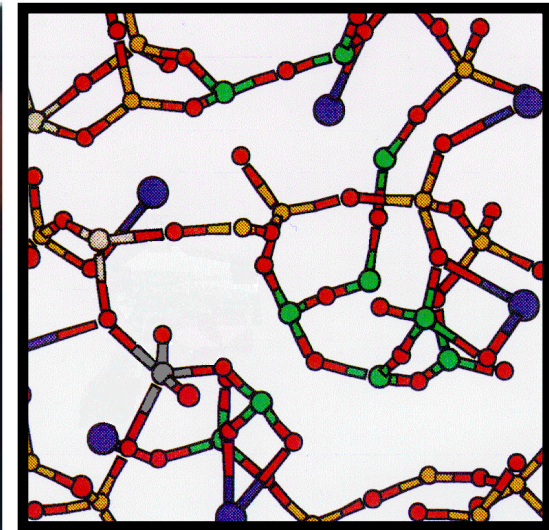


How to synthesize a good glass ?

From Fission Products Nuclear Glass
to New Glasses

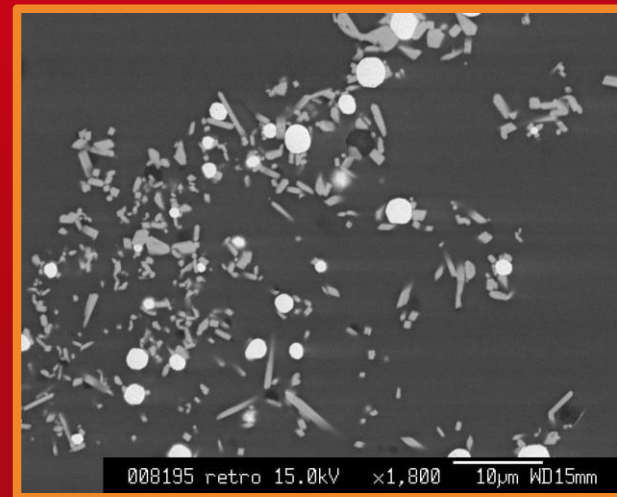
DE LA RECHERCHE À L'INDUSTRIE
cea den



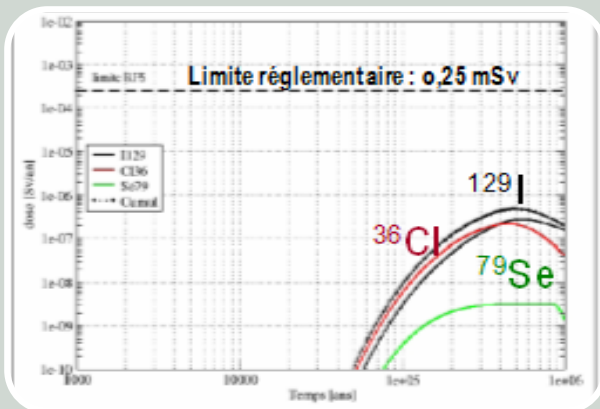
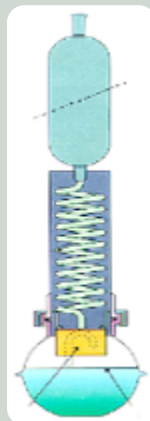
Florence Bart
Nuclear Energy Division – Marcoule Center

JOINT ICTP – AEA WORKSHOP 10-14 SEPTEMBER 2018 –
TRIESTE

What is a « Good » Glass ?



