

## **ANALYSIS OF THE CREOLE EXPERIMENT ON THE REACTIVITY TEMPERATURE COEFFICIENT OF THE UO<sub>2</sub> LIGHT WATER MODERATED LATTICES USING MCNP6.1**



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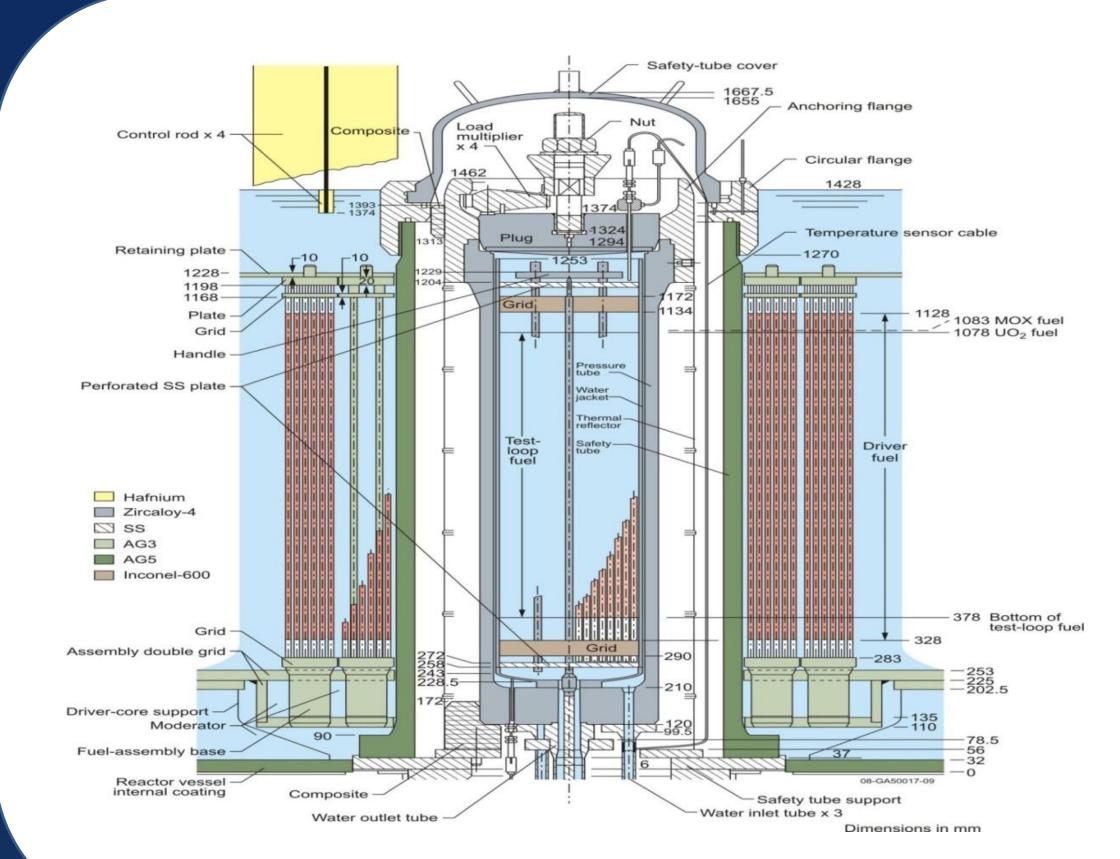
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## ABSTRACT

The CREOLE experiment have allowed us to get interesting and complete experimental information on the temperature effects in the light water reactor lattices. To analyze their experiments with accuracy an elaborate calculation scheme using the MONP6.1 code and the ENDF/B-VII.1 cross section library has been developed. We have used the ENDF/B-VII.1 data provided with the MCNP6.1.1 version in ACE format and the Makxsf utility to handle the data in the specific temperatures not available in the MCNP6.1.1 original data. We have analyzed the case of UO2 lattice with 1166 ppm of boron in the ordinary water moderator in specified temperatures. A detailed comparison of the calculated effective multiplication factors with the reference ones [1] in room temperature presented in this work shows a good agreement demonstrating the validation of our 3D modelization.

