

Medical Radionuclides and Related Nuclear Data

Part III. Therapeutic Radionuclides

Syed M. Qaim

Institut für Nuklearchemie
Forschungszentrum Jülich GmbH
D-52425 Jülich, Germany

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Topics

- General considerations
- Decay data of therapeutic radionuclides
- Commonly used production methods of important therapeutic radionuclides
- Development of new production routes
- Conclusions and perspectives

Radioisotopes for Therapy

- Differentiation between radiation therapy and internal radionuclide therapy is essential
 - Radiation therapy consists of treatment with external radiation sources
 - Internal radionuclide therapy involves the use of a radioisotope within the body either as a sealed source (brachytherapy) or via a biological pathway (endoradiotherapy).
- Of great significance in internal therapy are
 - Linear energy transfer (LET), similar to that in external radiation therapy
 - Range of ionising radiation in tissue

Internal Radionuclide Therapy

- **Brachytherapy**

(insertion of sealed sources near the tumour)

Examples: ^{192}Ir as wire
 ^{103}Pd and ^{125}I as seeds

- **Administration in cavities**

(for pain palliation)

Examples: ^{32}P colloid for arthritis
 ^{90}Y , ^{186}Re and ^{188}Re complexes for joint inflammation

- **Metabolic therapy**

(incorporation of radionuclide via a biochemical path)

Examples: ^{131}I for thyroid cancer
 ^{89}Sr , ^{186}Re and ^{153}Sm are bone seekers

- **Radioimmunotherapy**

(administration of a radionuclide chemically conjugated to antibodies)

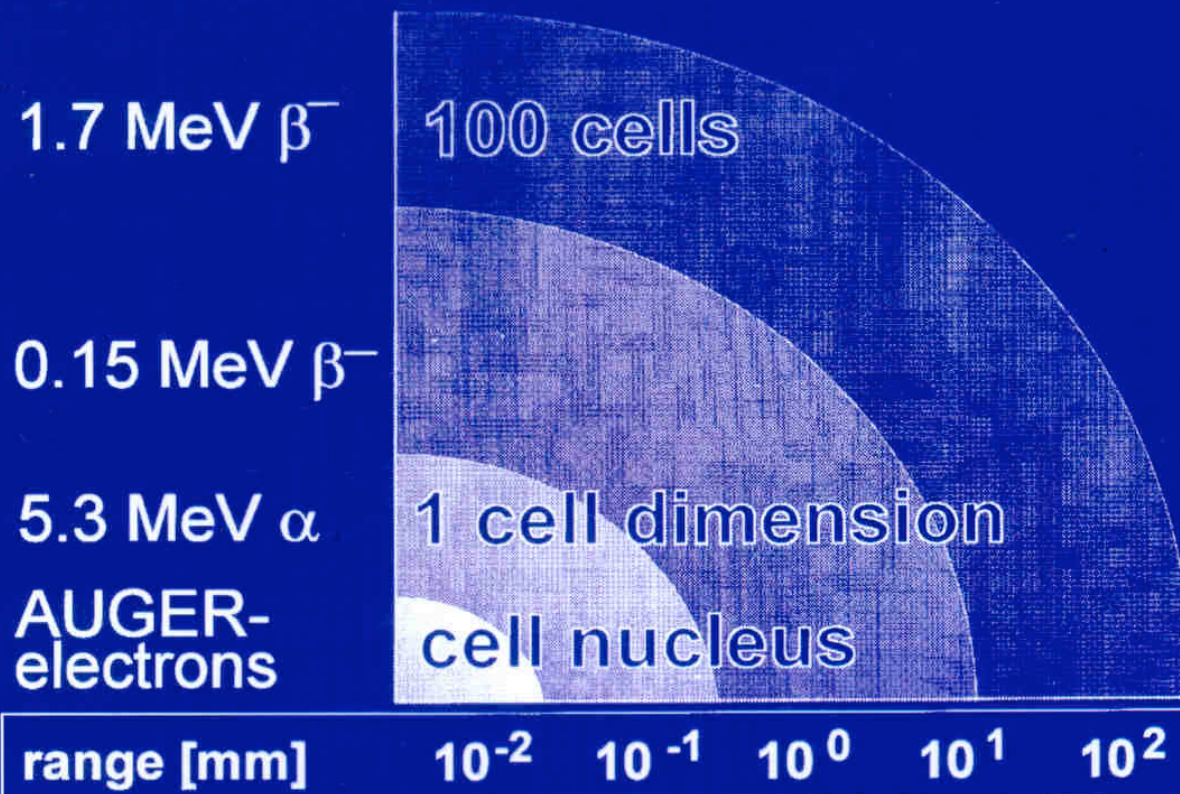
Examples: low-energy high-LET value radionuclides

Internal radionuclide therapy is a fast developing field.

Radioisotopes for Therapy

- Uptake of a radiopharmaceutical in tumour via physiological processes

Range of ionising radiation



Uptake of a Radiopharmaceutical in the Tissue

Concepts

- Radioimmuno reactions (monoclonal antibodies)
- DNA-precursors (attachment to cell nucleus)
- Receptor ligands (attachment to cell membrane)
- Tumour seeking agents (cell environment)

