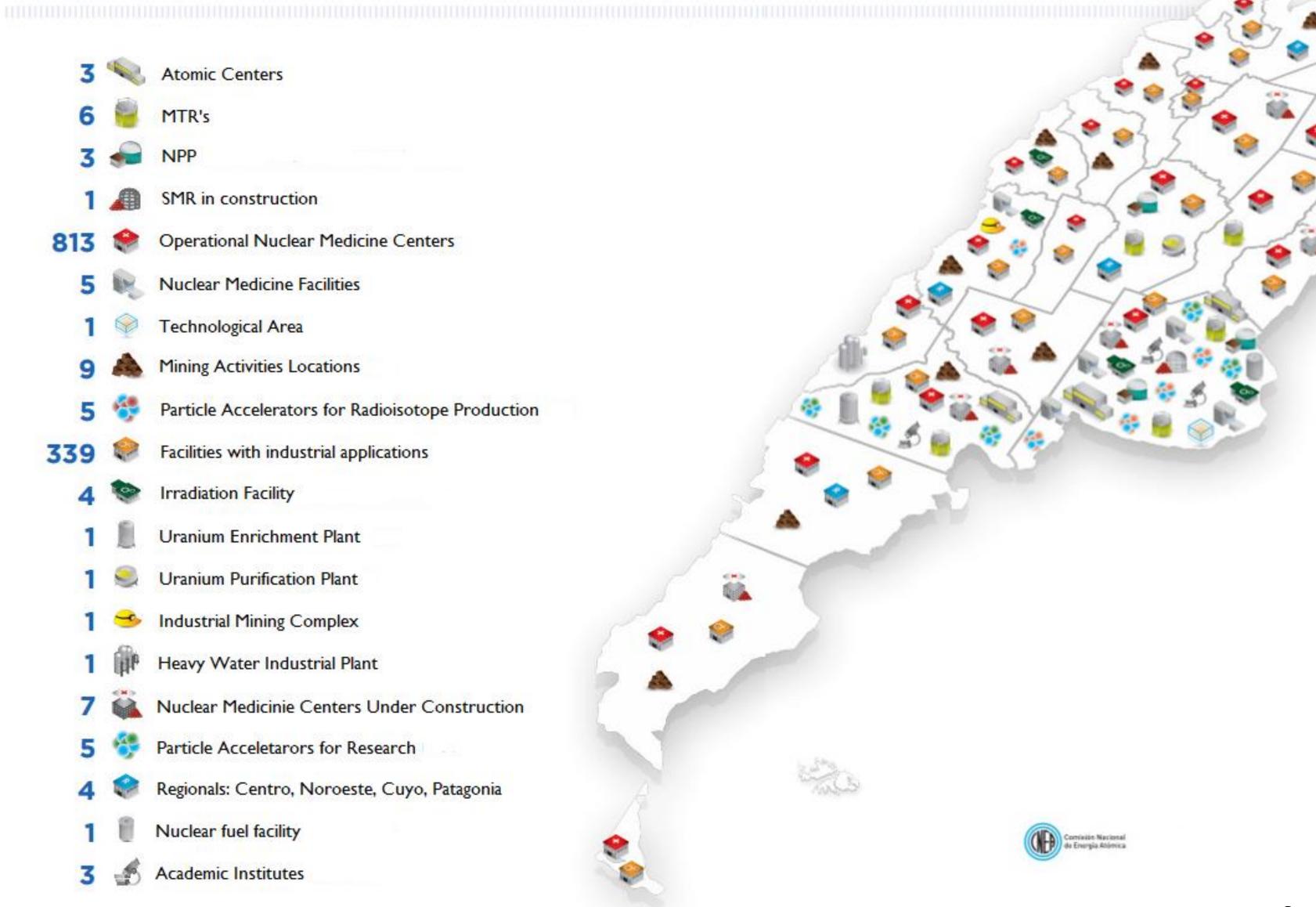






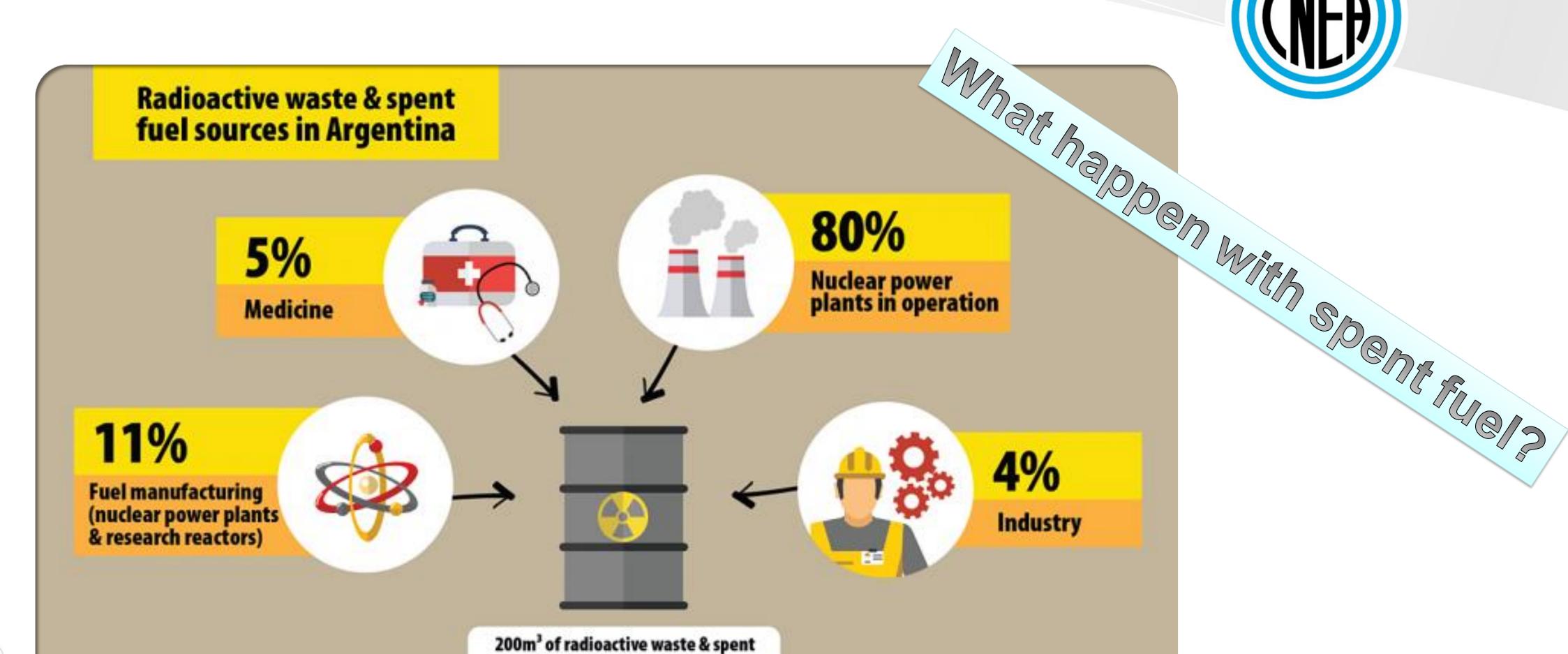
Argentine Antarctica
Source: mod of wikimedia.org

Argentina and the nuclear industry





Source of Spent Fuel and Radioactive Waste



Source: National Atomic Energy Commission (CNEA), Argentina.

fuel generated per year



Repositories



Site	Facility	
CNAI	I & II Pool Building	
CNAII	Pool Building	
CNE	Storage Pool	
	Storage Silos (ASECQ)	
Ezeiza Radioactive Waste Management Area (AGE)	Central Storage Facilities for Research Reactors SF (DCMFEI)	
Ezeiza Atomic Centre (CNEA)	Research Reactor Irradiated Fuel Storage Facility (FACIRI)	





Waste Management Facilities

- •NPP= systems of treatment and conditioning for solid, liquid and gaseous.
- •CAE= Radio Isotope Production plant
- •AGE= Storage of Rad. Sources, Long term Deposit, Cementing and compacting Pilot Plant
- Pilcaniyeu=LLW deposit

- •UO₂Plant= Raw material deposit
- Mining and processing U minerals (Mendoza,
 Córdoba, Chubut, Salta, La Rioja)

