




Neutron Inelastic Cross Section Measurement at the GAINS Spectrometer

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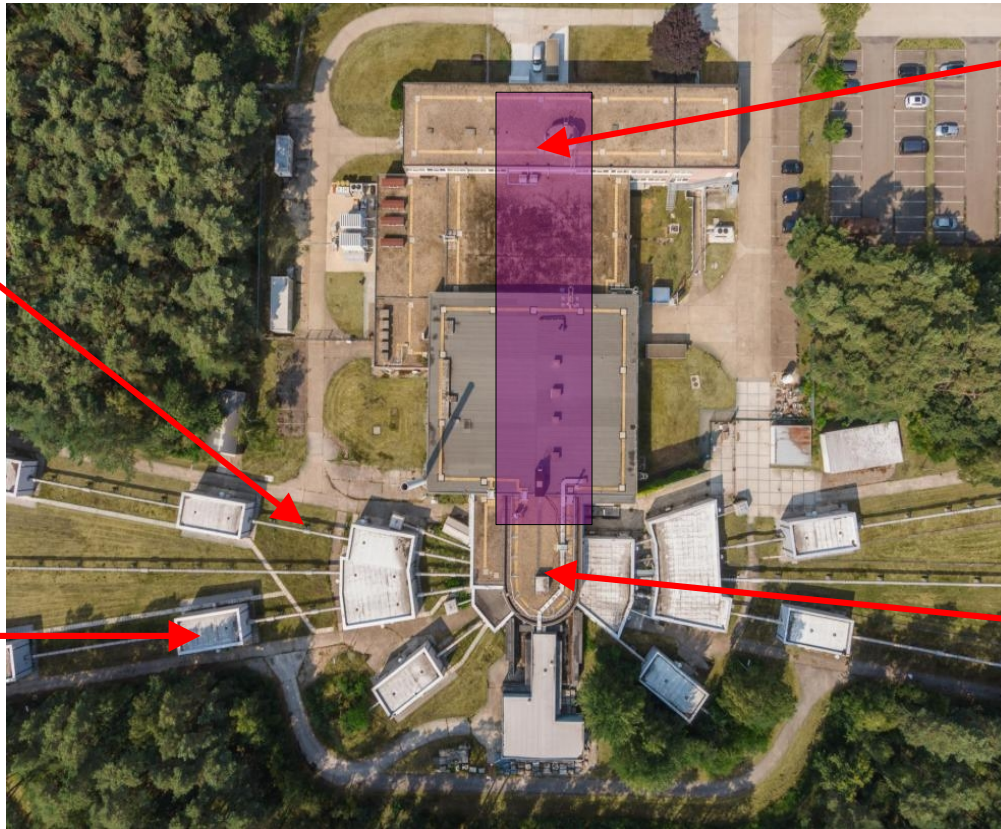


Photo is taken from
<https://joint-research-centre.ec.europa.eu>

GELINA Facility

Beam Tube

Electron Accelerator



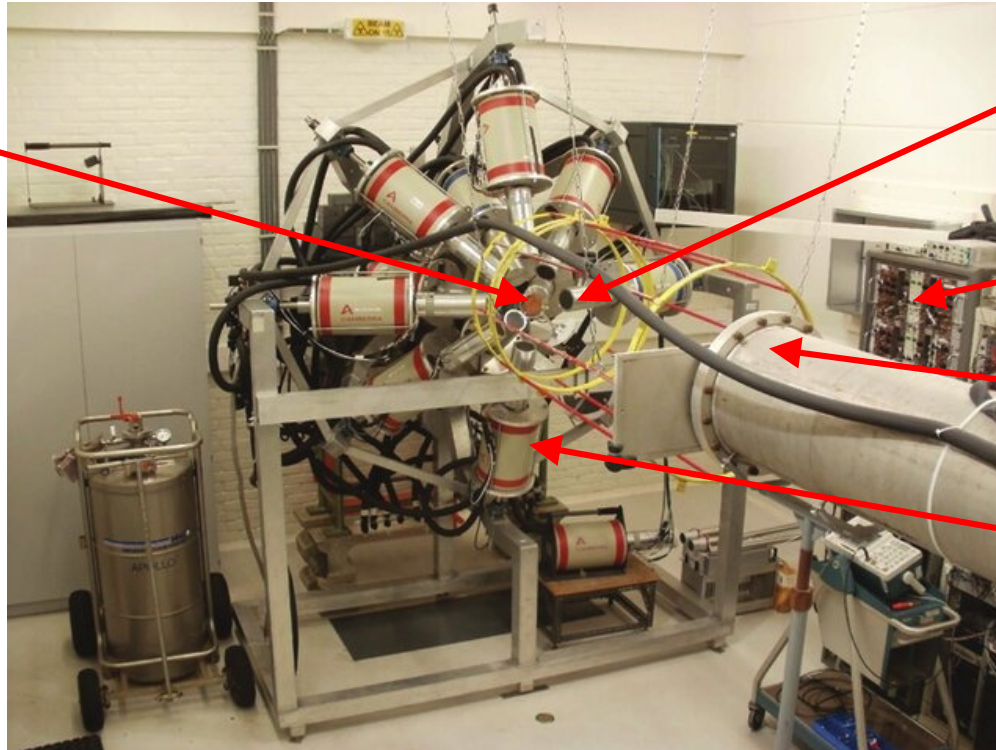
Experiment Room

Uranium target

The GAINS Spectrometer

GAINS: Gamma Array for Neutron Inelastic Scattering

Target Nuclei



HPGe Detector

Electronic Rack (High
 Voltage, Filters, etc.)