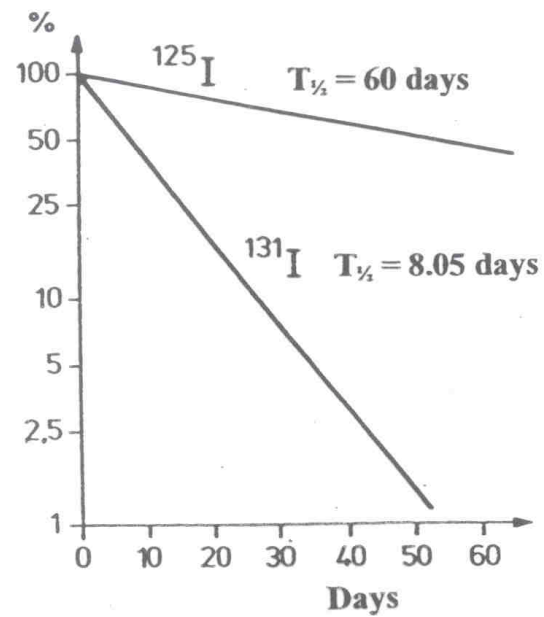
Problems

- Range of ionising radiation
- Tracer kinetics
- In-vivo stability of radiopharmaceutical
- Immuno chemical changes

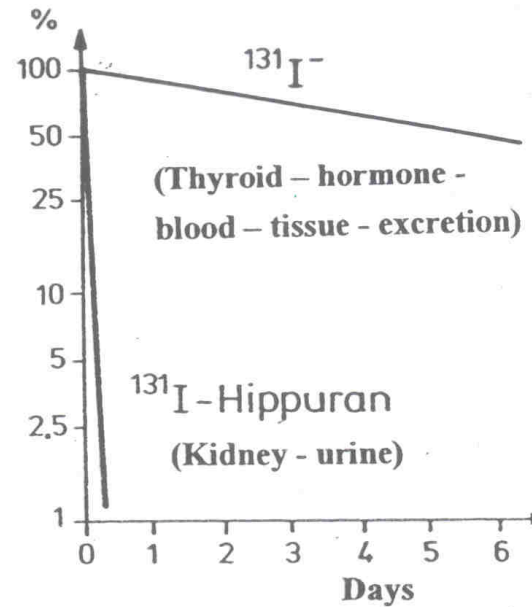
*Endoradiotherapy is a developing
field of study*

Effective Half-life (T_{eff})

- Of considerable significance in the dosimetry of target organ
 - T_{phys} (physical half-life)
 - T_{bio} (biological half-life)



T_{phys}



T_{bio}

$$\lambda_{\text{eff}} = \lambda_{\text{phys}} + \lambda_{\text{bio}}$$

$$T_{\text{eff}} = \frac{\ln 2}{\lambda_{\text{phys}} + \lambda_{\text{bio}}} = \frac{T_{\text{phys}} \cdot T_{\text{bio}}}{T_{\text{phys}} + T_{\text{bio}}}$$

Decay Data of Therapeutic Radionuclides

Distinctive Features

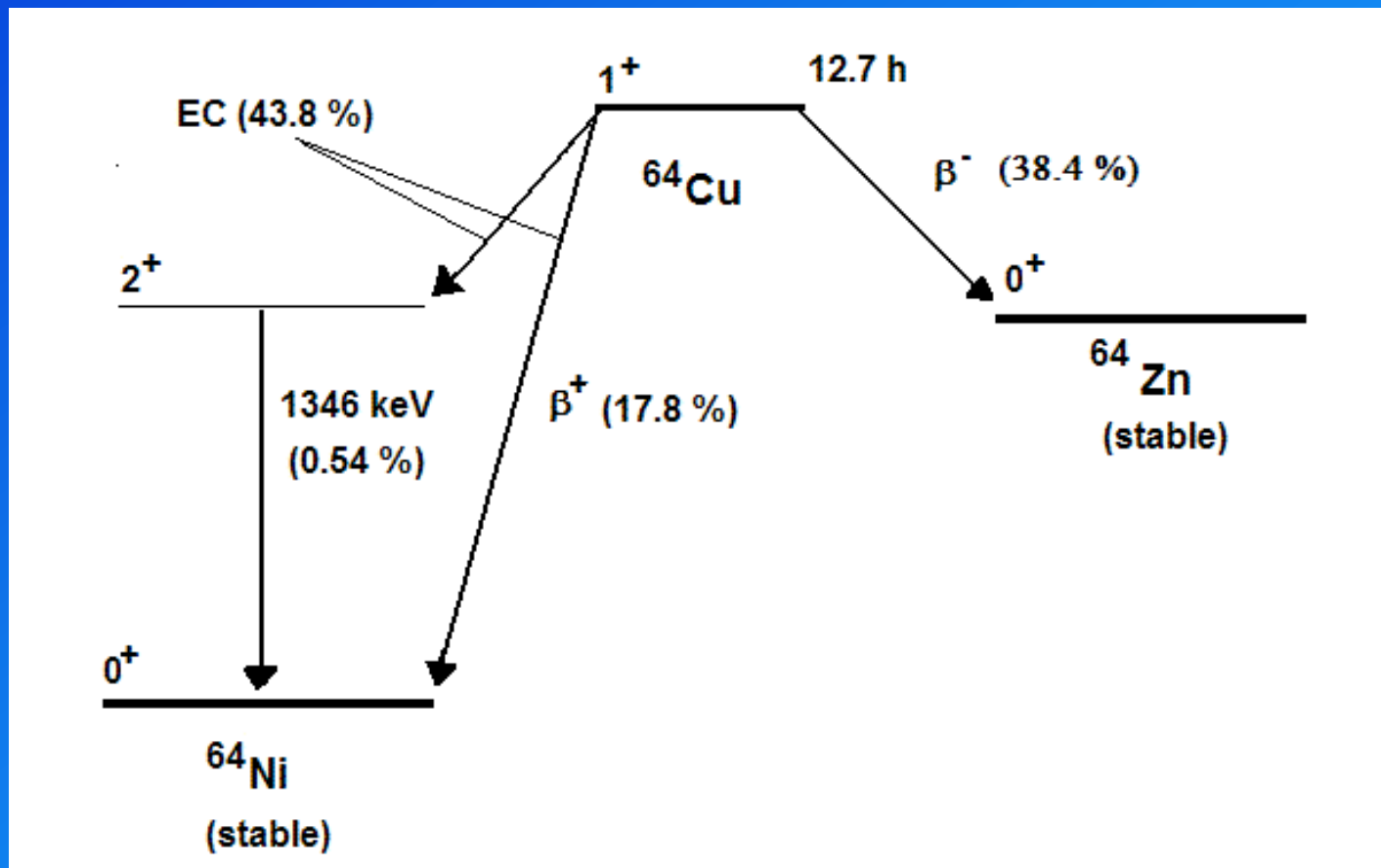
- $T_{1/2}$ relatively long
- I_{β} and I_{α} quite high
- I_{γ} rather low
- Energies of Auger and conversion electrons generally low