The discrepancies C - E on the Reactivity temperature Coefficient for the analysis of direct differential measurements of the reactivity temperature coefficient, we have also analyzed integral measurements using equivalency of the integral temperature reactivity worth with the driver core fuel reactivity worth and soluble boron reactivity worth.

CREOLE



Zircaloy-4 tie rod x 4 Vater jacket \*\*\*\*\* Inconel-600 grid (0.4 mm thickness) Central tube (thimble) Pressure tube – max gap 0.9 mm, min gap 0.5 mm Fuel element x 200 <del>de cie</del>loie e 2-mm ga max gap 0.9 mm

#### **Table I:** Critical sizes at room temperature

Core configuration	Driver-core temperature (°C)	Central-loop temperature (°C)	Central-loop pressure (bar)	Doubling time (s)	Residual Reactivity (pcm)	Driver-core size (fuel rods)
UO <sub>2</sub> reference	18.5 ± 0.2	20.27 ± 0.2	93.2 ± 0.5	7.06 ± 0.2	312 ± 13	1620
UO <sub>2</sub> 166 ppm boron	19.6 ± 0.2	21.83 ± 0.2	66.5 ± 0.5	6.86 ± 0.2	316 ± 13	1772

Figure 2: Radial cross section of the central loop

Figure 1: Axial cross section of CREOLE reactor

UO<sub>2</sub> configuration with 1166 ppm of soluble boron

#### for: $20^{\circ}C \le T \le 300^{\circ}C$

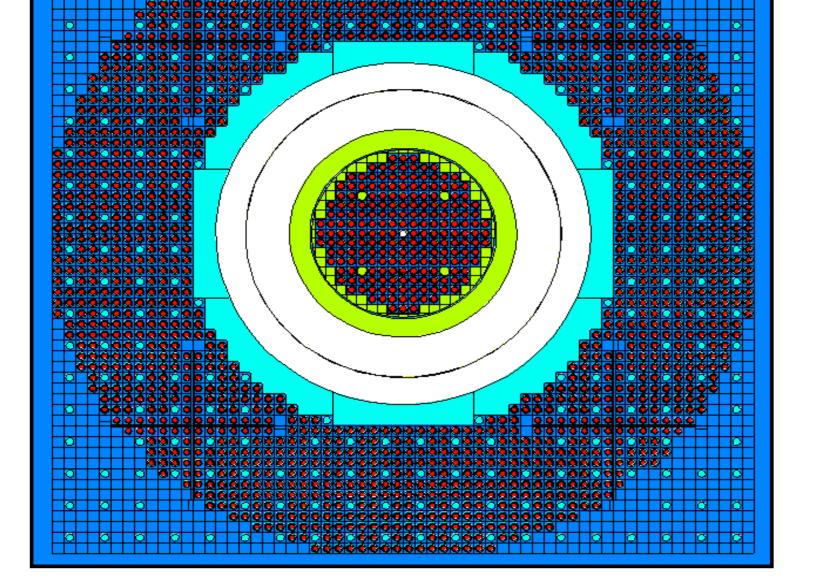


Figure 3: Radial cross section of the CREOLE using MCNP code (at all *temperatures*) – 1772 *driver fuel rods*.

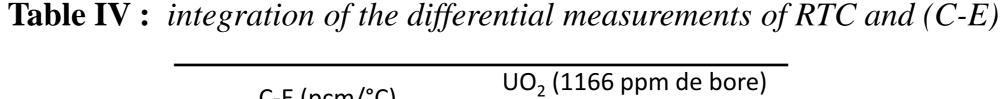
**Table II :**  $k_{eff}$  calculation and experiment comparison at room temperature.