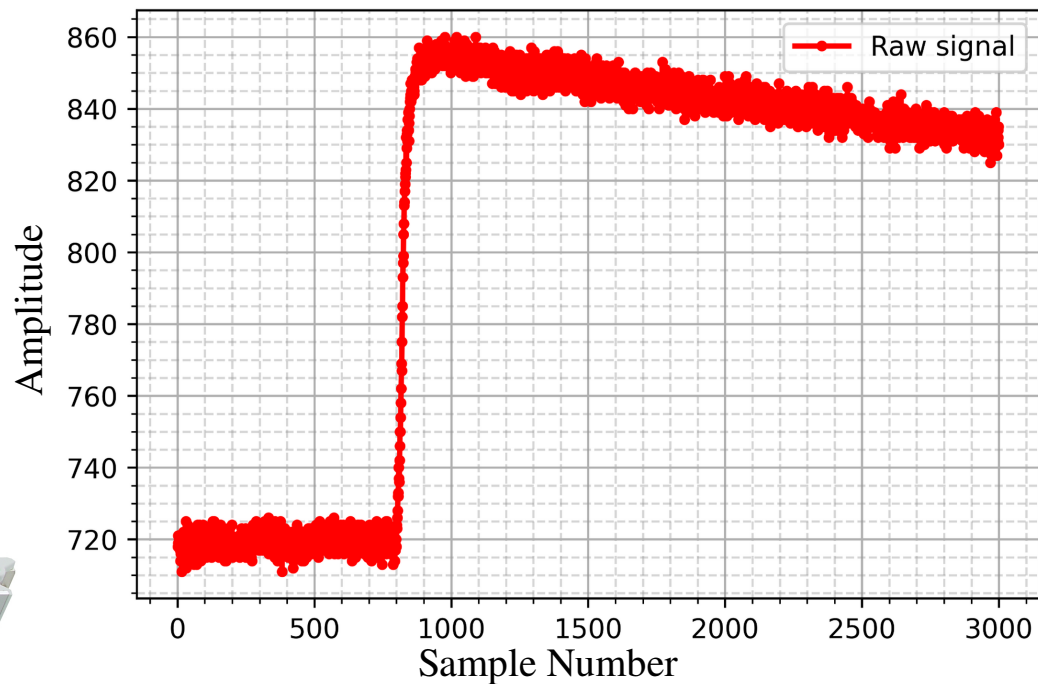
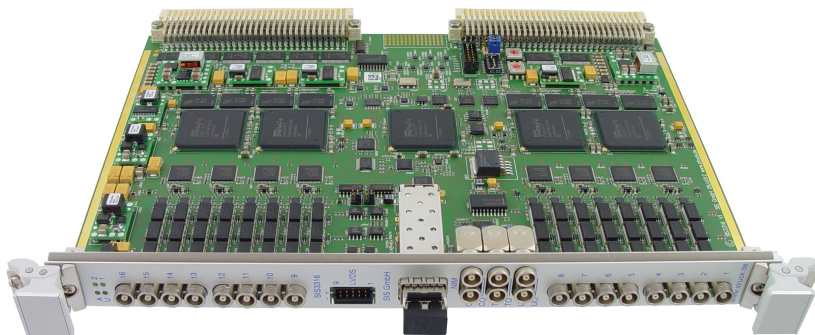
Neutron Beam Dump

Liquid Nitrogen
 (for HPGe Detector)