Status of Data

- Decay data generally well known
- Occasionally some uncertainty in I_{β} due to
 - use of impure samples
 - lack of high-precision β -ray spectroscopy
- Occasional uncertainty in low-energy electron spectra

Decay Scheme of Copper-64

- Preparation of thin, high-purity sample via the nuclear reaction $^{66}\text{Zn}(d,\alpha)^{64}\text{Cu}$
- Use of several counting methods (γ -and X-ray spectrometry; β -counting; $\gamma\gamma$ -coincidence, etc.)



Production Methods of Important Therapeutic Radionuclides

(β^- Emitters)

Nuclide	$T_{1/2}$	Production route	Nuclide	$T_{1/2}$	Production route
^{32}P	14.3 d	$^{32}\text{S}(\text{n},\text{p})$	^{153}Sm	1.9 d	$^{152}\text{Sm}(\text{n},\gamma)$
^{64}Cu	12.7 h	$^{64}\text{Ni}(\text{p},\text{n})$	^{169}Yb	32.0 d	$^{168}\text{Yb}(\text{n},\gamma)$
^{67}Cu	2.6 d	$^{68}\text{Zn}(\text{p},2\text{p})$	^{177}Lu	6.7 d	$^{176}\text{Lu}(\text{n},\gamma)$
^{89}Sr	50.5 d	$^{89}\text{Y}(\text{n},\text{p})$	^{186}Re	3.8 d	$^{186}\text{W}(\text{p},\text{n})$
^{90}Y	2.7 d	$^{90}\text{Sr}/^{90}\text{Y}$ (Generator)	^{188}Re	17.0 h	$^{188}\text{W}/^{188}\text{Re}$ (Generator)
^{131}I	8.0 d	$^{130}\text{Te}(\text{n},\gamma)^{131\text{m,g}}\text{Te} \xrightarrow{\beta^-}$ $^{235}\text{U}(\text{n},\text{f})$	^{192}Ir	73.8 d	$^{191}\text{Ir}(\text{n},\gamma)$

Production is done using both nuclear reactors and cyclotrons

Production Methods of Important Therapeutic Radionuclides (Cont'd)

Nuclide	T _{1/2}	Production route	Nuclide	T _{1/2}	Production route
<i>β⁺ Emitters</i>			<i>X-Ray/Auger Electron Emitters</i>		
⁶⁴ Cu	12.7 h	⁶⁴ Ni(p,n)	⁷⁷ Br	2.4 d	⁷⁵ As(α,2n)
⁷⁶ Br	16.0 h	⁷⁶ Se(p,n)	¹⁰³ Pd	17.0 d	¹⁰³ Rh(p,n)
¹²⁴ I	4.2 d	¹²⁴ Te(p,n)	¹¹¹ In	2.8 d	¹¹² Cd(p,2n)
<i>α Emitters</i>			¹²⁵ I	60.0 d	¹²⁴ Xe(n,γ) ¹²⁵ Xe ^{EC}
²¹¹ At	7.2 h	²⁰⁹ Bi(α,2n)			
²¹³ Bi	46 min	²²⁵ Ac/ ²¹³ Bi (Generator)			
²²⁵ Ac	10.0 d	from nuclear waste ²²⁶ Ra(p,2n)			

Increasing use of cyclotrons in production of therapeutic radionuclides

Examples of Endoradiotherapy

Thyroid therapy

^{131}I -iodide; 0.6 – 1.5 GBq; in case of benign tumour

- Biochemical concept (I-metabolism)
- Enrichment factor and uptake kinetics determined via γ -scintigraphy
- Radiation dose calculated using pharmacokinetics

Palliative therapy

(bones, joints, etc.)

$[^{32}\text{P}]$ -Phosphate; $[^{90}\text{Y}]$ -Citrate, etc.

- Tumour seeking agents
- Enrichment factors and uptake kinetics generally unknown (purely β^- emitters)
- Dosimetry is empirical

Other forms of therapy are in development.

Preparation of Radiotherapeutical

- Limited use of therapeutic radionuclides in simple ionic forms

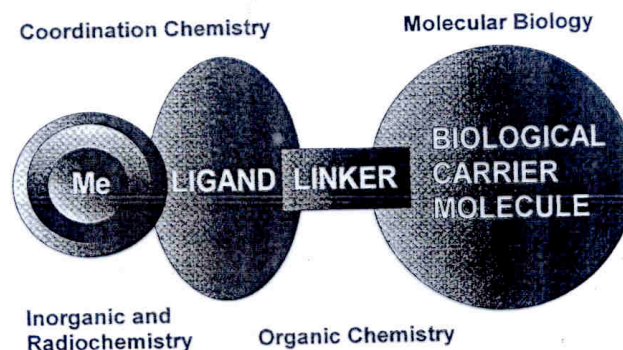
Examples: $[^{131}\text{I}]\text{I}^-$ for thyroid cancer

$[^{89}\text{Sr}]\text{Sr}^{2+}$ for bone metastases

- Most applications demand one of the following forms

- metal-ligand complex
- particles or colloids
- labelled biomolecules

- *radiohalogenation (in case of $^{80\text{m}}\text{Br}$, ^{125}I , ^{131}I or ^{211}At)*
- *coordinate bonding of metal cations via bifunctional chelators*



Multidisciplinary approach to the development of coordinately labeled radiopharmaceuticals.

