



the
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**Workshop on
Nuclear Reaction Data and Nuclear Reactors:
Physics, Design and Safety**

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**Monte Carlo Method:
Theory and Exercises (2)**

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These are preliminary lecture notes, intended only for distribution to participants

**Monte Carlo simulation of a
TRIGA source driven core
configuration:
Preliminary results**

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Aim of the work

Simulate the time behaviour of a TRIGA sub-critical core configuration when driven by neutron source.

Design of in core experiments:
Choice of optimal measurements points.
Fuel and control rods worth estimation.

?? Experiments ??

Check
&
revision

Monte Carlo for particles transport simulation.

MCNP4c

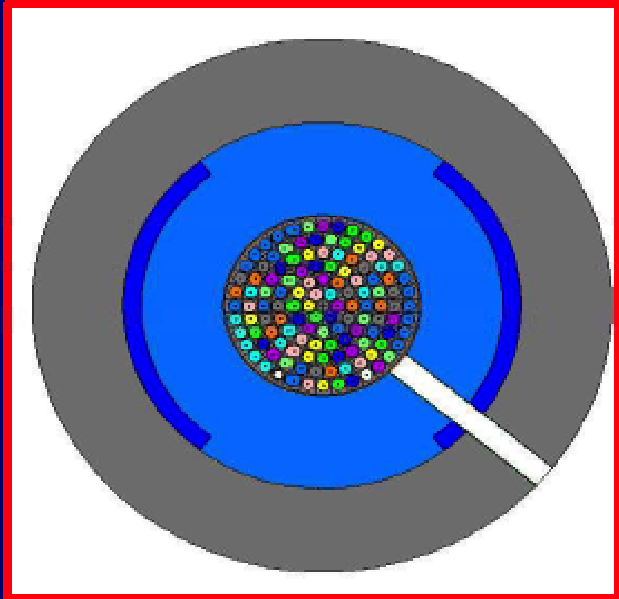
Points of strenght:

- "Continuos" Cross Section database
- High detailed geometrical description.
- Multi level source setup.
- Statistical checks for tally convergence.
- Variance reduction schemes.

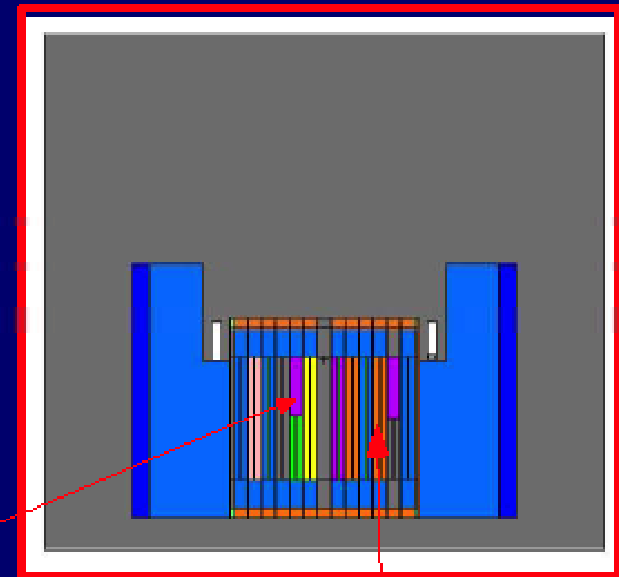
Points of weakness:

- Long calculation time (parallel computer highly recommended)
- Uncertainties in the time dependent cross sections
- False tallies convergenge in time.

Main components of the MCNP model of the 1 MW TRIGA reactor of Casaccia.



MCNP4c plotter cross section



Control & fuel rods

Boron Carbide

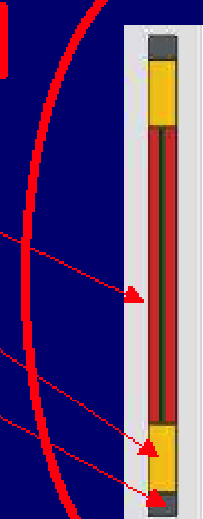
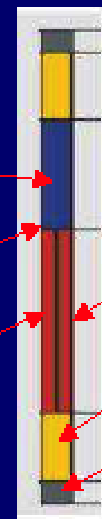
Floating surface

Fuel Follower

Fuel

Graphite

AISI 304



Criticality calculation methodology (kcode)

- Fresh fuel in the reference core configuration (235)
- Temperature field related to the desired power level.
- Control rods position compatibles with the desired power level.
- Iterative refinement of the criticality source (scrtp file).
- Fix source calculation for flux tallies (wssa file).
- ENDF/B-VI cross sections
- ENDF data for molecular binding effect in thermal neutron scattering (Light Elements).