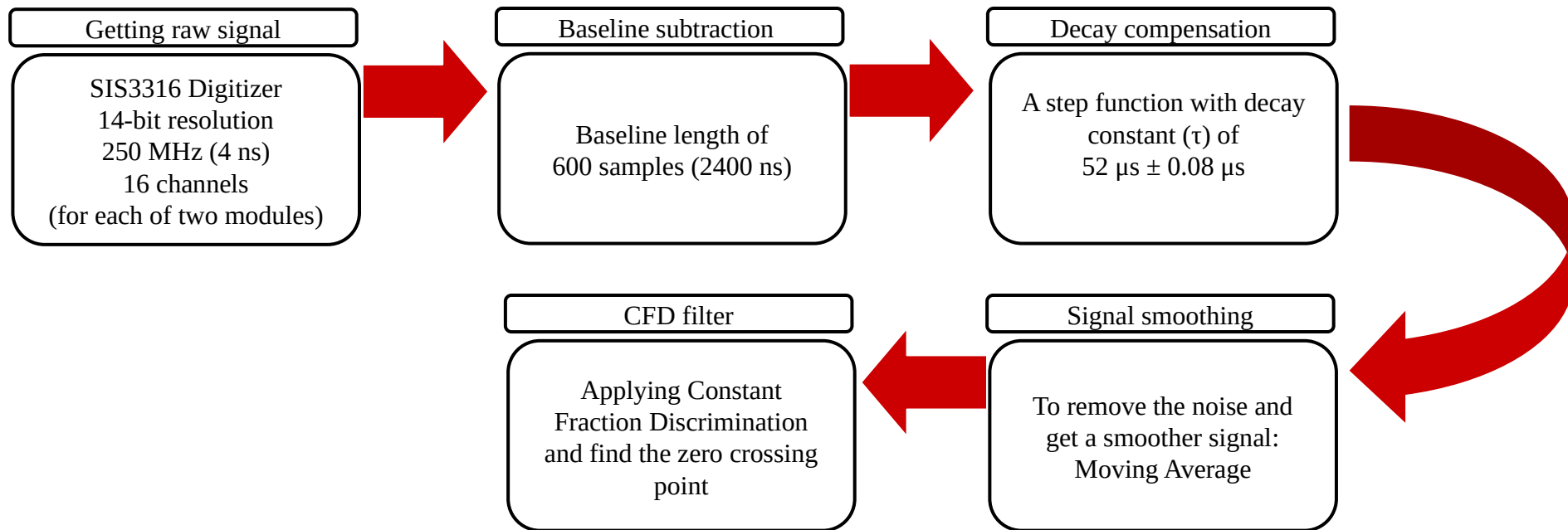
Upgrading of Data Acquisition System

Struck SIS3316-250-14

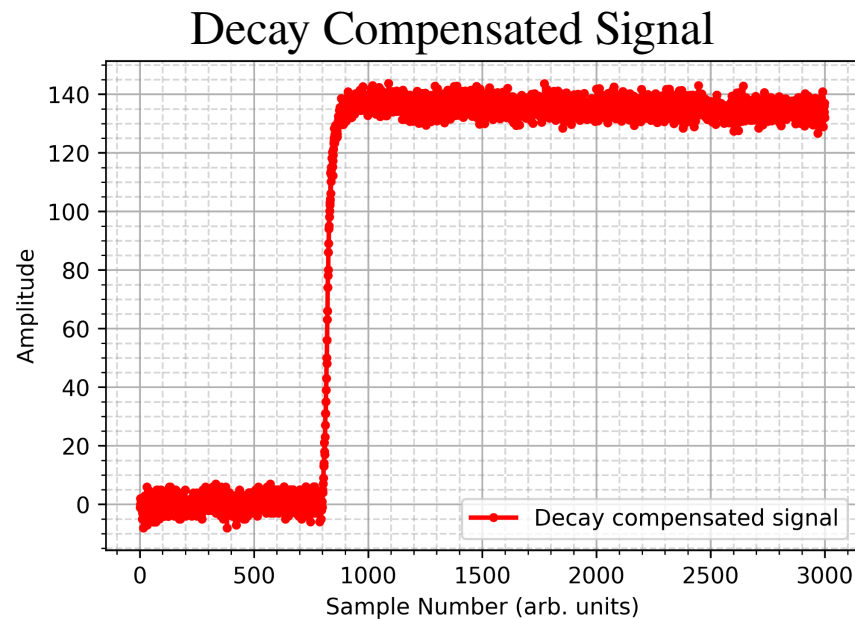
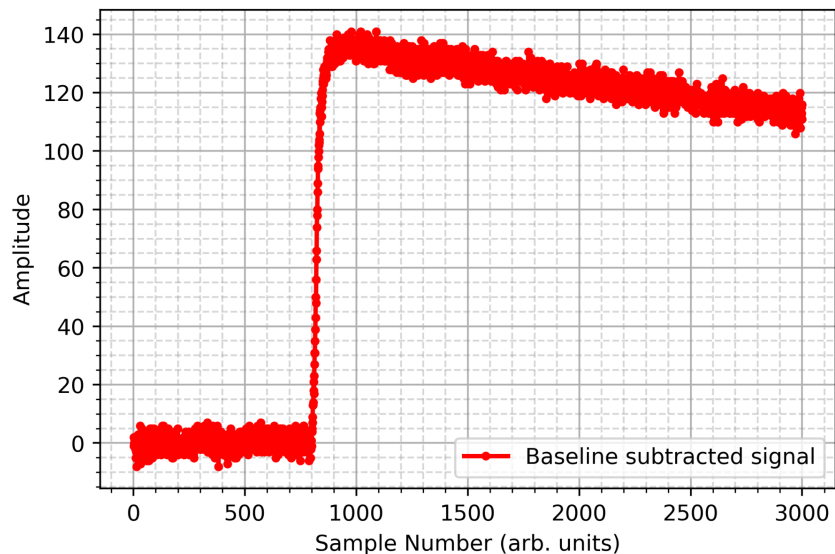
- 16 Channels
- 14-bit Resolution
- 250 MHz Sampling Rate



Signal Processing Chain for Timing Measurements



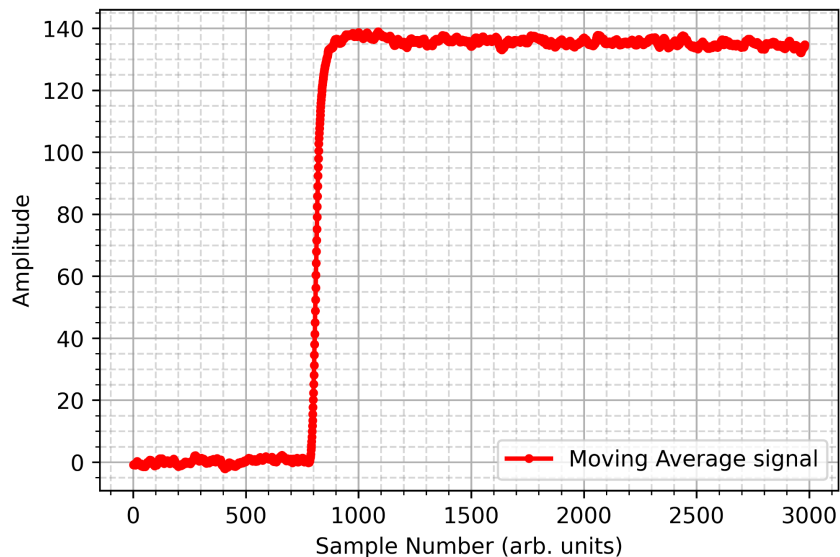
Signal Processing Chain for Timing Measurements