	Experiment [1]	Model- Benchmark [1,8]	MCNP6.1 ENDF/B7.1	TRIPOLI 4.5 JEFF3.1.1 [1]
L.	1.00299	1.00328	1.00452	1.00556
ĸ <sub>eff</sub>	±0.00182	±0.00183	±0.00004	±0.00010

 $\rho_{MCNP}$  = - 5.38951 10<sup>-18</sup> T<sup>6</sup> + 4.94248 10<sup>-15</sup> T<sup>5</sup> - 1.76762 10<sup>-12</sup> T<sup>4</sup> + 2.64586 10<sup>-10</sup> T<sup>3</sup> + -1.56852 10<sup>-8</sup> T<sup>2</sup> + 4.9992 10<sup>-7</sup> T - 0.00224

**Table III :** Analysis of the Differential Measurements of the RTC and (C-E)

a and C-E (pcm/°C)	20 °C – 111 °C	111 °C – 186 °C	186 °C – 242 °C	242 °C – 296 °C
EXPERIENCE (a) [1]	$+ 0.02 \pm 0.04$	- 0.12 ± 0.04	- 0.35 ± 0.05	- 0.67 ± 0.06
MCNP6.1 (ENDF/B7.1) (C – E)	- 0.08 ± 0.04	+ 0.11 ± 0.04	- 0.02 ± 0.05	+ 0.04 ± 0.06
TRIPOLI4 (JEFF3.1.1) (C -E) [1]	- 0.10 ± 0.06 (0.04)*	+ 0.08 ± 0.06 (0.05)*	- 0.01 ± 0.09 (0.07)*	+ 0.05 ± 0.10 (0.08)*
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$C \in (n cm / C)$	
C-E (pcm/°C)	20°C – 296°C
EXPERIENCE (a) [1]	- 0.22 ± 0.02
MCNP6.1 (ENDF/B-VII.1)	- 0.008 ± 0.06
TRIPOLI4 (JEFF3.1.1)	- 0.004 ± 0.07