K_{eff} estimations

Power 20 Watt

$$K_{\text{eff}} = 1.00972 \pm 0.00044 \text{ (no burnup taken into account)}$$

Power 1 Mwatt

$$K_{\text{eff}} = 1.00309 \pm 0.00068 \text{ (no burnup taken into account)}$$

Estimated

$$P_{\text{max}} / P_{\text{av}} = 1.57$$

$$\phi_{\text{rabbit}} = 1.86 \cdot 10^{13} \text{ n cm}^{-2} \text{ sec}^{-1}$$

$$\phi_{\text{ct}} = 4.30 \cdot 10^{13} \text{ n cm}^{-2} \text{ sec}^{-1}$$

Experimental

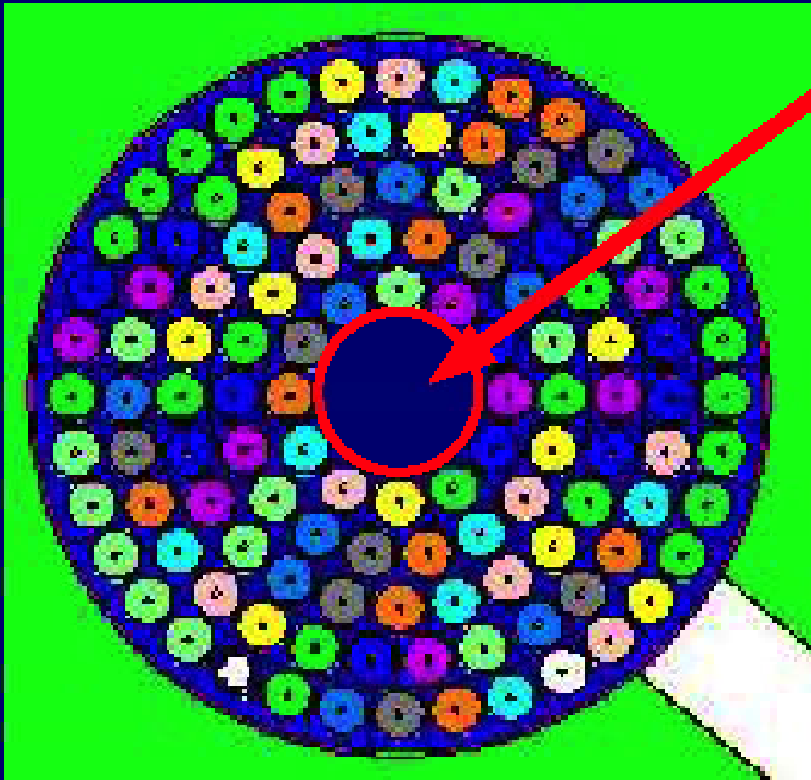
$$P_{\text{max}} / P_{\text{av}} = 1.55$$

$$\phi_{\text{rabbit}} = 1.91 \cdot 10^{13} \text{ n cm}^{-2} \text{ sec}^{-1}$$

$$\phi_{\text{ct}} = 6.35 \cdot 10^{13} \text{ n cm}^{-2} \text{ sec}^{-1}$$

Hypothesis of subcritical core configuration

No ring B



The subcritical configurations were achieved by removing ring B and setting different control rods positions. The k_{eff} values, obtained running MCNP KCODE, give a relative estimation of the subcriticality level of each configuration.

*Results:

shim1 (C10)	shim2 (C7)	safety (C4)	K_{eff}	σ
100%	45%	100%	0.9806	$7 \cdot 10^{-4}$
40%	0%	100%	0.9602	$7 \cdot 10^{-4}$
0%	0%	0%	0.9308	$8 \cdot 10^{-4}$

*Control rod insertions are reported as % of fuel follower in core

The neutronic source (driver) in the central thimble.

Deuterium – Tritium source.

Neutron energy : 14 MeV (mean) , $\sigma = 0.8$ MeV

Spectrum : gaussian distribution.

Intensity : 2×10^8 n /sec / 4π

Pulse rate : up to 20 kHz.

Pulse width : from 5 ms to continuous mode.

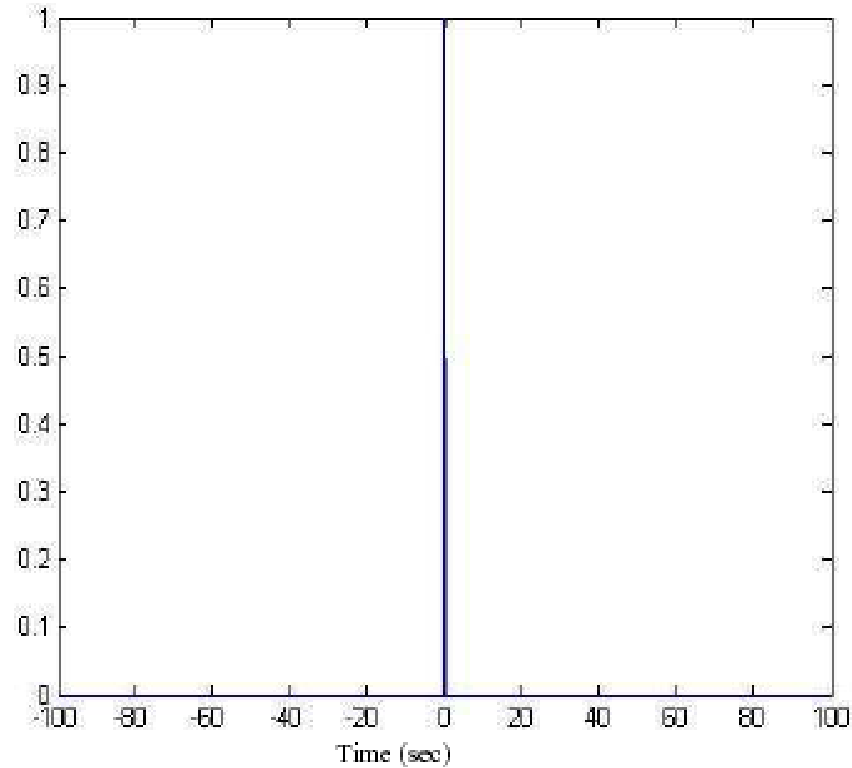
An MCNP point source with same spectrum and suitable time behaviour was placed in the central thimble on the core middle plane (SDEF routine) of the model.