$$\text{Decay compensation filter: } \Psi_{i+1} = \Psi_i + e^{\frac{\Delta t}{2\tau}} (x_{i+1} - x_0) - e^{-\frac{\Delta t}{2\tau}} (x_i - x_0)$$

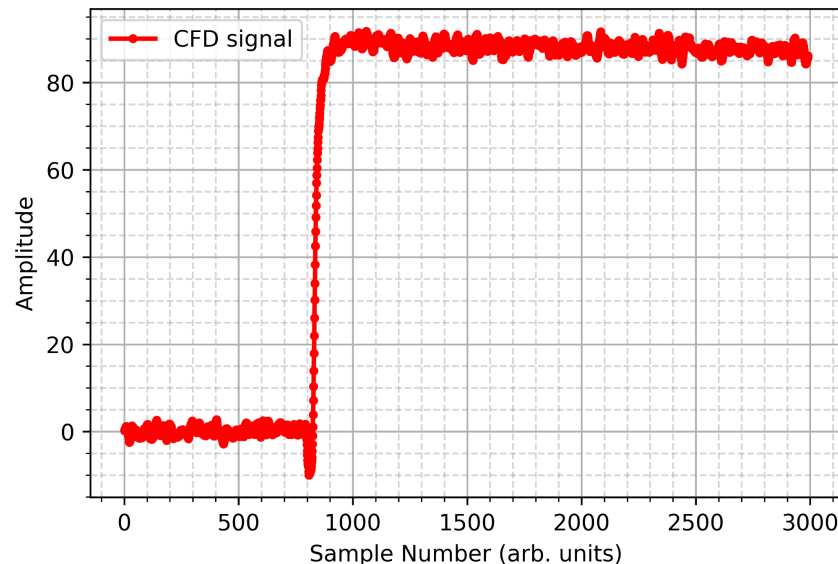
Signal Processing Chain for Timing Measurements

Smoothed Signal



Moving average filter:
$$\Phi_i = \frac{1}{L} \sum_{j=0}^{L-1} x_{i+j}$$

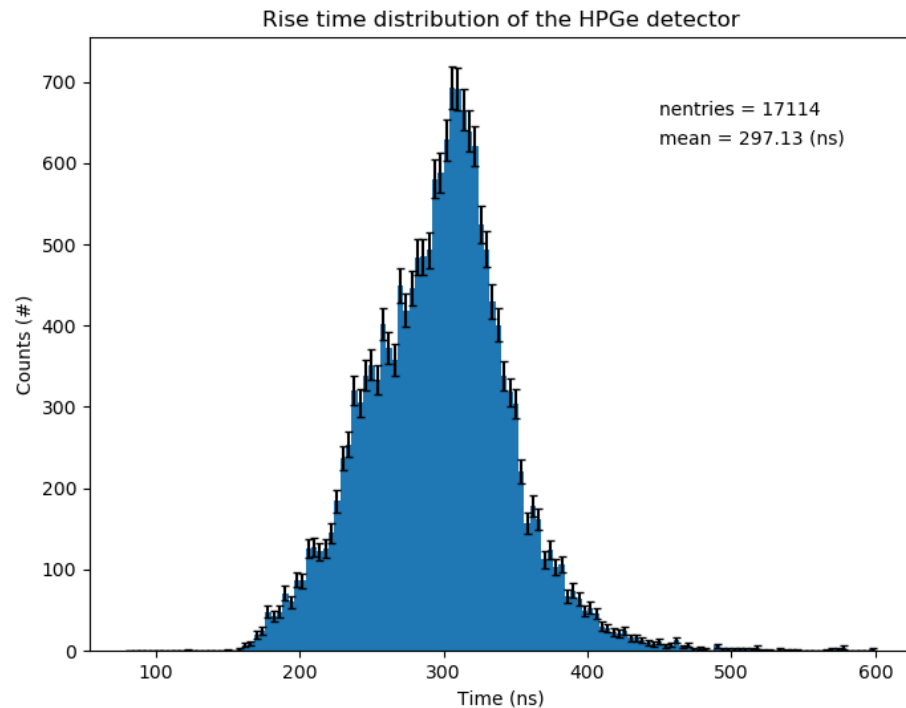
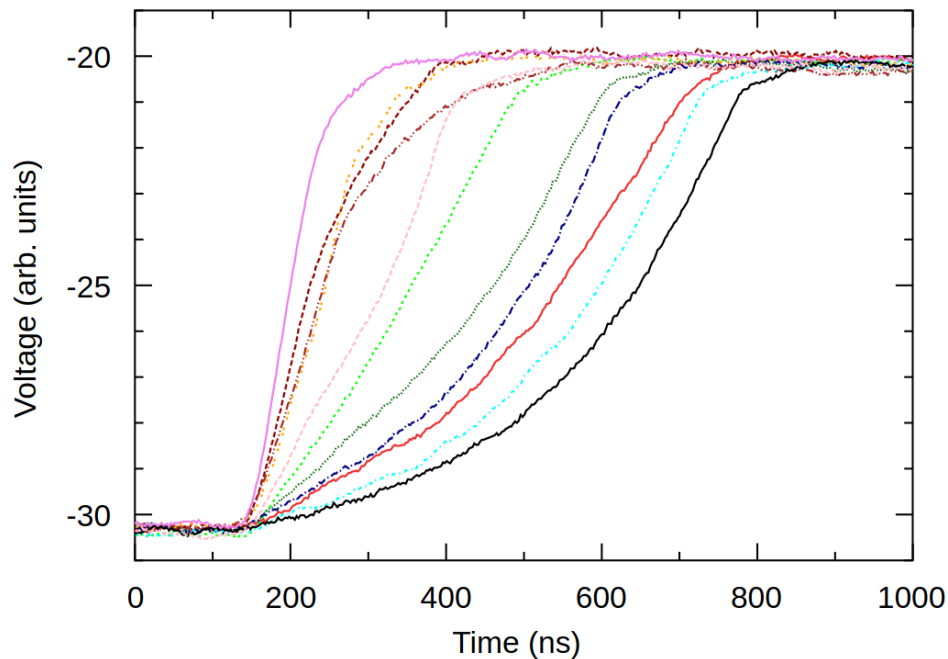
CFD Signal