Available Radiopharmaceuticals for Targeted Therapy

RADIOPHARMACEUTICAL	INDICATION	AVAILABILITY
[¹³¹ I] iodide	thyrotoxicosis, goiter thyroid carcinoma	+
Sodium [³² P] phosphate	polycythemia vera essential thrombocythemia	+
[¹³¹ I] MIBG	pheochromocytoma neuroblastoma paraganglioma, carcinoid medullary thyroid carcinoma	+
[⁸⁹ Sr] chloride	bone metastases (palliation)	+
[⁹⁰ Y] citrate	bone metastases (palliation)	+
[⁹⁰ Y] colloid	synoviorthesis (knee) malignant pleural/peritoneal effusions	+
[¹⁸⁶ Re] sulphide	synoviorthesis (medium size joints)	+
[¹⁶⁹ Er] citrate	synoviorthesis (small joints)	+
[³² P] phosphate colloid	malignant effusion	+
[¹⁸⁶ Re] HEDP	bone metastases (palliation)	clin.trial
[¹⁵³ Sm] EDTMP	bone metastases (palliation)	clin.trial
[¹³¹ I]-,[⁹⁰ Y]-,[⁶⁷ Cu]- labelled antibodies	variety of tumours	clin.trial
[⁹⁰ Y] particles	hepatoma	clin.trial
[¹³¹ I] lipiodol	liver malignancy	clin.trial

(+ = available for routine use, clin. trial = under investigation)

Development of New Routes for Production of Novel Therapeutic Radionuclides

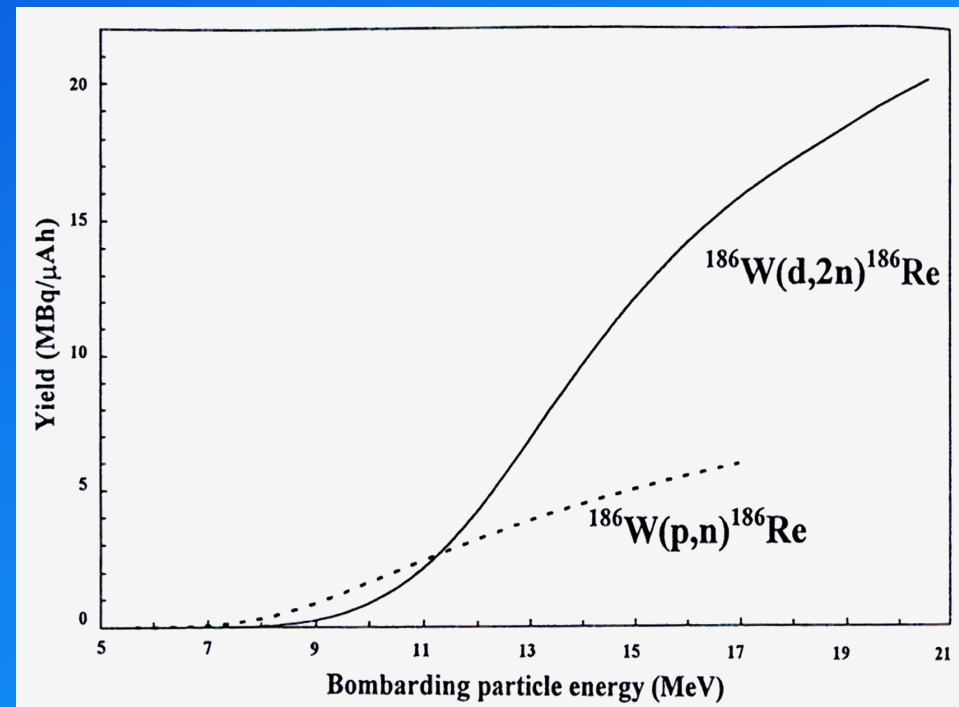
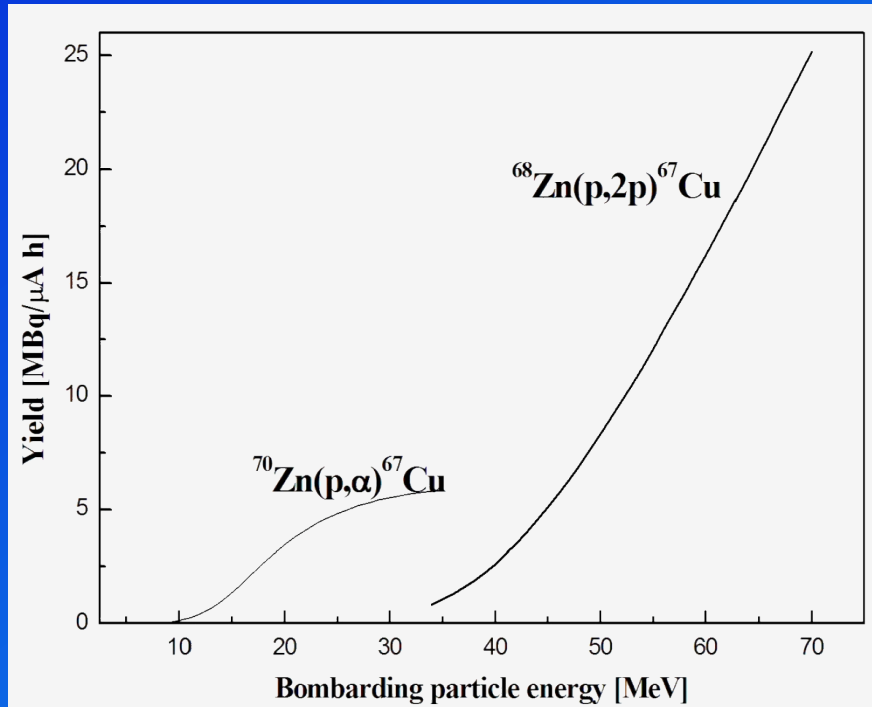
Aims