A transport calculation (non kcode) in the core multiplicative media was executed on each of the three configuration with a source statistics of at least 2 millions neutron source.

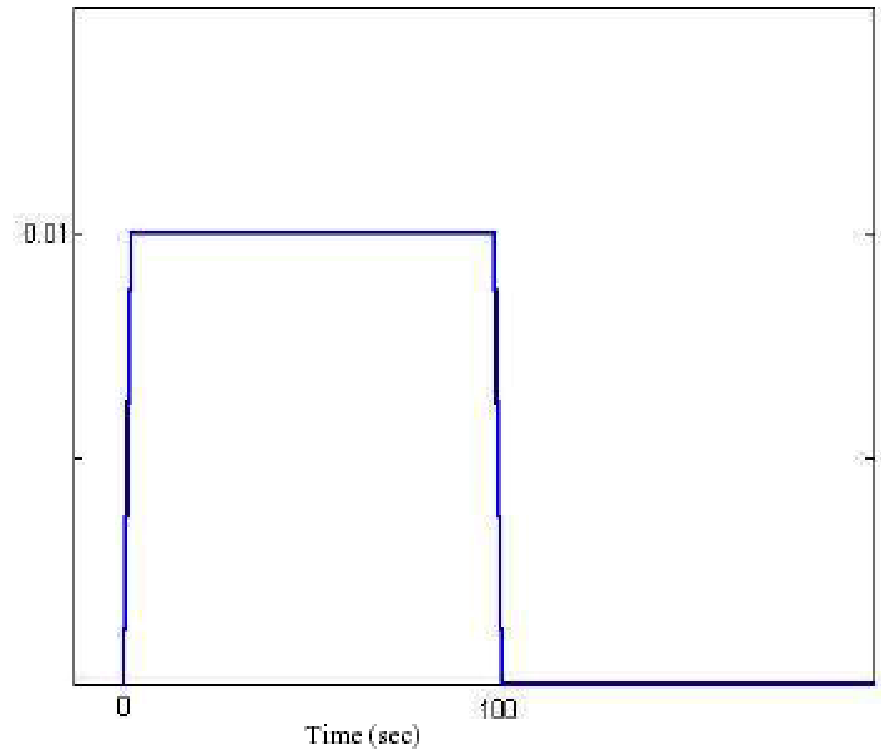
Flux and energy tallies vs time were accumulated on all fuel rods of each configuration.

All the time range of the simulation are well below the feedback effects time scale!

Test responses for TRIGA source driven system .