CFD filter:
$$\Lambda_i = x[i - d_{CF}] - f_{CF} \cdot x[i]$$

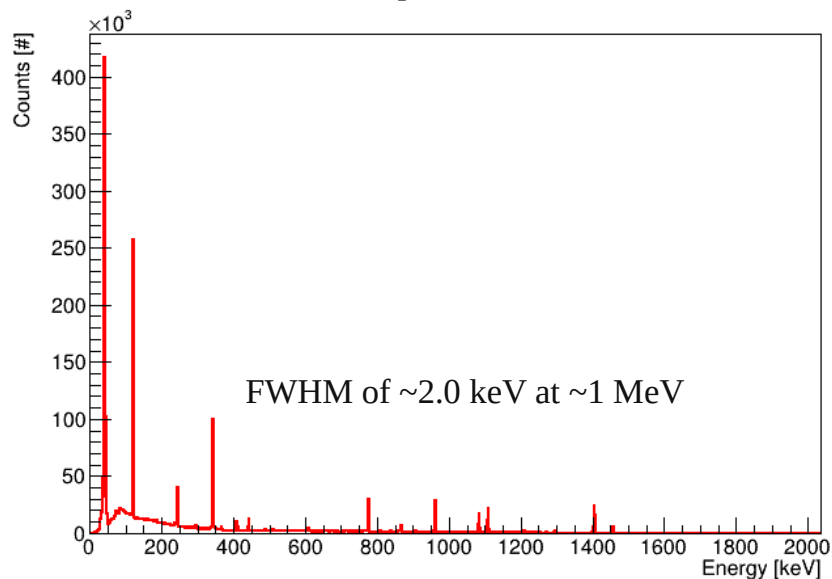


Rise time of HPGe detector

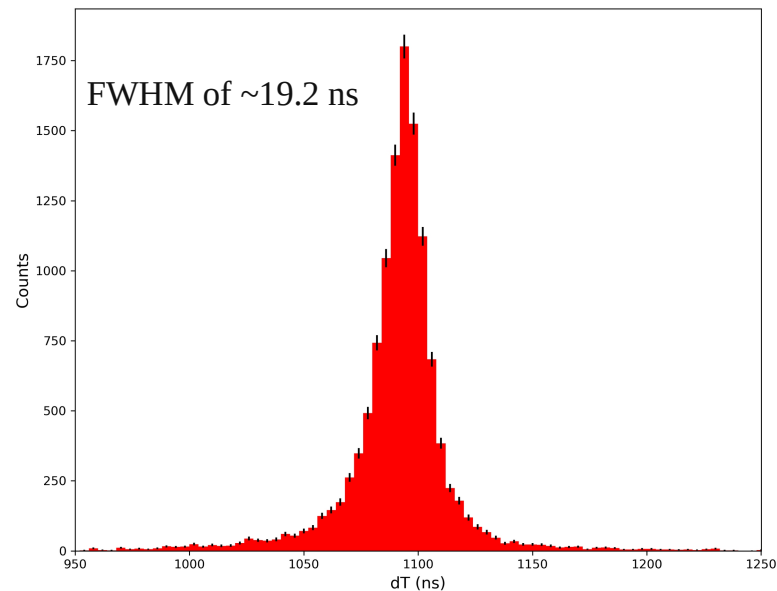


Energy and Timing Resolution of HPGe Detector

^{152}Eu Spectrum



Time difference between T0 and gamma-flash



Neutron Inelastic Cross-Section Measurement Formula

$$\frac{d\sigma_j}{d\theta}(\theta_i, E_n) = \frac{1}{4\pi} \frac{Y_j(E_n)}{Y_{FC}(E_n)} \frac{\varepsilon_{FC}\sigma_U(E_n)}{\varepsilon_j} \frac{\rho_U}{\rho_s} \frac{A_s}{A_U} \frac{1}{c_{ms}(E_n)}$$