- Enhance the yield, purity and specific activity of presently used radionuclides
- Establish novel or future-oriented technology
- Development of novel therapeutic radionuclides
- Improve internal radiation dose dosimetry

New Trends

- Low-range high-intensity radiation emitters (α -particles, low-energy electrons, X-rays)

Comparative Studies on Production of Copper-67 and Rhenium-186



- *Different production yields using different reactions*
- *Choice of production method depends on the available projectile and its energy*

Routes for Production of ^{64}Cu

Production route	Suitable energy range [MeV]	Integral yield [MBq/ $\mu\text{A}\cdot\text{h}$]
$^{64}\text{Zn}(\text{n},\text{p})^{64}\text{Cu}$	Fission spectrum	14.5*
$^{64}\text{Ni}(\text{d},2\text{n})^{64}\text{Cu}$	19 \rightarrow 15	389
$^{64}\text{Ni}(\text{p},\text{n})^{64}\text{Cu}$	12 \rightarrow 9	241
$^{\text{nat}}\text{Zn}(\text{d},\text{x})^{64}\text{Cu}$	25 \rightarrow 10	50
$^{66}\text{Zn}(\text{d},\alpha)^{64}\text{Cu}$	13 \rightarrow 7	6.6
$^{68}\text{Zn}(\text{p},\alpha\text{n})^{64}\text{Cu}$	35 \rightarrow 20	~100

•Activity/mg Zn at $\Phi_{\text{n}} = 8.7 \times 10^{13} \text{ n cm}^{-2} \text{ s}^{-1}$ for 150 h

$^{64}\text{Ni}(\text{p},\text{n})^{64}\text{Cu}$ reaction is method of choice

Specific Activity of Reactor Produced Radionuclides

often depends on

- isotopic abundance of target isotope
 - (n,γ) cross section
 - neutron flux density
-
- Demand on specific activity depends upon the mode of accumulation of the labelled therapeutical at the tumour surface
 - If receptor-based interactions are involved, a high dilution with the stable isotope cannot be tolerated

Rare earth isotopes (lanthanoids) can be advantageously produced since the (n,γ) cross section is high.

- Trivalent lanthanoids show affinity for tumours

Improving the Specific Activity

Example: Samarium-153

$(T_{1/2} = 46.3 \text{ h}, E_{\beta^-} = 0.8 \text{ MeV}, I_{\beta^-} = 100 \%)$

- *An important therapeutic radionuclide of increasing significance*

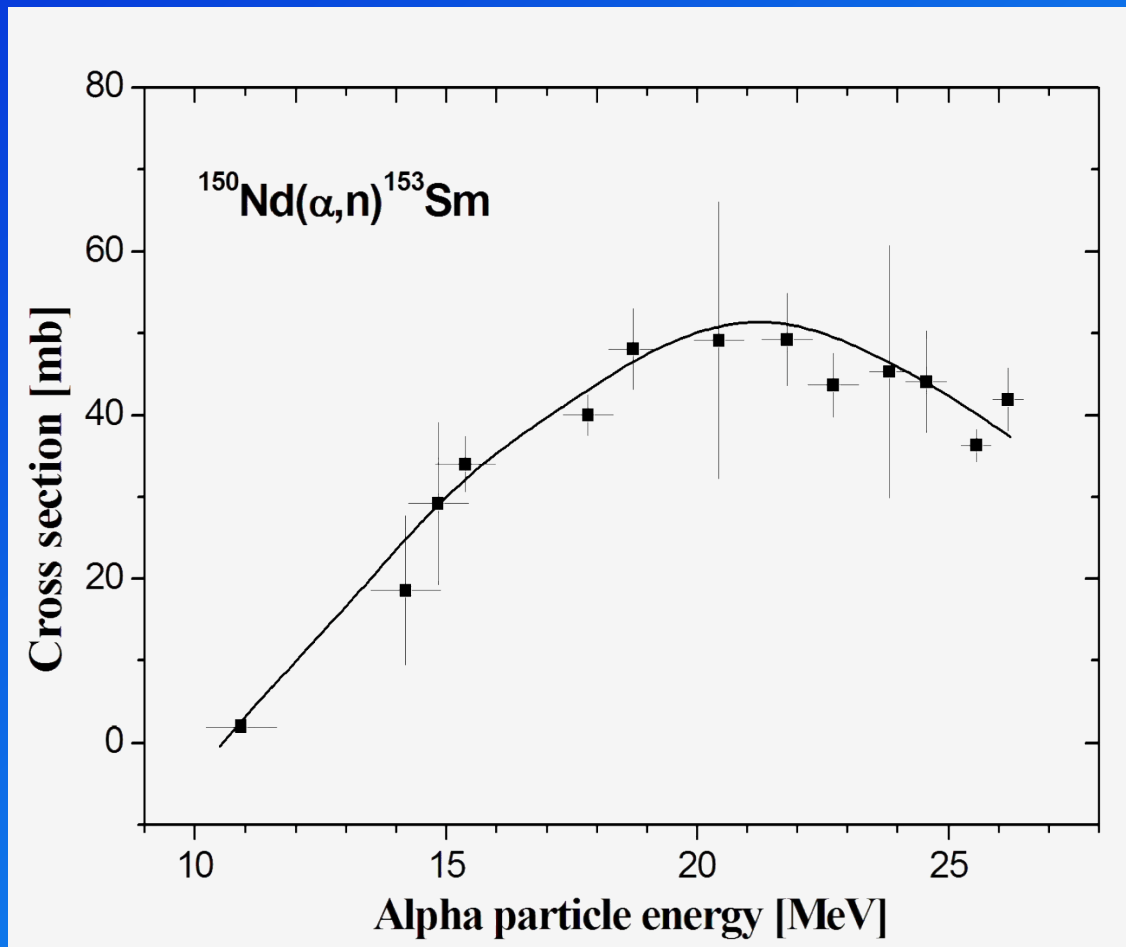
Commonly used
production method:



- ▶ high yield
- ▶ low specific activity

Sm-153

New Route: $^{150}\text{Nd}(\alpha, n)^{153}\text{Sm}$



^{153}Sm -yield:

($E_{\alpha} = 25 \rightarrow 15 \text{ MeV}$)

2.2 MBq/ $\mu\text{A}\cdot\text{h}$

- Production in GBq amounts possible
- Product: „no-carrier-added“

Palladium-103

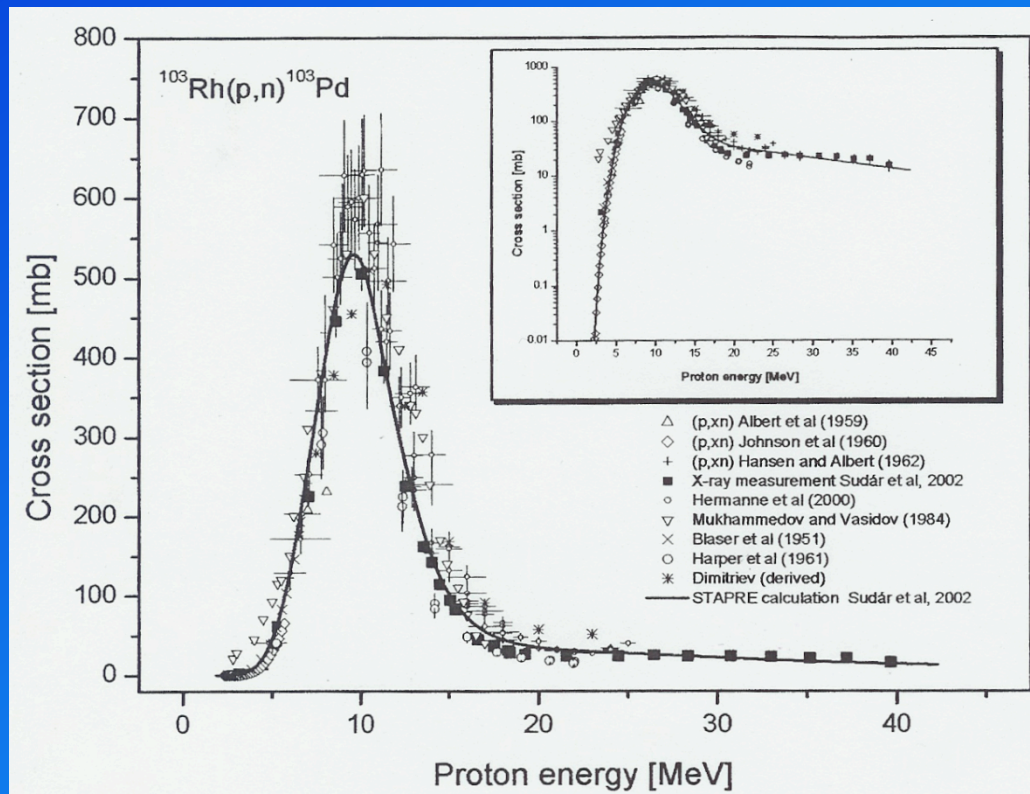
($T_{1/2} = 17.0$ d; EC = 100 %; X-rays)

- *Extremely important radionuclide for prostate cancer treatment*

Measurement: Stacked-foil technique; X-ray-spectrometry

Theory: Hauser-Feshbach calculation (code: STAPRE)

Excitation Function



^{103}Pd -yield:

($E_p = 14 \rightarrow 7$ MeV)