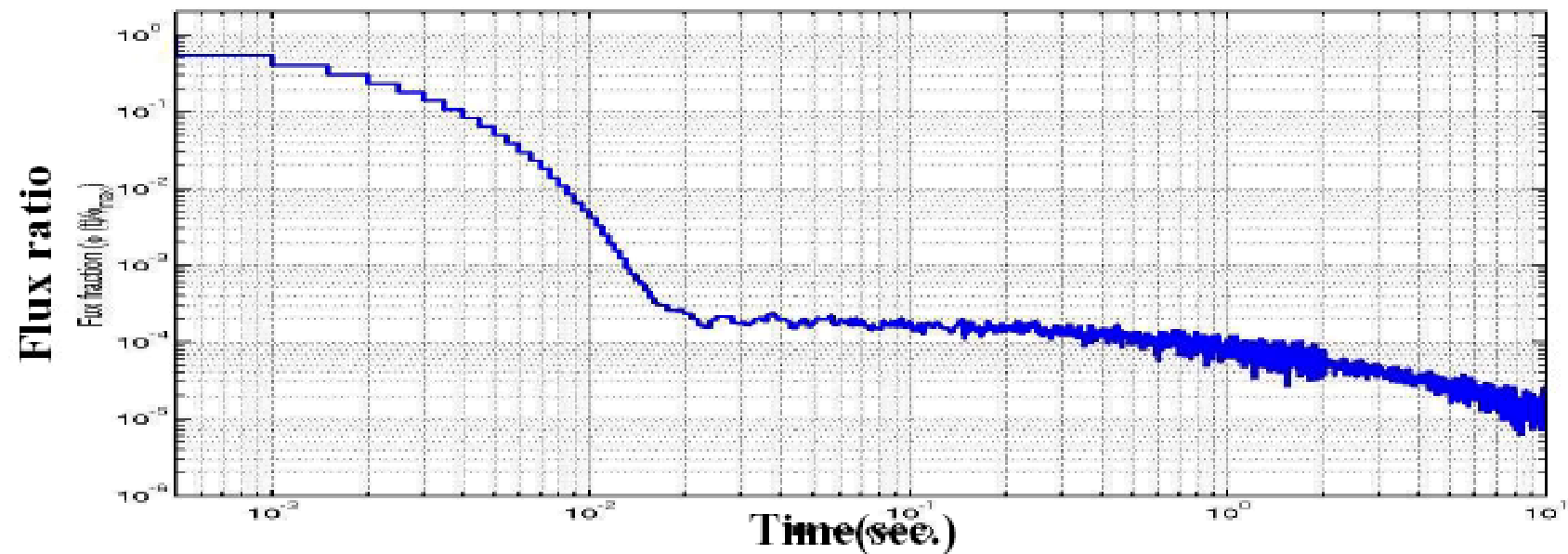
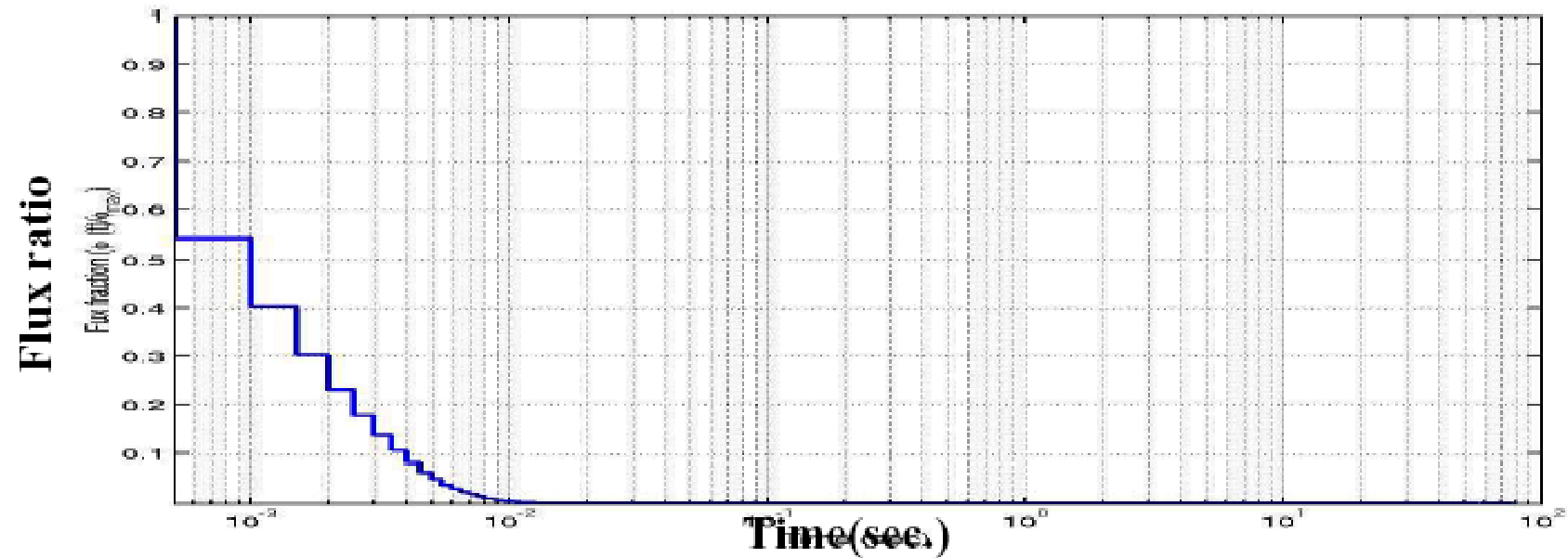