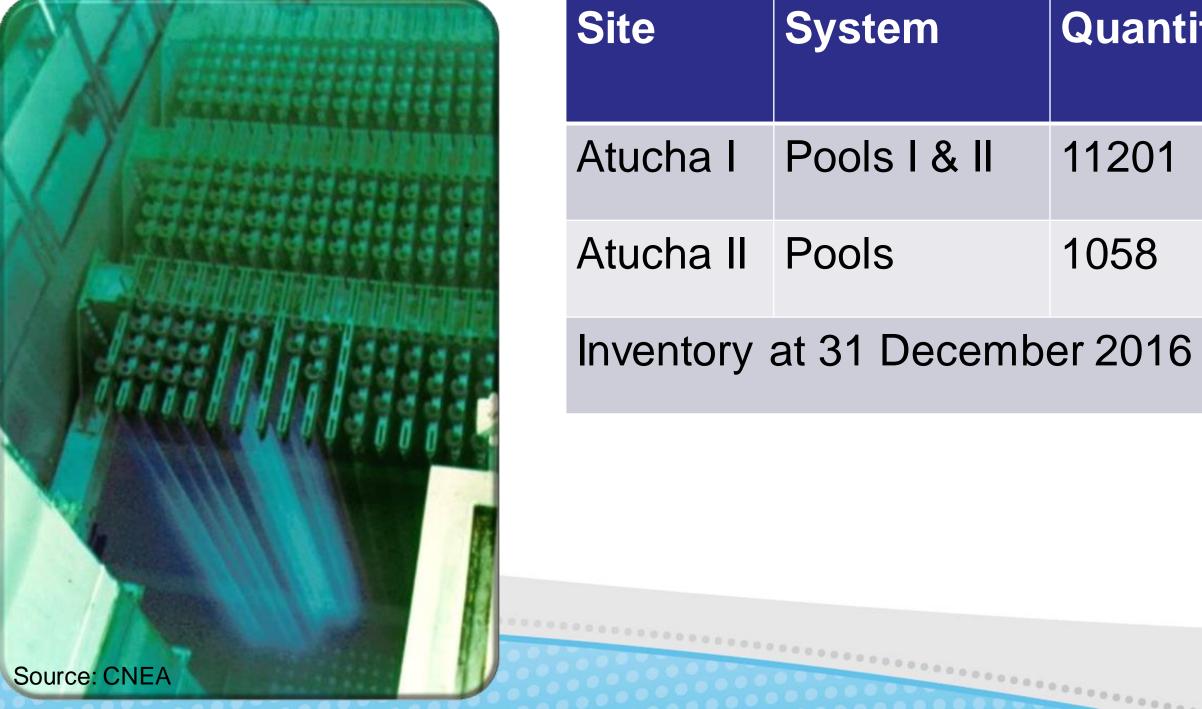
Repositories – Atucha NPP Complex







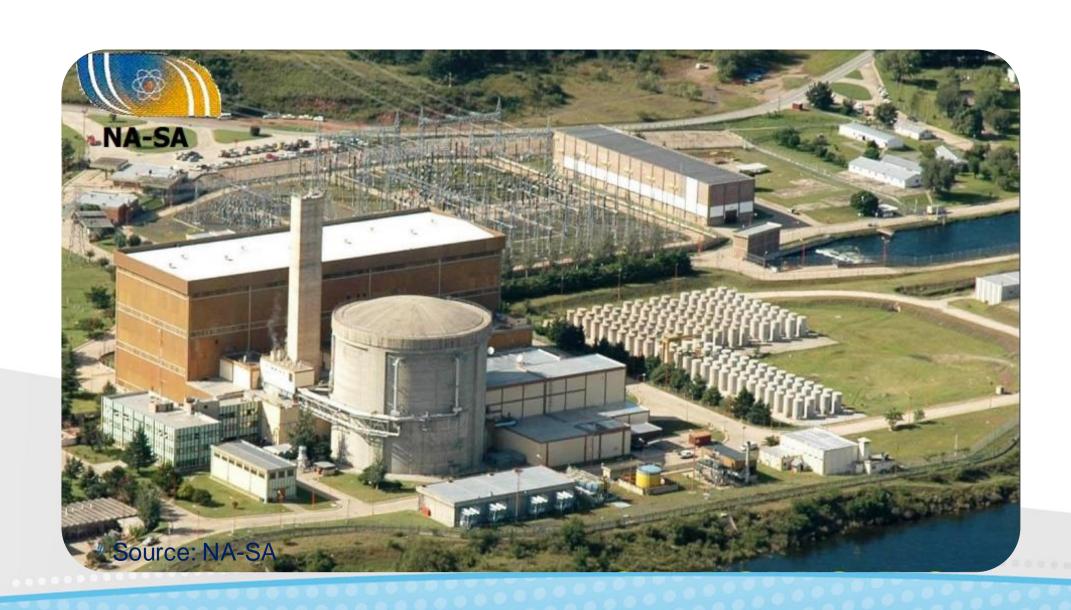




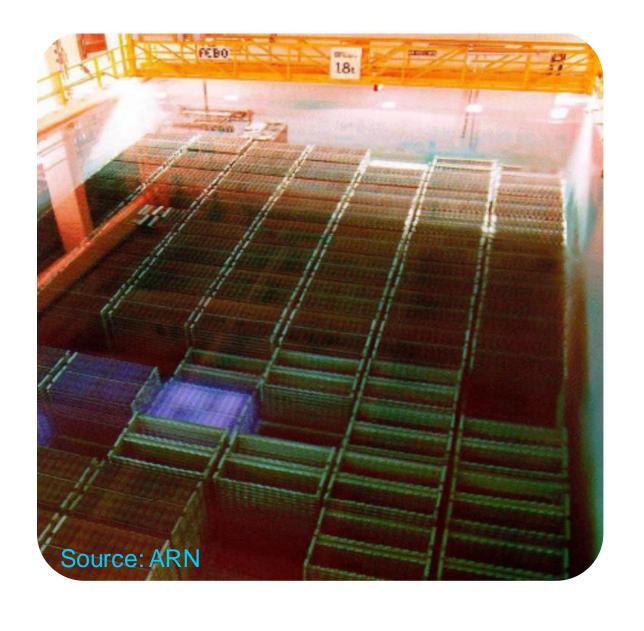
Site	System	Quantity	U_tot (kg)	Pu (est) (kg)
Atucha I	Pools I & II	11201	1705127.373	6405.647
Atucha II	Pools	1058	185789.718	679.015
Inventory at 31 December 2016				