The answer is evolving with time



At the early beginning
(50')

« A glass that can be poured is a good glass »

Intrinsic durability of the glassy material

Radionuclide release in function of time, in specific storage conditions

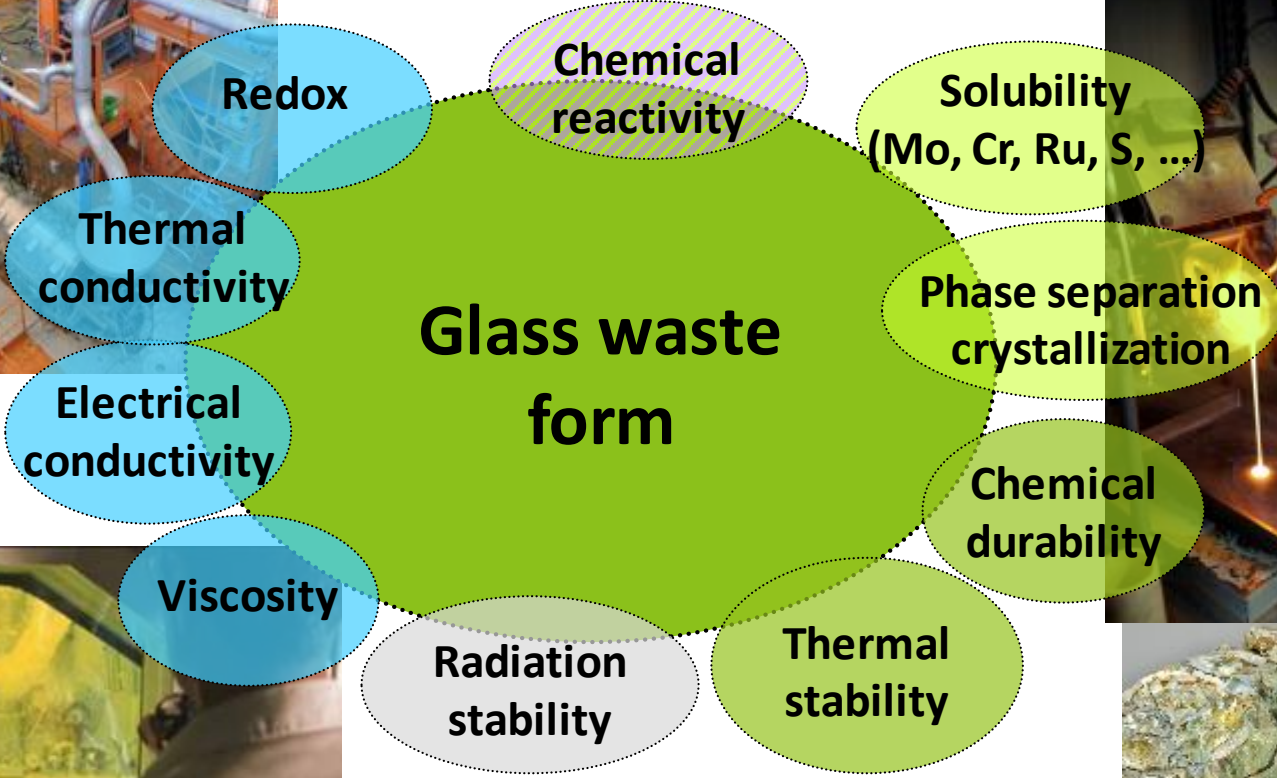
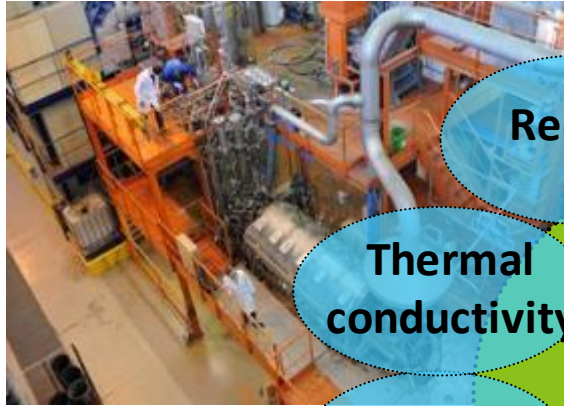
Origin

A good glass is defined thanks to an iterative process between material and process development