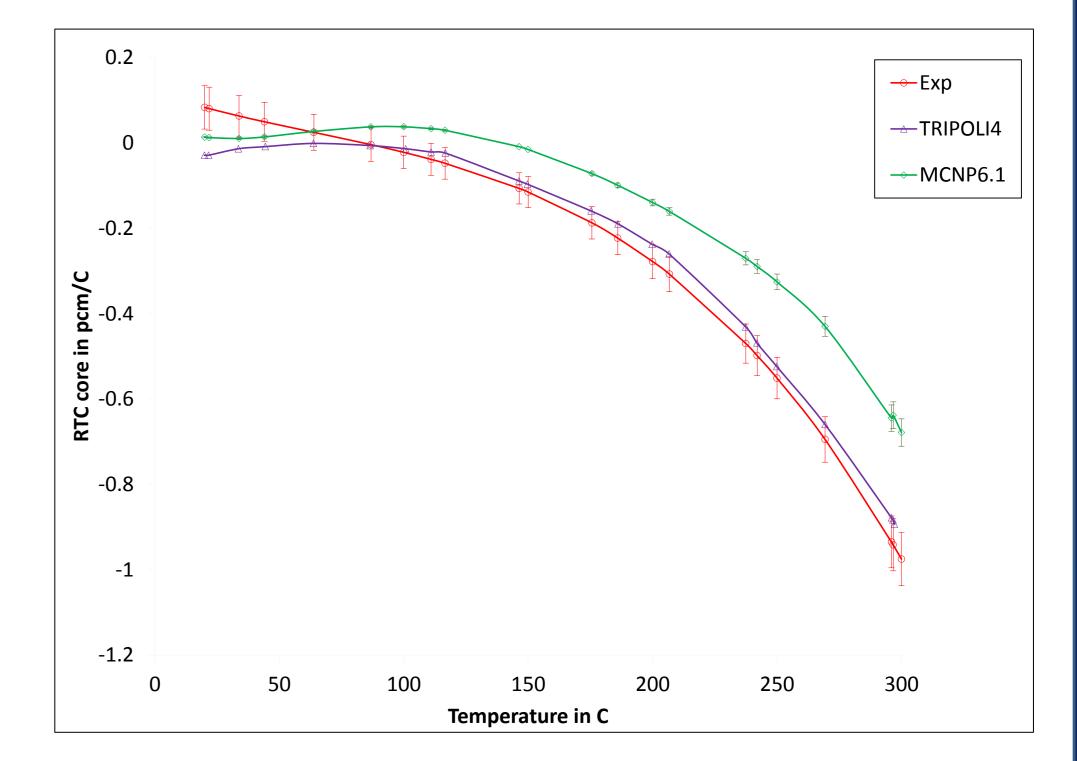


Figure 4: Radial Calculated and measured RTC as a function of the loop temperature with fitting curves.

UO<sub>2</sub> Clean Lattice

**Equivalency Between the Temperature Reactivity** 

#### **Equivalency Between the Temperature Reactivity**

#### Worth and Driver Core Fuel Reactivity Worth

**Table V** : Benchmark model parameters [1].

State A	State B
n	
Z	.95
1620	1680
20.27	243.22
	1620

**Table VI :** The keff for the two states and the calculation bias of the

 average RTC

	State A		State B
	MCNP6.1 (ENDF/B7.1)	$1.00224 \pm 4$	$1.00221 \pm 4$
k <sub>eff</sub> ± Std (pcm)	MCNP6.1 (JENDL-4)	$1.00164 \pm 4$	1.00184±4
$[\overline{lpha}_{calc} - \overline{lpha}_{meas}]$ (pcm/°C)	MCNP6.1 (ENDF/B7.1)	- 0.013	± 0.06
	MCNP6.1 (JENDL-4)	$+ 0.089 \pm 0.04$	
	TRIPOLI4 (JEFF3.1.1)	- 0.02 ± 0.05	

# $\left[\bar{\alpha}_{calc} - \bar{\alpha}_{meas}\right] = \frac{1}{\Delta T} \left| \frac{\left(K_{eff}\right)_B - \left(K_{eff}\right)_A}{\left(K_{eff}\right)_{+} \left(K_{eff}\right)_{-}} \right|$

#### Worth and the Boron Reactivity Worth

**Table VII** :Benchmark model parameters [1].

	State A	State B
Driver core loading (rods)		1680
Residual measured reactivity (pcm)		139
Central loop temperature (°C)	18.4	273.51
Boron content (ppm)	454±2	0.

**Table VIII** : The keff for the two states and the calculation bias of the
 average RTC

	State A	State B	
k <sub>eff</sub> ± Std (pcm)	MCNP6.1 (ENDF/B7.1)	$1.00109 \pm 4$	1.00086 ± 4
	MCNP6.1 (JENDL-4)	$1.00063 \pm 4$	$1.00039 \pm 4$
$[\overline{lpha}_{calc} - \overline{lpha}_{meas}] \ (pcm/^C)$	MCNP6.1 (ENDF/B7.1)	$-0.09 \pm 0.04$	
	MCNP6.1 (JENDL-4)	- 0.09 ± 0.04	
	TRIPOLI4 (JEFF3.1.1)	- 0.15 ± 0.06	

### Conclusion

The obtained results for the prediction of critical sizes at room temperature show a good agreement between the MCNP6.1 code and the reference ones, which validates our Monte Carlo modeling and assure us for the prediction of the RTC with sufficient accuracy.

The discrepancies between calculations and experiment on the RTC for clean and Boron poisoned UO<sub>2</sub> LWR lattices is small, less than 1 pcm/°C which corresponds to the current target accuracy in LWR design calculations. we should particularly mention the coherence of the results obtained by different type of measurements: direct differential measurements and integral measurements through equivalency.

## References

1. CEA/SPRC (2008). "CREOLE PWR Reactivity Temperature Coefficient Experiment. Centre de Cadarache". Rapport CEA – R – 6215.