Repositories – Embalse NPP





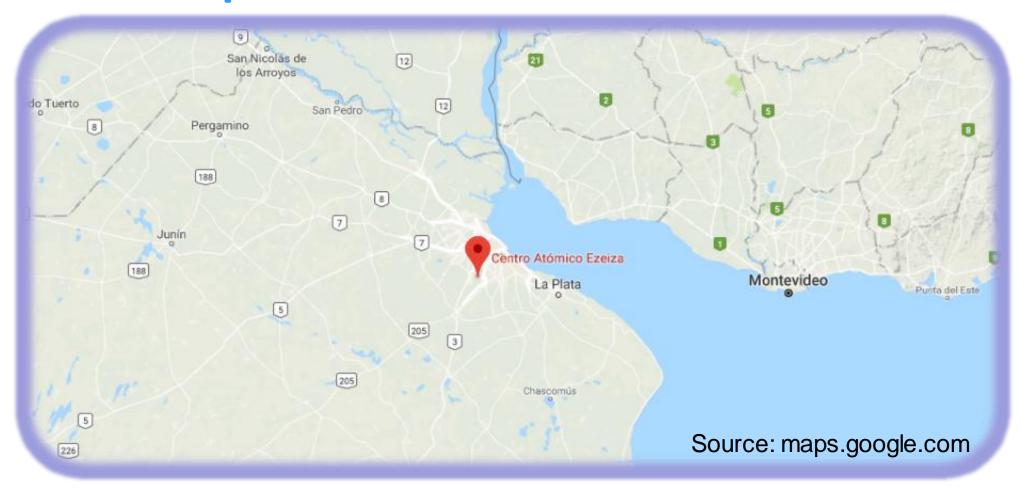
Site	System	Quantity	U_tot (kg)	Pu (est) (kg)
Embalse	Pools	32101	600475.400	2056.496
	Silos	113160	2111529.164	7753.933
Inventory at 31 December 2016				





- Pool → 8 years at 80% power
- SF \rightarrow Basket (x60) \rightarrow 9 bskt x silo
- 6 years of wet storage

Repositories - AGE





Site	Туре	Quantity	Kg
AGE	MTR	146	173327
	PINS	232	14188
Inventory at 13 August 2013 (IAEA PIV)			

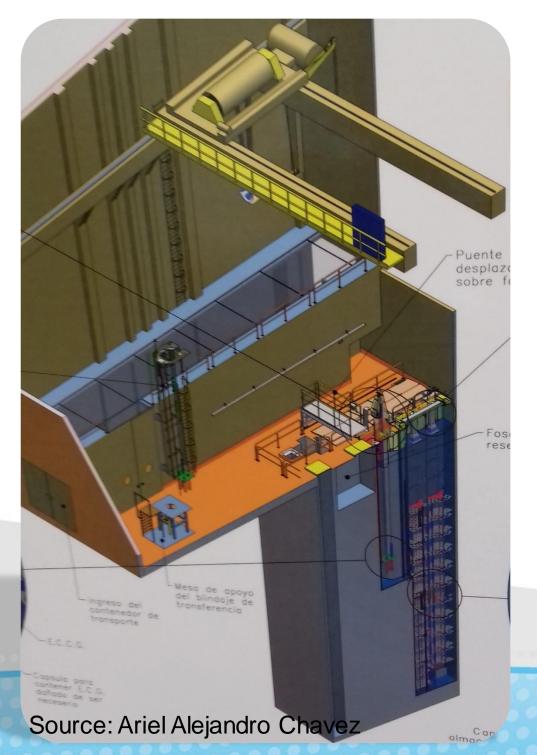


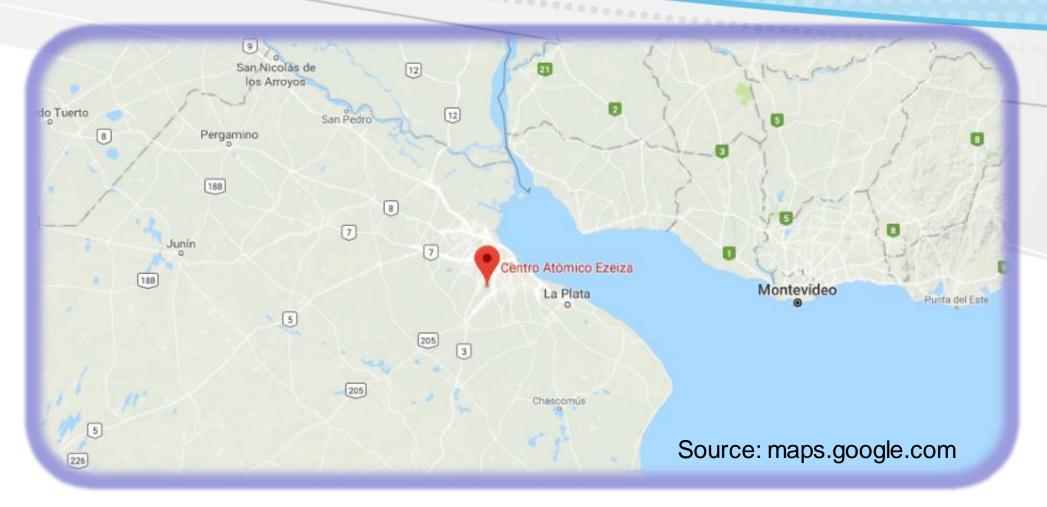


- Started 1972
- In 2003 studies of condition
 - Replace the facility

Repositories - FACIRI









Site	Туре	Quantity	Kg	
FACIRI	MTR	60	73065	
Inventory at 31 December 2016				

- Safety improvements
 - Conservation of SF
 - Water quality monitoring
- > 2 collumns along 16 m
- > 19 grids/ collumn
- > 32 SF assemblies
- Total capacity of 608 SF