6.6 MBq/ $\mu\text{A}\cdot\text{h}$

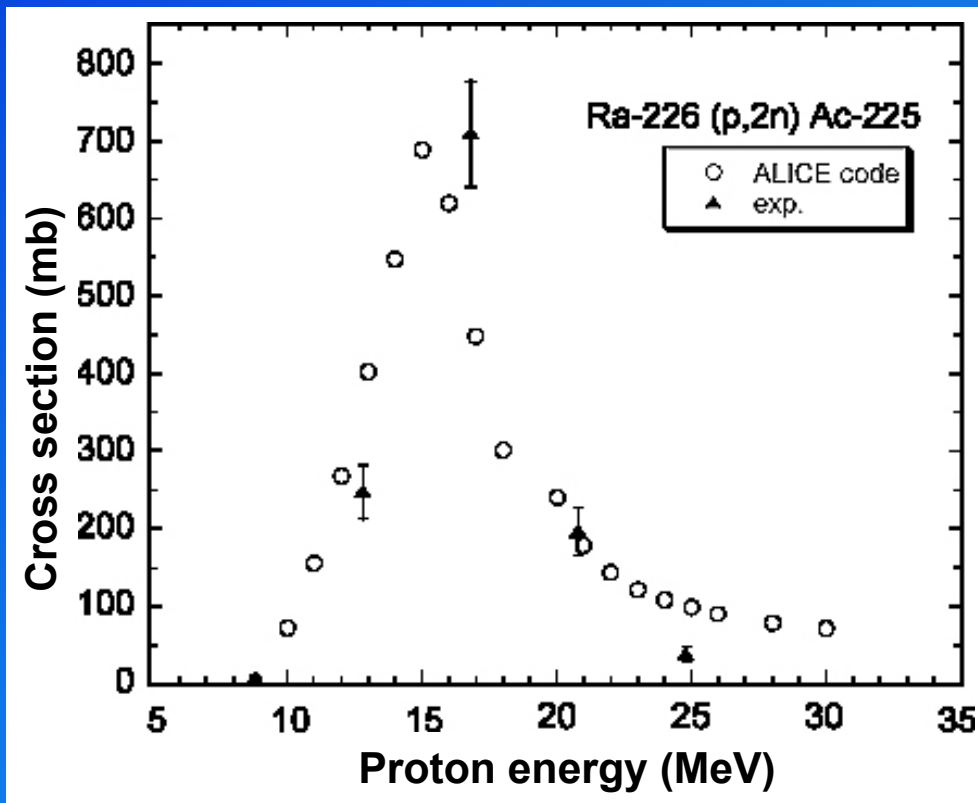
Establishing Novel Technology

Example: Actinium-225

($T_{1/2} = 10.0$ d, $E_{\alpha} = 5.8$ MeV, $I_{\alpha} = 100$ %)

Presently used Route: Separation from nuclear waste (Th-229)

New Route: $^{226}\text{Ra}(p,2n)^{225}\text{Ac}$



- Radioactive target
- Chemical processing very demanding
- **^{225}Ac -yield:**
($E_p = 22 \rightarrow 10$ MeV)
ca. 7 MBq/ $\mu\text{A}\cdot\text{h}$
- Production in GBq amount possible

Studies Related to Future Technologies

Example: Measurement of $\sigma(n,p)$ with 14 MeV „d(Be)-breakup“-neutrons
(Simulation study for production at a spallation neutron source)

Therapeutic radionuclide	Nuclear reaction	Reaction cross section (mb)	
		Fission neutrons	14 MeV d(Be) neutrons
^{32}P	$^{32}\text{S}(n,p)^{32}\text{P}$	69	152
^{67}Cu	$^{67}\text{Zn}(n,p)^{67}\text{Cu}$	1.07	5.13
^{89}Sr	$^{89}\text{Y}(n,p)^{89}\text{Sr}$	0.31	0.91
^{153}Sm	$^{153}\text{Eu}(n,p)^{153}\text{Sm}$	0.015	0.26

- ***Spallation neutron source would be more suitable than a fission reactor for production of therapeutic radionuclides via (n,p) process***

Development of Novel Therapeutic Radionuclides

Example: ^{149}Tb ($T_{1/2} = 4.1 \text{ h}$, $E_{\alpha} = 3.97 \text{ MeV}$, $I_{\alpha} = 16.7 \%$)

- *Potentially interesting radionuclide due to low α -particle energy*

Production Methods

- (a) $^{\text{nat}}\text{Gd}(p,xn)^{149}\text{Tb}$ ($E_p > 100 \text{ MeV}$)
- (b) $^{165}\text{Ho}(p,\text{spall})^{149}\text{Tb}$ ($E_p > 200 \text{ MeV}$)
Mass separation
- (c) $^{141}\text{Pr}(^{12}\text{C},4n)^{149}\text{Tb}$ ($E > 120 \text{ MeV}$)

Production reactions are rather exotic; the yields are low; intensive chemical processing or mass separation is essential

Development of Novel Therapeutic Radionuclides

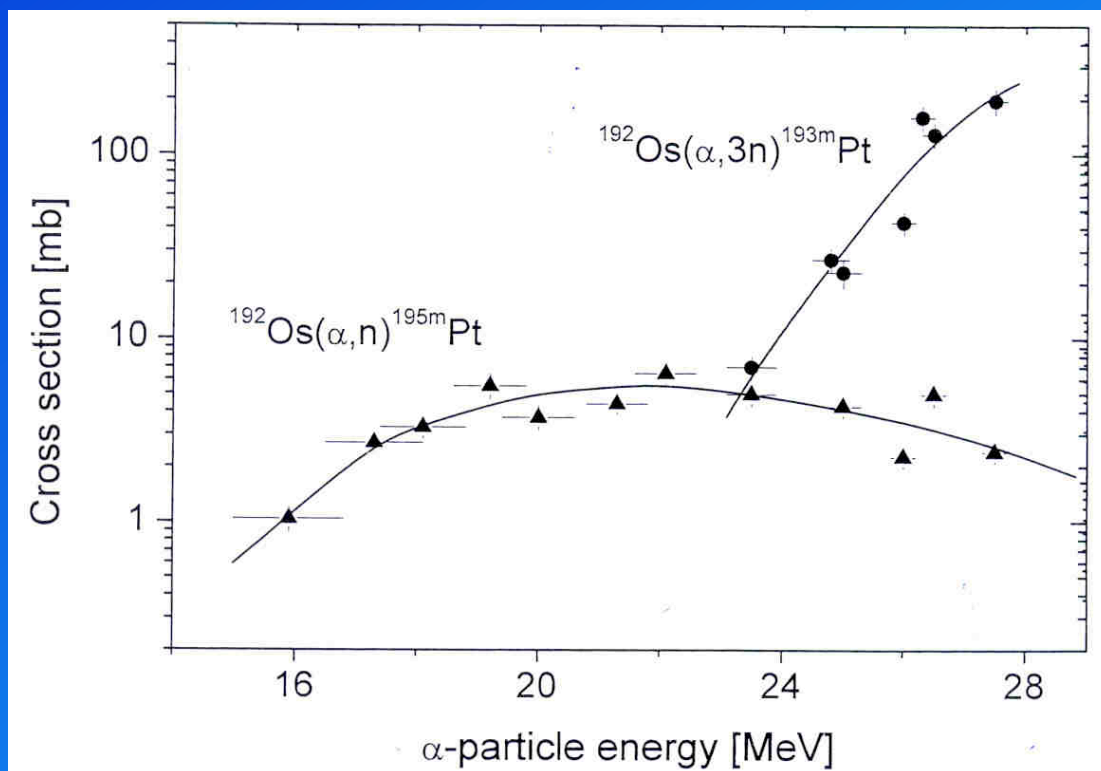
Examples: ^{193m}Pt ($T_{1/2} = 4.33$ d; Auger electrons ~ 33 per decay)