Delta Pulsed Source

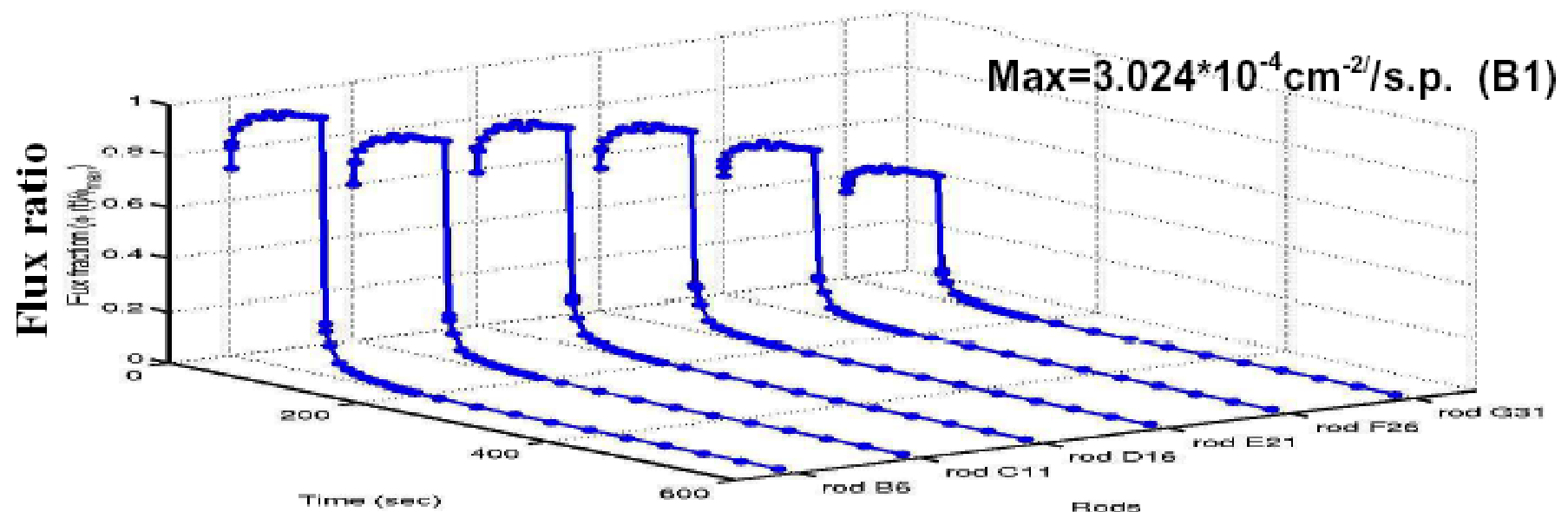
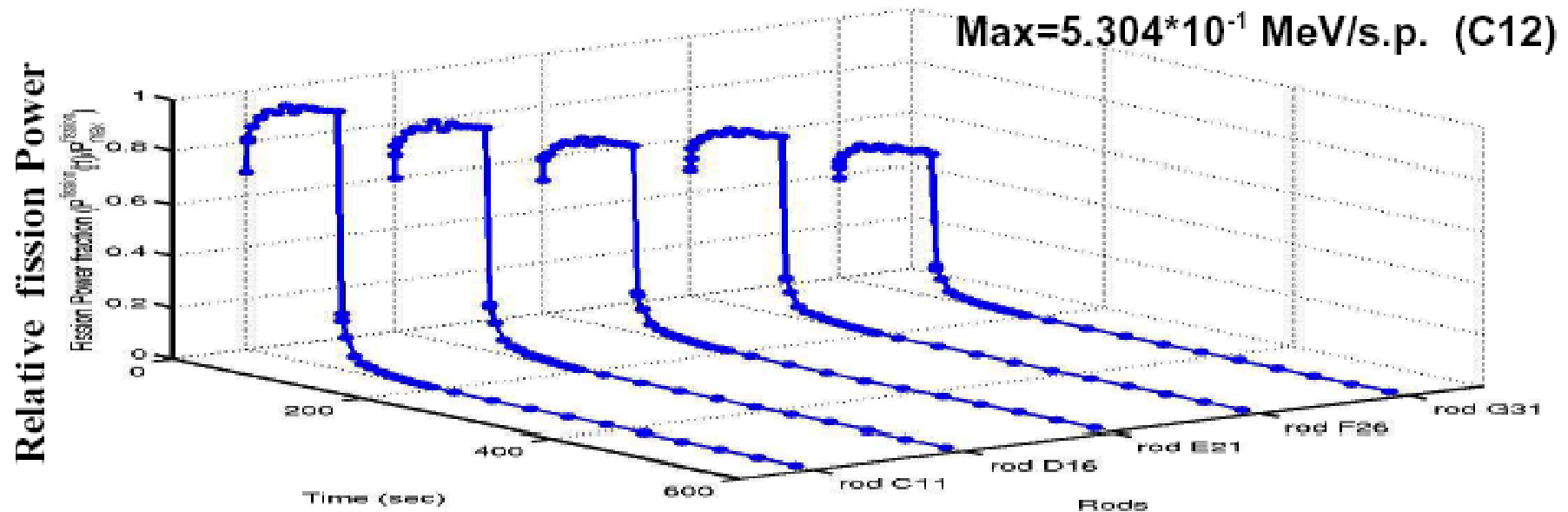