Repositories

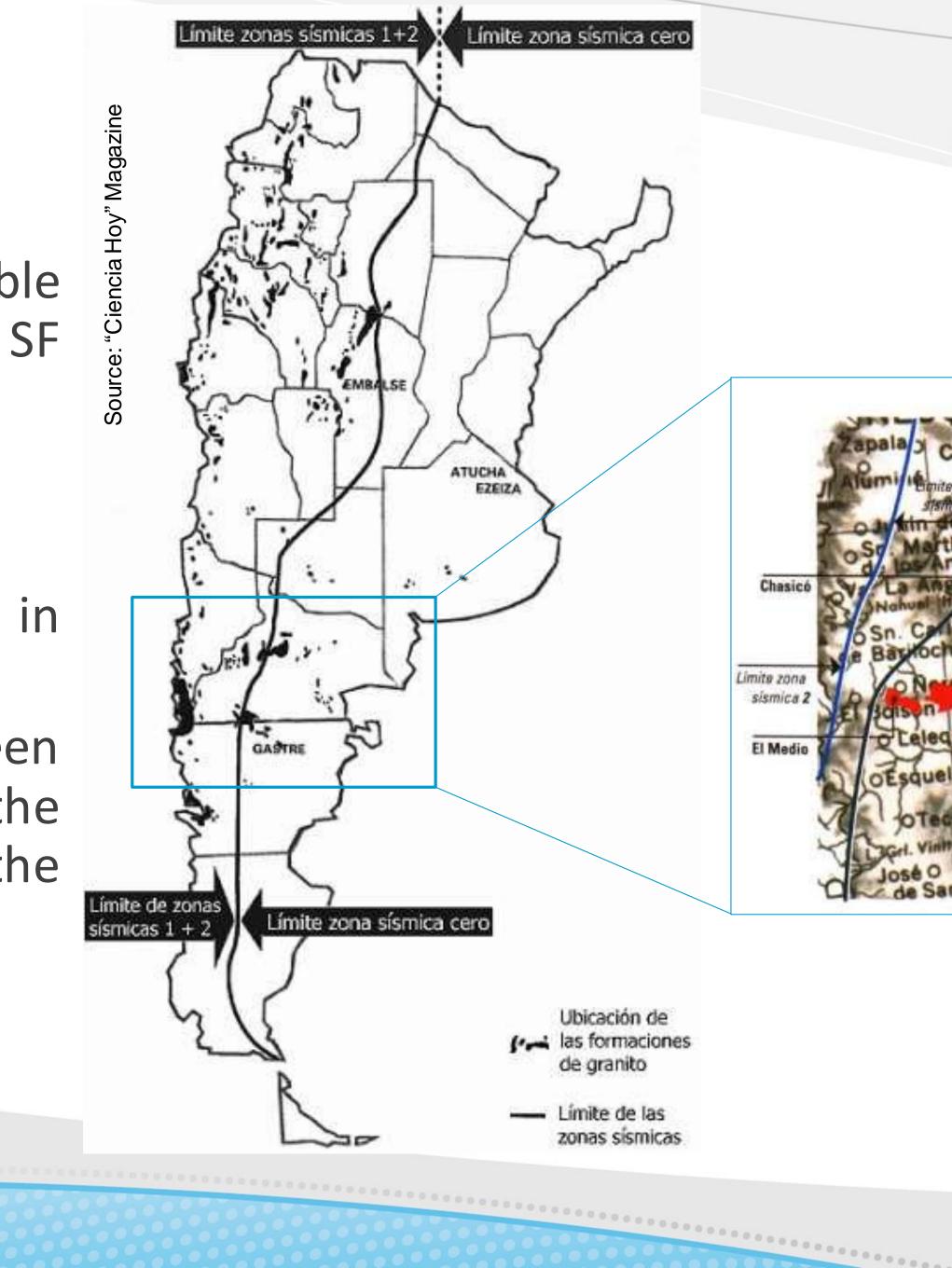
Projects

- •The decision on the possible reprocessing or final disposal of the SF will be made before 2030.
- Atucha I- Burned Fuel Assemblies Dry Storage
 - Vertical subterrain Silos
 - •2 baskets, 9 SF per basket
- •CAREM-25 NPP
 - Spent Fuel Storage Pool
 - Store SF originated during 10 years of full power operation
 - Cooling and clean up systems
 - •Quality of the water, compensate water loos by evaporation, monitoring radiological, physical and chemical parameters
- •RA-10 Reactor
 - Storage Pool, to cover 10 years of operation