Y_j : γ yield in the detector j ,

Y_{FC} : Fission chamber yield,

ε_{FC} Fission Chamber Absolute Efficiency

ε_j : Absolute Photopeak Efficiency of the Detector j

σ_U : $^{235}\text{U}(n, \text{fission})$ Cross Section

ρ_U : Areal Density of the Uranium Deposit

ρ_s : Areal Density of the Sample

A_U and A_s : Atomic Masses of Uranium and Sample

c_{ms} : Neutron Multiple Scattering Correction Factor

$$\sigma(E_k) = 2\pi \left[w_{110^\circ} \frac{d\sigma}{d\theta}(110^\circ, E_k) + w_{150^\circ} \frac{d\sigma}{d\theta}(150^\circ, E_k) \right]$$

E_k : Incident Neutron Energy

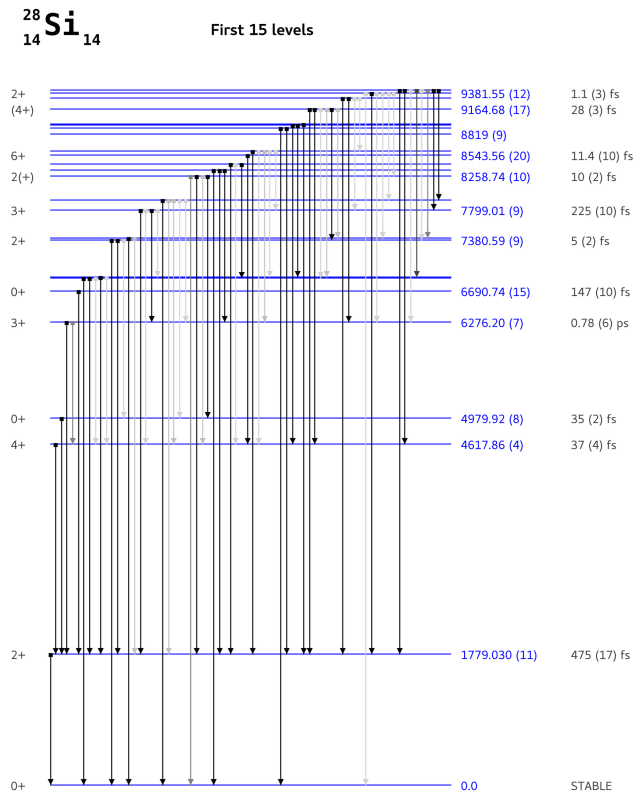
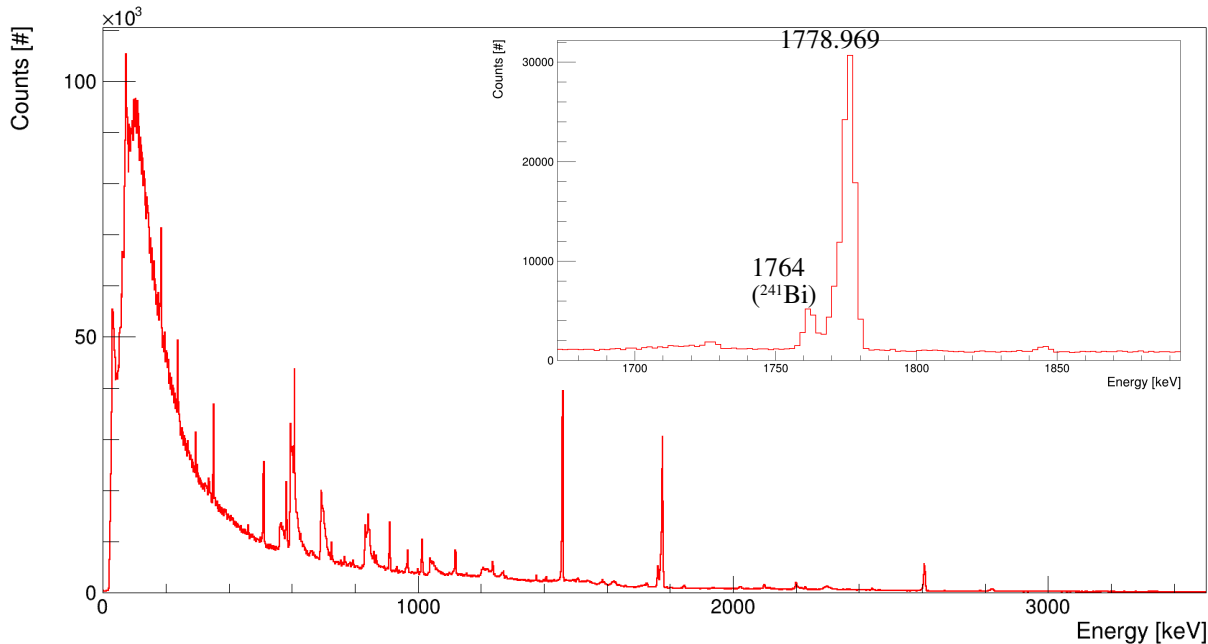
$\frac{d\sigma}{d\theta}(110^\circ, E_k)$ and $\frac{d\sigma}{d\theta}(150^\circ, E_k)$: Differential Cross Section at 110° and 150°