^{195m}Pt ($T_{1/2} = 4.03$ d; Auger electrons ~ 26 per decay)

High-spin isomers

Production Method: α -particles on enriched ^{192}Os

Excitation Functions



^{195m}Pt -yield:

$E_{\alpha} = 24 \rightarrow 18$ MeV

0.013 MBq/ $\mu\text{A}\cdot\text{h}$

^{193m}Pt -yield:

$E_{\alpha} = 28 \rightarrow 24$ MeV

0.25 MBq/ $\mu\text{A}\cdot\text{h}$

Production of high-specific activity ^{193m}Pt in sufficient quantity feasible

Combination of PET and Endoradiotherapy

- Internal radiotherapy using a pure β^- emitter (^{89}Sr , ^{90}Y , etc.) entails rather uncertain radiation dosimetry
- Mixing an isotopic β^+ emitter with the therapeutic radionuclide allows a better therapy planning through PET measurement of the β^+ emitter.

Examples : ^{86}Y (^{90}Y)

^{83}Sr (^{89}Sr)

Production : $^{86}\text{Sr}(\text{p},\text{n})^{86}\text{Y}$

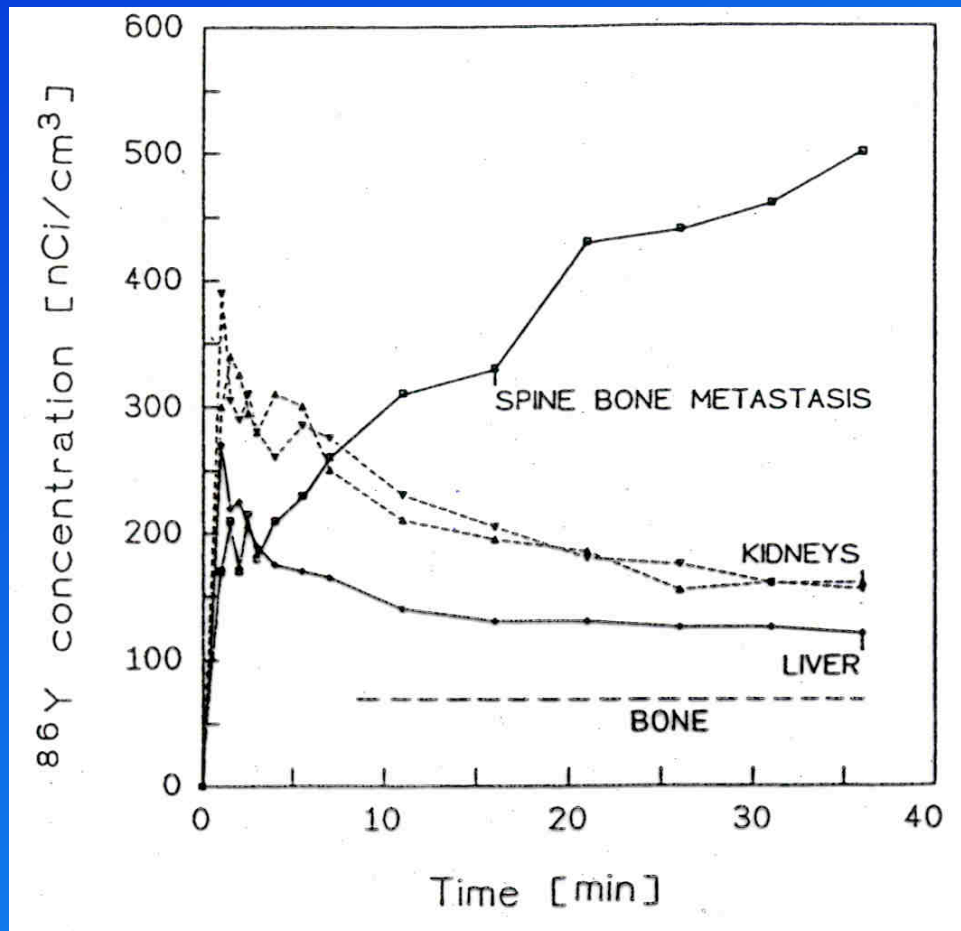
$^{85}\text{Rb}(\text{p},3\text{n})^{83}\text{Sr}$

Production of the positron emitter is done using a cyclotron

Combination of PET and Endoradiotherapy

(Determination of Uptake Kinetics and Enrichment Factor)

- Addition of β^+ emitting ^{86}Y analogue to the therapy nuclide ^{90}Y
- Pharmacokinetic data obtained using PET



- Exponential uptake of $[^{86}\text{Y}]$ -citrate in bone metastasis
 - Enrichment factor (bone metastasis/normal bone): 8 (at 5h)

Accurate dosimetry and therapy planning possible

Conclusions and Perspectives

- Decay data of prime importance in choosing a therapeutic radionuclide; status of data generally good; more precise information on low-energy electrons may be needed.
- Production of commonly used therapeutic radionuclides mainly done using a nuclear reactor; strong need to enhance the specific activity. Some therapeutic radionuclides produced at accelerators.
- New demands related to low-energy high intensity radiation emitters. Considerable research and development work needed to produce them in required quality and quantity.
- Endoradiotherapy (ERT) is a fast developing field and has great potential. New chemical and biochemical concepts and ideas on tumour targeting are needed. A combination of PET and ERT leads to better therapy planning.