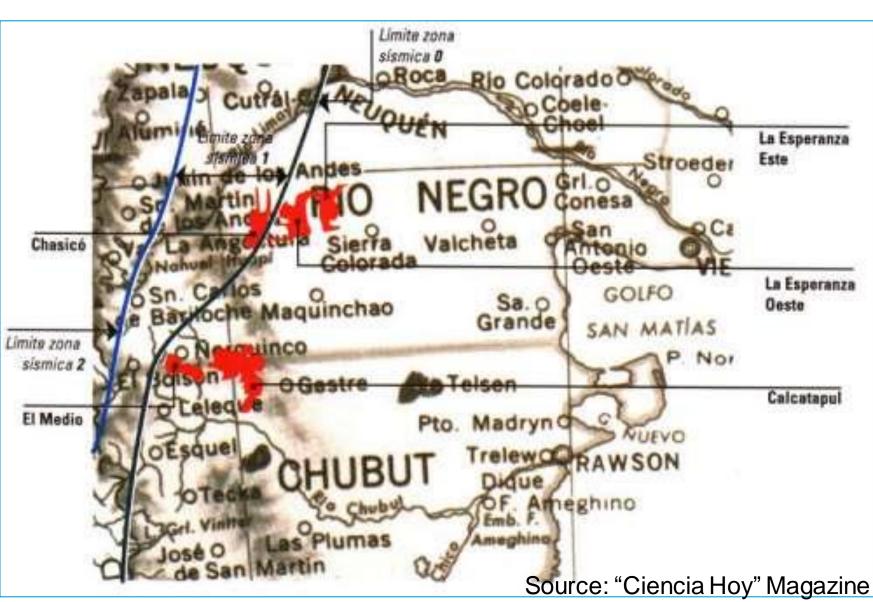
Repositories

Projects

- •The decision on the possible reprocessing or final disposal of the SF will be made before 2030.
- Final Disposal of Spent Fuel
 - Deep geological repository in Argentina by 2060
 - R&D activities have been conducted in relation to the geology of sites to locate the repository location.
 - Sierra del Medio Project
 - Granite
 - •Closed in 1996







CERUS/VITROCERUS is a method developed by National Atomic Energy Comission (CNEA) to conditioning Materials Test Reactors Spent Fuel Elements (MTRSFE). The key of the process is the use of natural or depleted uranium oxide to isotopically dilute the spent fuel, to reach a final enrichment of 1 to 2% U²³⁵, as the immobilization matrix, obtaining a monolithic ceramic waste form.

- SF enrichment ≈ 11%
- Addition of U₃O₈ (natural 0.711% or depleted 0.3% U²³⁵)



Precursor Powder

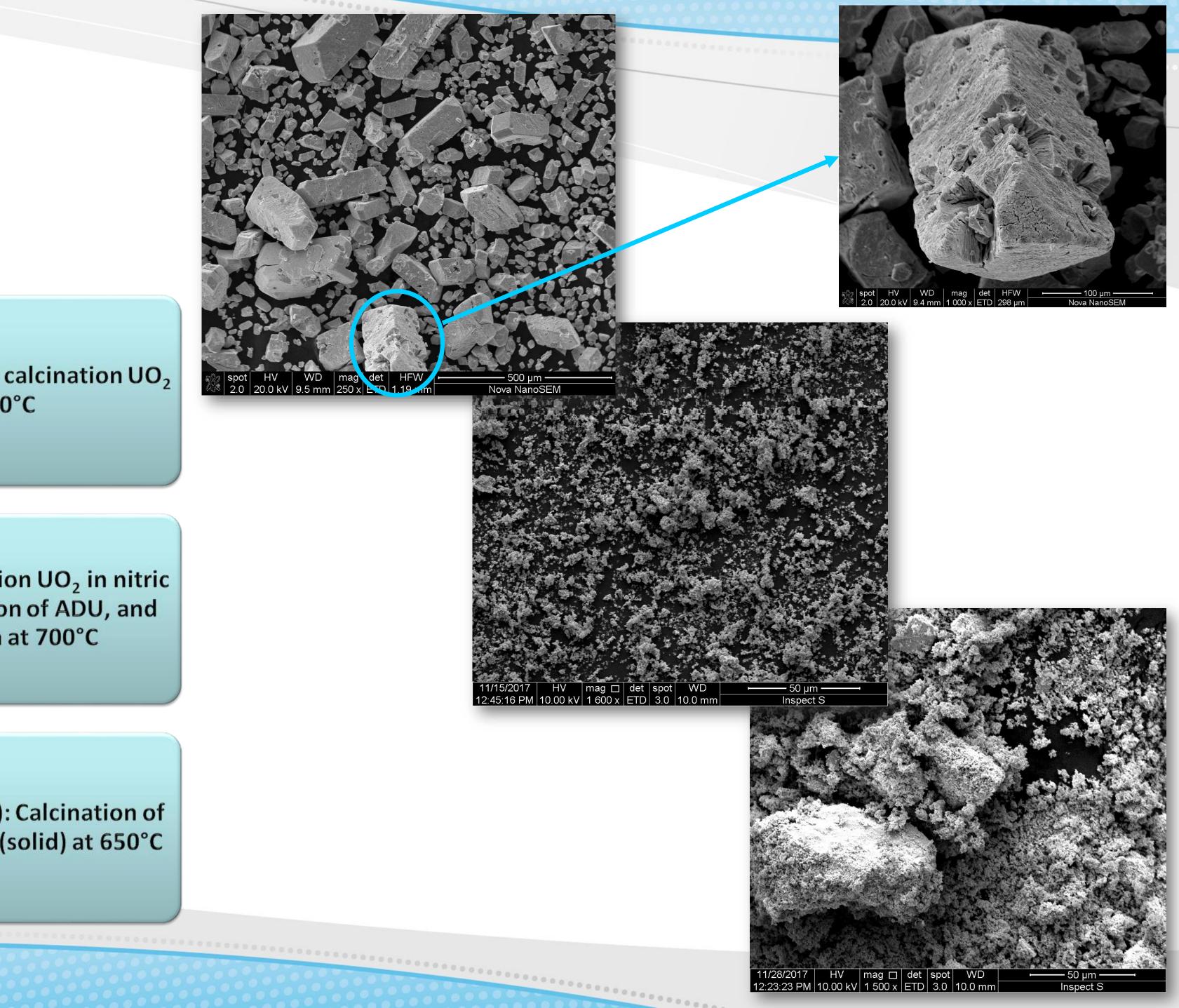
Route 1: Thermal calcination UO₂ at 700°C