w_{110° : 1.30429

w_{150° : 0.69571

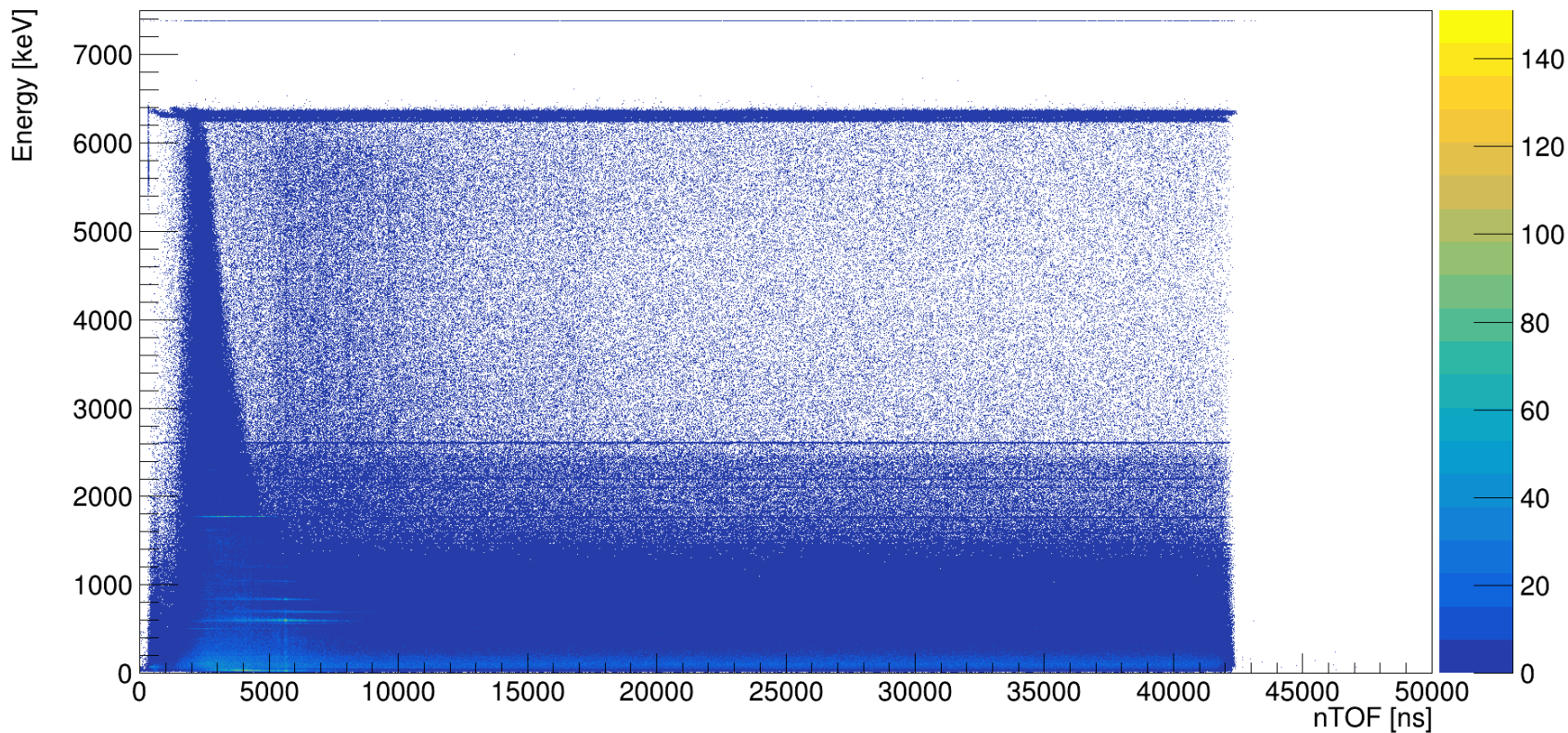


Preliminary Results for ^{28}Si for 1 Detector



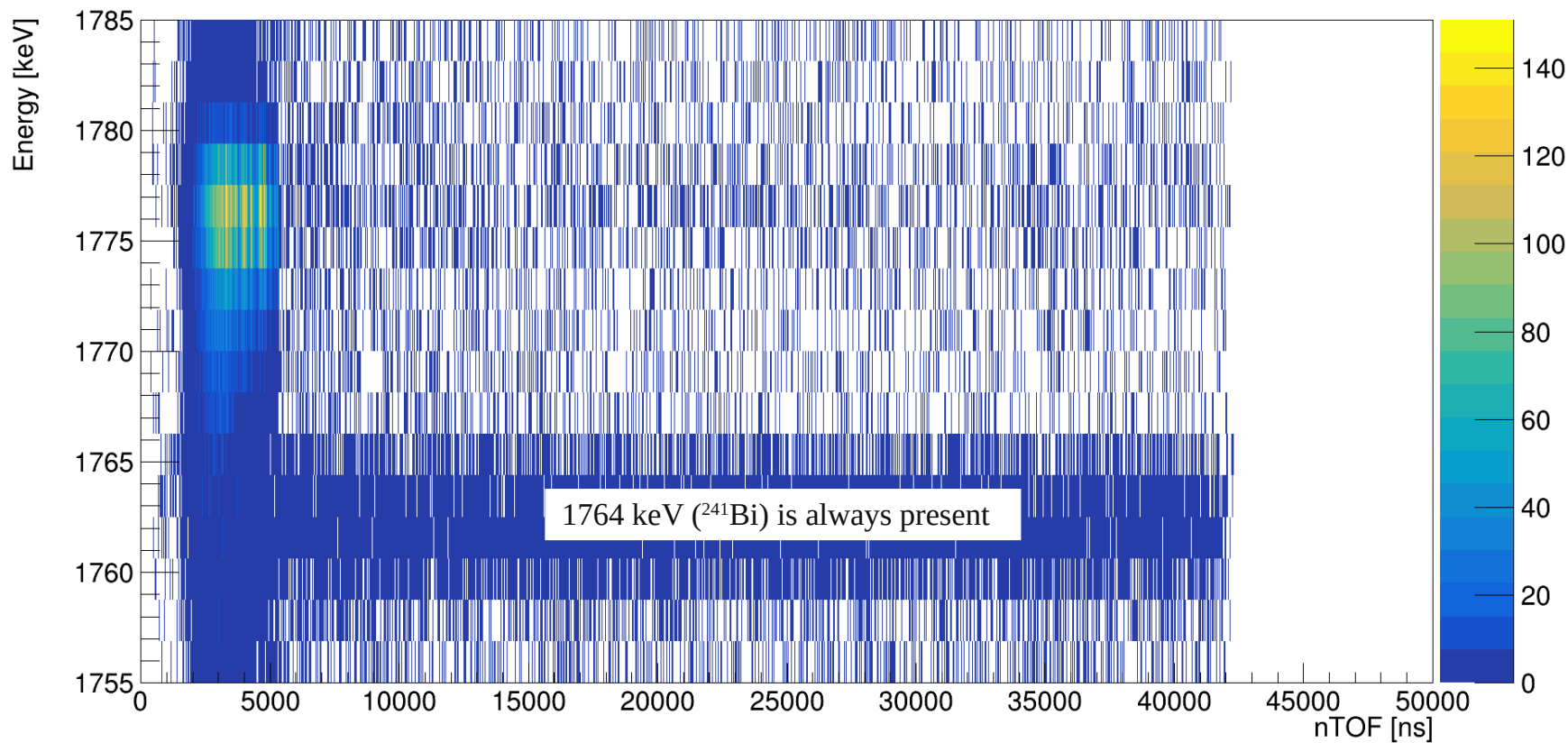


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