Source Jerk like

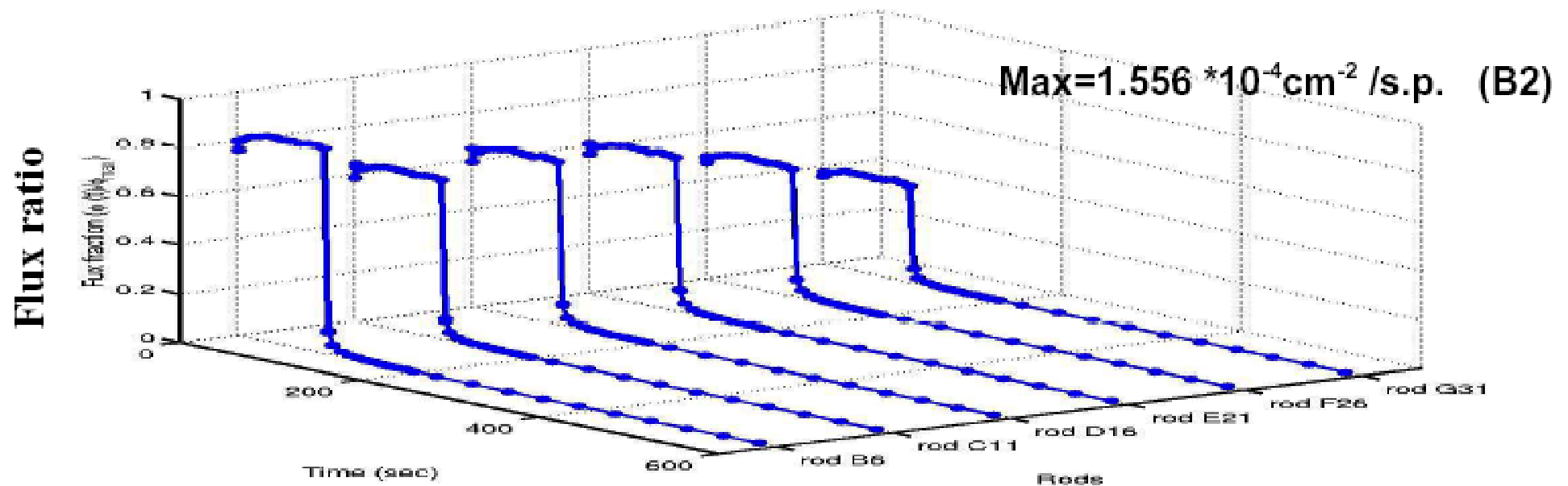
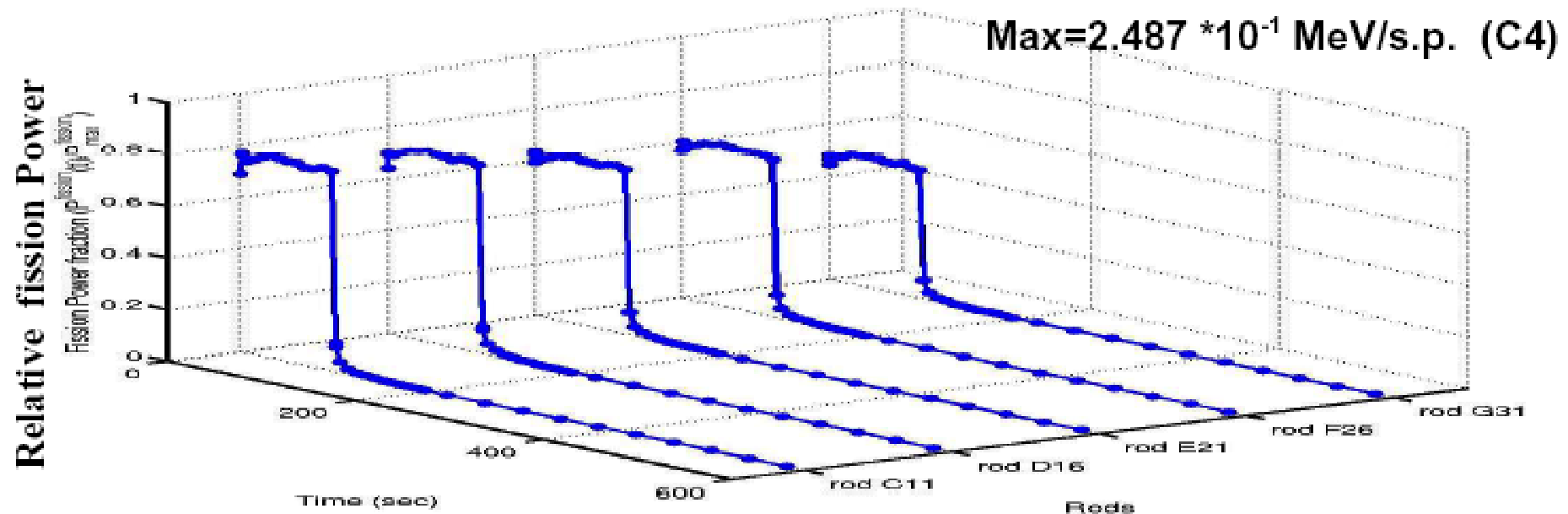
Flux response at fuel rod c1 with a time delta shaped source ($k_{eff}=0.9806$, $M=30$)