U₃O₈ Powder sinthesis

Route 2: Dissolution UO₂ in nitric acid, Precipitation of ADU, and calcination at 700°C



Route 3 (posible): Calcination of Uranium nitrate (solid) at 650°C



Pellets

- •Pressed at 432, 504, 576 y 752 Mpa, SS matrix Ø=9.2 mm
- •Sintered at 1100, 1200, 1300 and 1400°C
 - Geometric density
 - •Water and Hg immersion to obtain density and porosity
- •HSM pellet 490 Mpa, 1300°C. Ø=4 mm

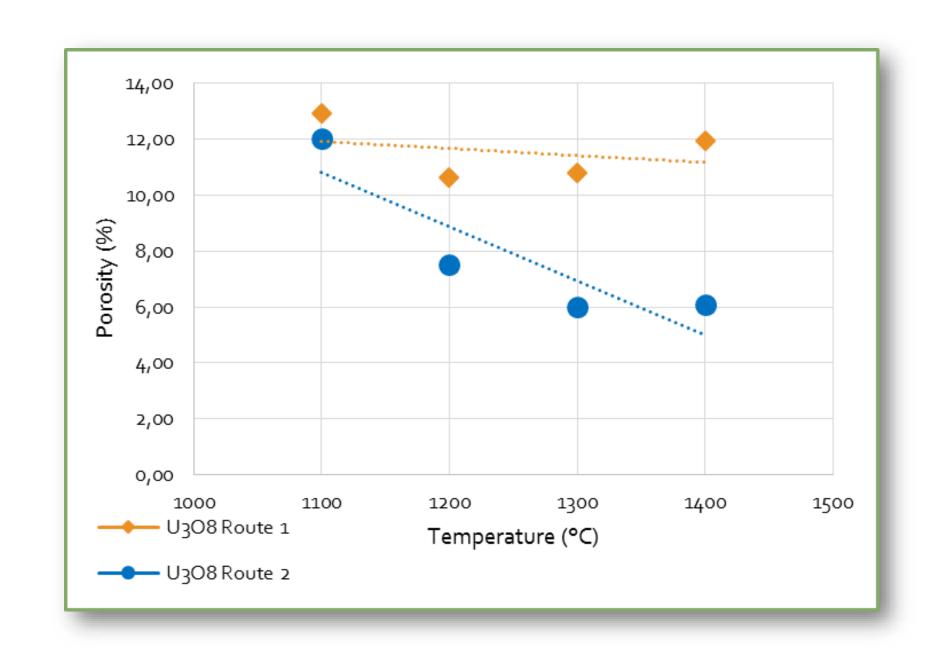


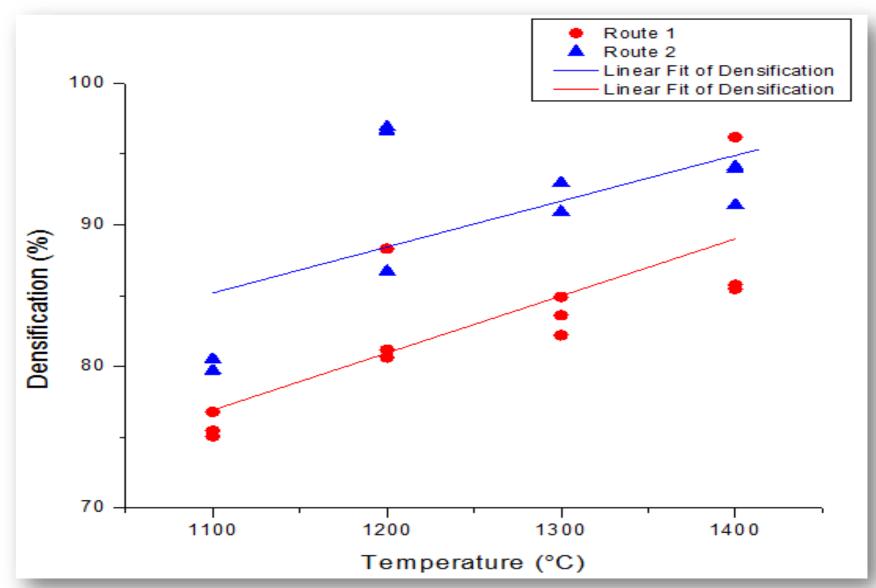








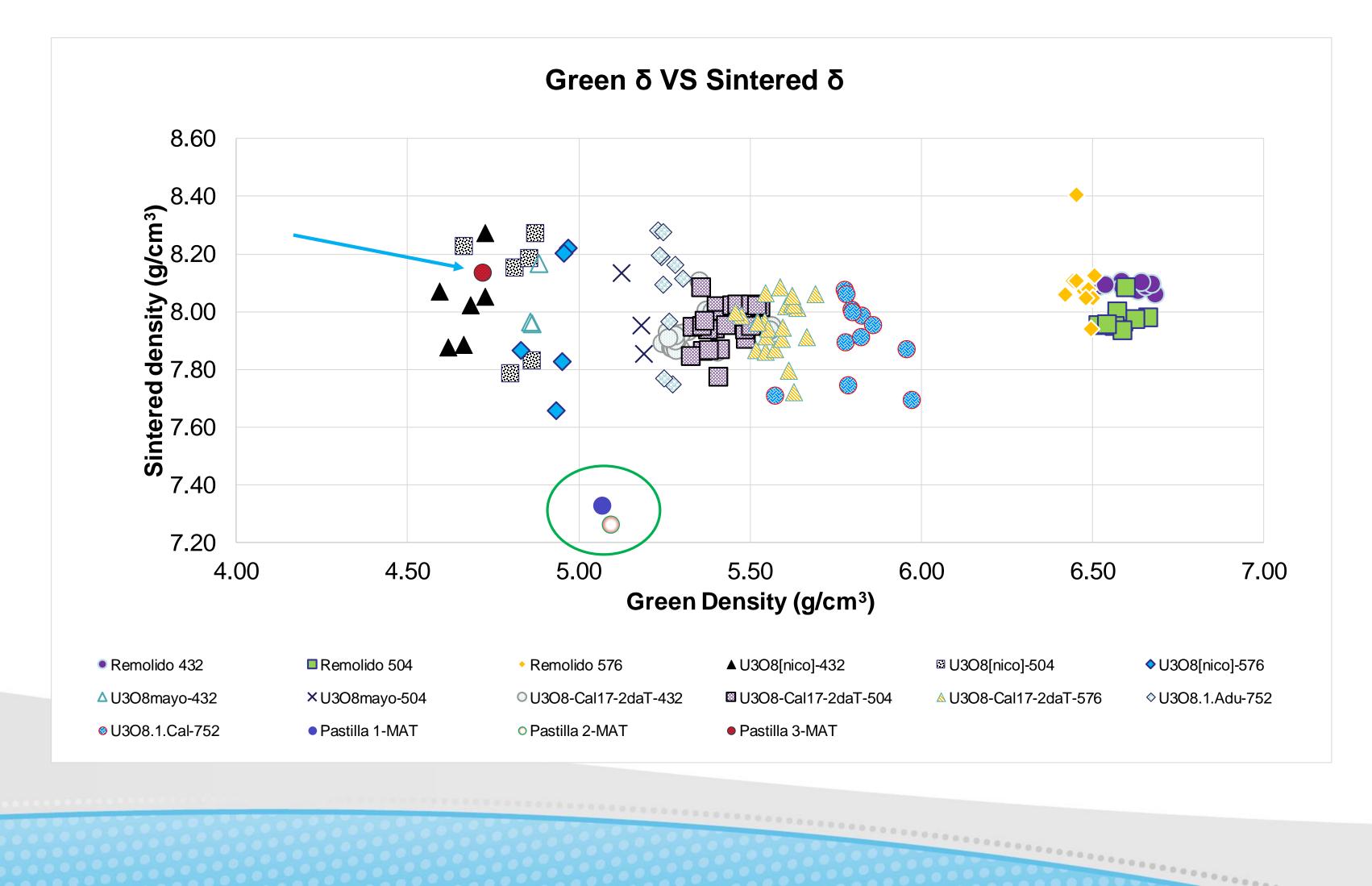






Pellets







U₃Si₂ - Al plates







Conclusions and Outlook

Conclusion

- •The U₃O₈ powder was successfully obtained by three routes
- Best ceramic matrix is obtained by ADU route (densification, porosity)
- Successfully oxidized and milled the Al-plate

Outlook

- •Mix of oxidized Al-plate + U_3O_8 (ADU), press and sintered
- Characterize pellets
 - Mechanical behavior
 - Chemical behavior
 - Porosity
- Scaling in size the pellets



Thank you very much for your attention!



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