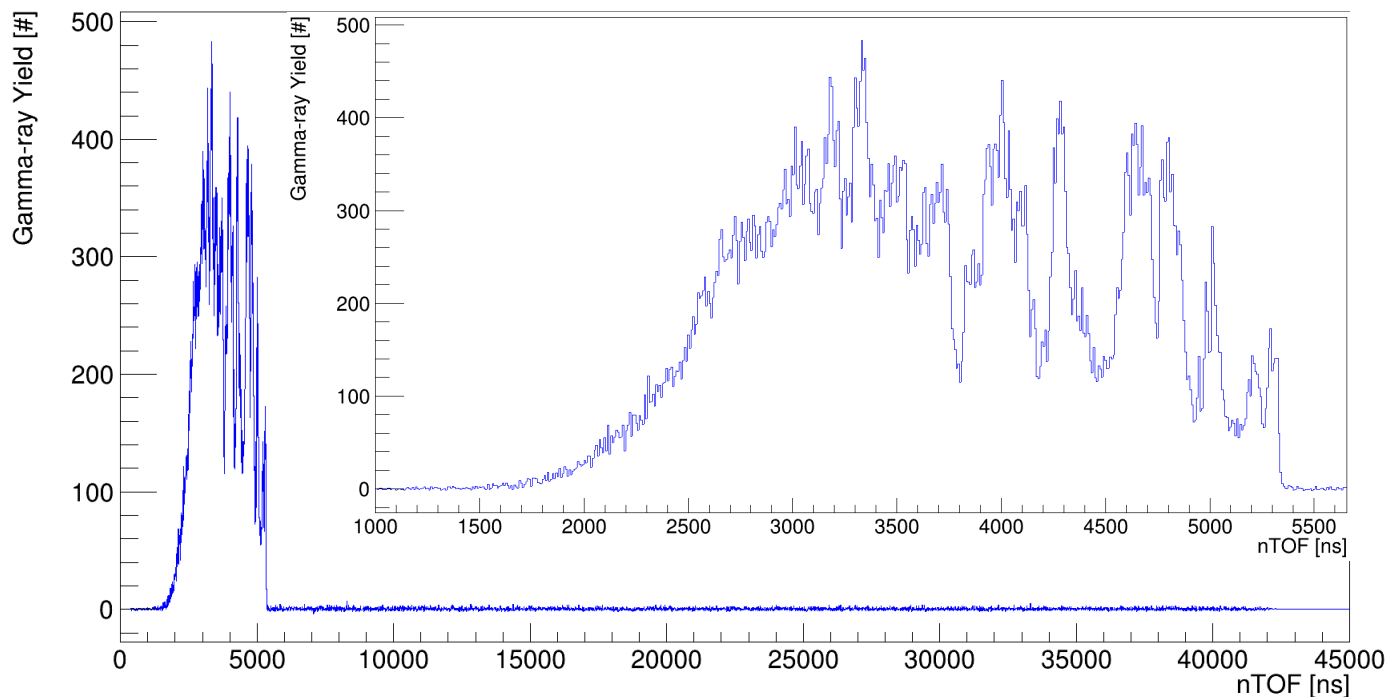




Preliminary Results for ^{28}Si for 1 Detector



Preliminary Results for ^{28}Si for 1 Detector





Thank you for your attention.

Any questions?