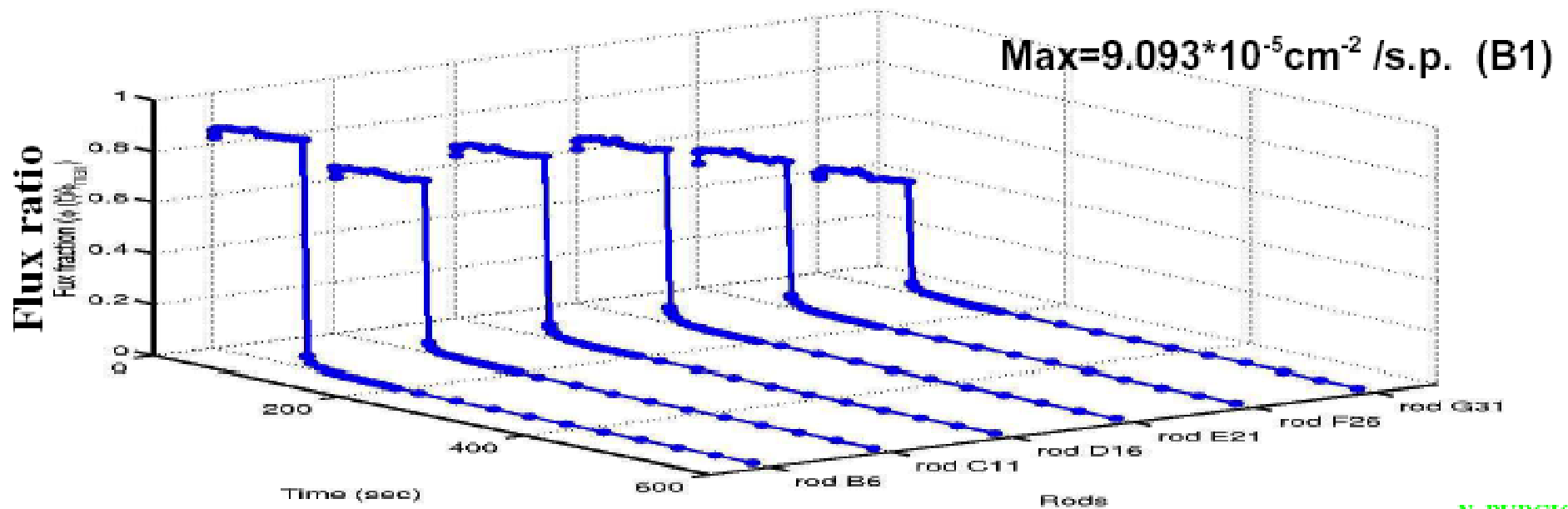
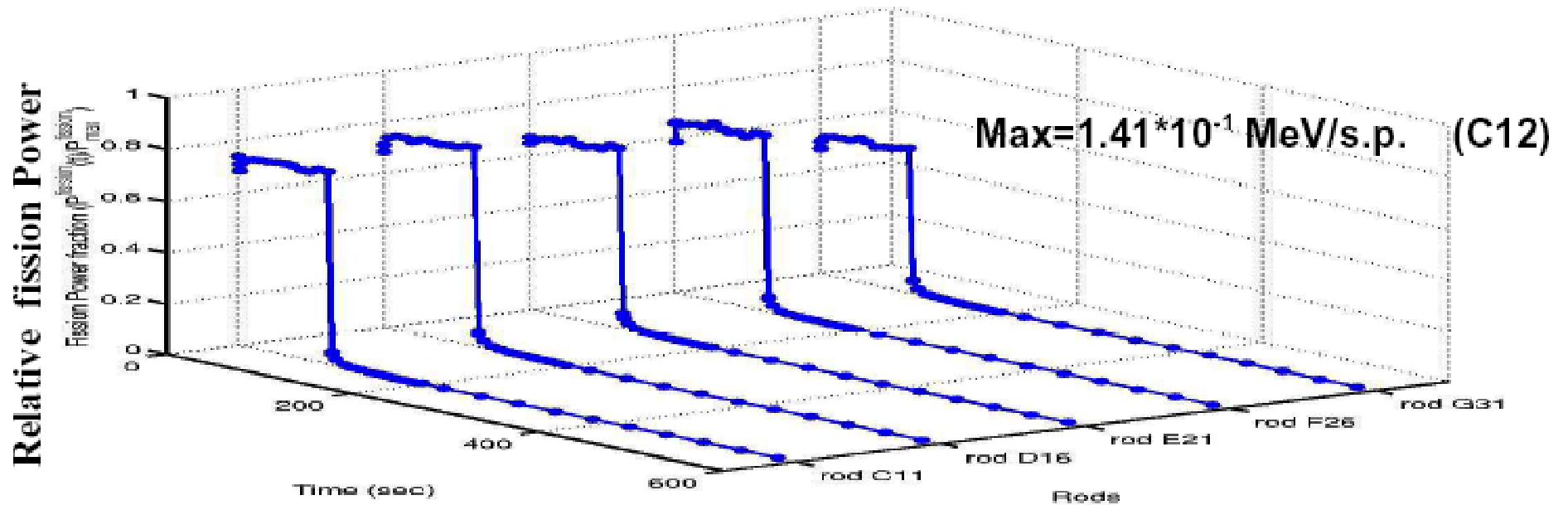
Response at rod c11,d16,e21,f26,g31 (keff=0.9806,source-jerk, M=30)



