### **Introduction to ITER**

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**ITER Organization** 

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### This presentation follows heat transfer processes in ITER.



Proton Neutron







### Plasmas span a large range of Temperatures and densities



Cold plasma : neutrals >> ions ≈ electrons

- Aurora Borealis: 10<sup>7</sup> e<sup>-</sup>/m<sup>3</sup>, < 0.1 eV
- Gas discharge lamp:  $10^{17} \text{ e}^{-1}/\text{m}^{-3} < 0.5 \text{ eV}$





#### Hot plasma : neutrals << ions ≈ electrons

- Sun (core): 10<sup>32</sup> e<sup>-</sup>/m<sup>3</sup>, 1.2 keV
- ITER (core): 10<sup>20</sup> e<sup>-</sup>/m<sup>3</sup>, 25 keV





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### ITER plasmas are twenty times hotter than the Sun's core



Sun (core): 10<sup>32</sup> e-/m3, 1.2 keV

- Proton (<sup>1</sup>H) fusion reaction chain
- Gravitational confinement: 28 g



### ITER (core): 10<sup>20</sup> e-/m3, 25 keV

- Deuterium (<sup>2</sup>H) -Tritium (<sup>3</sup>H) fusion
- Magnetic confinement: 5.3 T





# Fusion performance is characterized by the triple product of <u>density</u>, <u>temperature</u> and <u>confinement time</u>.

To achieve high Q >5 requires hot >10 keV plasmas with sufficient density that keep energy for sufficiently long time

### **Fusion technology demonstration:**

Superconducting magnets Plasma facing components H&CD

Tritium breeding



## Our mission is to demonstrate the scientific and technical feasibility of fusion power for peaceful purposes.



### ITER will see the largest steady temperature gradient in the universe.



Plasma core: 150,000,000 °C

Blanket / Divertor: 550 °C

Vacuum Vessel : 100 °C

Thermal Shield : -193 °C

Superconducting Magnet : -269 °C

### Toroidal magnetic topology used to confine charged particles.

At high temperatures Deuterium and Tritium atoms ionize to form a charged plasma which may be contained by magnetic fields.



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## External heat sources are required to attain target plasma temperatures of 150,000,000 °C.





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### Neutral beams and EC and IC provide additional heat sources.



### Controlling the flow of heat from the plasma is crucial.



### Components designed to handle high heat fluxes line the divertor.



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## Why change to a tungsten first wall?

- Physics basis for tokamak operation with W walls is much stronger than it was at start of ITER construction
- Several issues with Be as PFC:
  - Erosion lifetime
  - Tritium retention in co-deposits
  - Low melting point → lower margin in I<sub>p</sub> before potential "gap bridging" on FW panels (disruption current quench)
- Major benefit in assembly complexity and avoid costly later wall changeout



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### Neutronic shielding protects superconducting coils and diagnostics.



### Gaps in blanket shield blocks generate inhomogeneous heat loads.

Neutron heat source maps across two poloidal slices of an ELM coil (right)



Vessel and ELM coil temperature map



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### Thermal loads drive the design of many in-vessel components.

ELM coil mounted to inner vessel shell

Ccompliant fixings developed to handle heat loads





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### Active water cooling removes neutronic heat loads from the vessel.

Neutronic heat source



Active water cooling



Vessel temperature





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Differences in neutronic heat generation between inner and outer vessel shells generate large thermally generated vessel loads.







magnification x200

## Thermal shields protect magnets from vessel, pipe, and cryostat radiation

Thermal shields are cooled to 80K and manufactured with a low emissivity surface.





### Superconducting magnets will be necessary for any future plant.



# The ITER superconducting magnet system comprises 48 separate coils.



### Superconducting strand is actively cooled using supercritical He.

Fluctuating fields and Ohmic losses at conductor joints generate unwanted heat which must be removed to maintain low strand temperatures.







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## Grooved 'radial plates' locate 5km of conductor for each toroidal field coil.

# Service H



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### Pancakes are stacked and insulated to form a coil winding-pack.





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## The alignment of ITER's 17 meter high, 360 tonne D-shaped Toroidal Field magnets is a feat of precision engineering.

Exceptionally low tolerances that are repeatable and stable over time.



## Field line deviations with resect to the first wall generate high start-up heat loads.

Field lines and first wall should be approximately concentric and circular with a long wave peak-peak misalignment H < 6mm.







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### Error fields slow plasma rotation and increase disruptivity.

Overlap error field limits are defined via a semi-empirical scaling for n=1 & n=2 modes (GPEC).



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### Component alignment is governed by targets and tolerances.



In-pit sector alignment











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### Coil energization aligns the toroidal field coil vault.

High-field and low-field sides of the TF coils are structurally decoupled.





magnification x50

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### A cryostat maintains a good vacuum around all SC magnets.

A vacuum suppresses convective heat transfer.



### The Tokamak pit ready to receive its first large components



## All ITER components pass through the assembly hall before being hoisted into the Tokamak pit





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### The cryostat base on its way to the pit



### The cryostat is assembled in four sections





### An outer thermal shield lines the cryostat.

### Special tooling supports in-pit assembly and alignment



### Captive coils are lowered into the pit before the first sector.



### Sector components are upended...



### And transferred to the sub-sector assembly tool.



### The vessel is wrapped in an actively cooled thermal shield.



### A pair of TF coils are swung over each side of the vessel sector.



### Successful demonstration of in-pit sector alignment.





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## In summary the control of heat transfer is critical to the successful fulfilment of the ITER's scientific and technical goals