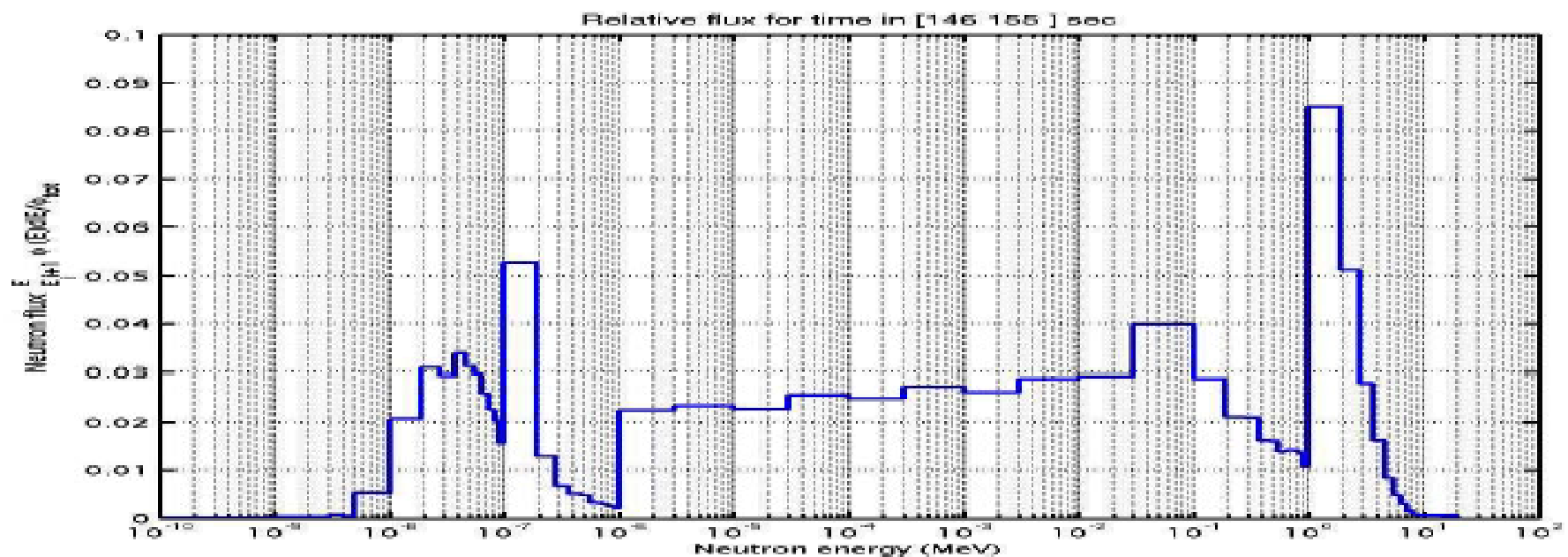
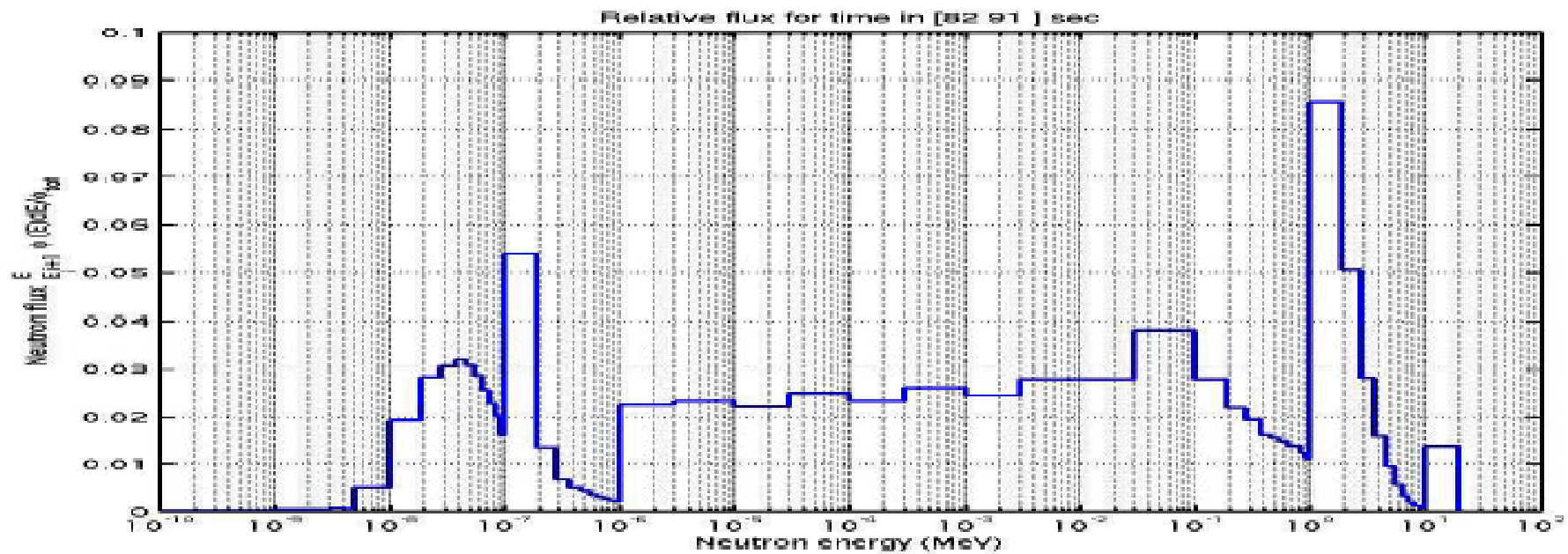
Response at rod c11,d16,e21,f26,g31 (keff=0.9602,source-jerk, M=14)



Response at rod c11,d16,e21,f26,g31 (keff=0.9308,,source-jerk, M=8)



Spectrum response at rod c1 for t=90 and 150 sec. (keff=0.9806, source-jerk)



Conclusions

- **M.C. Dynamic models could help the design of time dependent neutronic experiments.**
- **Further calculation, with improved statistical sampling, permits more quantitative considerations.**