TECHNOLOGICAL FEASIBILITY

Glass melt properties

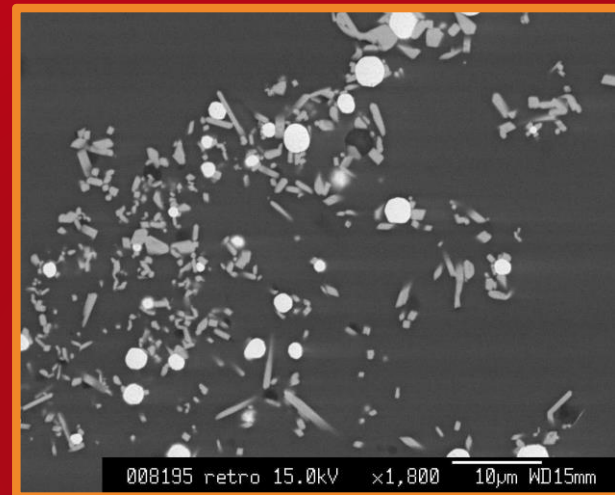
LOADING RATE



Glass Long term behavior

Glass properties

Glass Formulation



Major critical chemical elements coming from nuclear waste to be vitrified

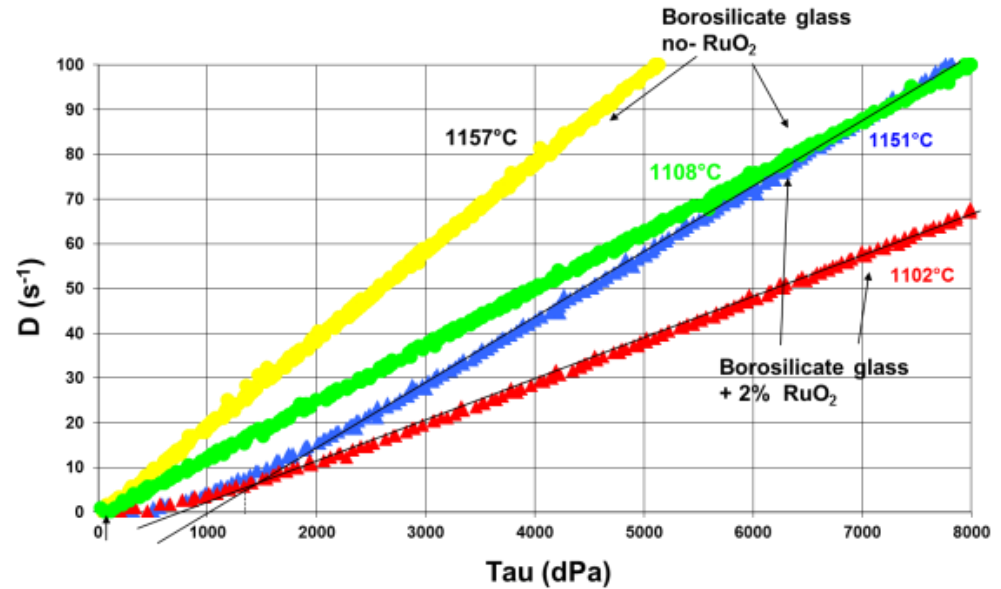
Mo	→	Phase separation and molybdates crystallization
Ru, Pd, Rh, Ag	→	Chemical reactivity, particle settling, electrical conductivity, viscosity
Nd, La, Pr, Ce, P	→	Apatite crystallization
Fe, Ni, Cr	→	Spinel crystallization
Ru, Cs, Tc	→	Volatility

Important properties

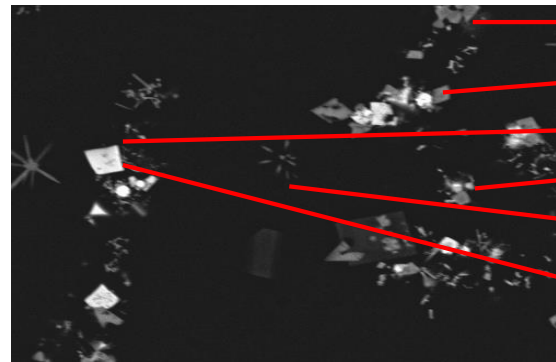
Viscosity



Melting process can be impacted by noble metal content in glass melt
(Convection, Pouring rate, Capacity)

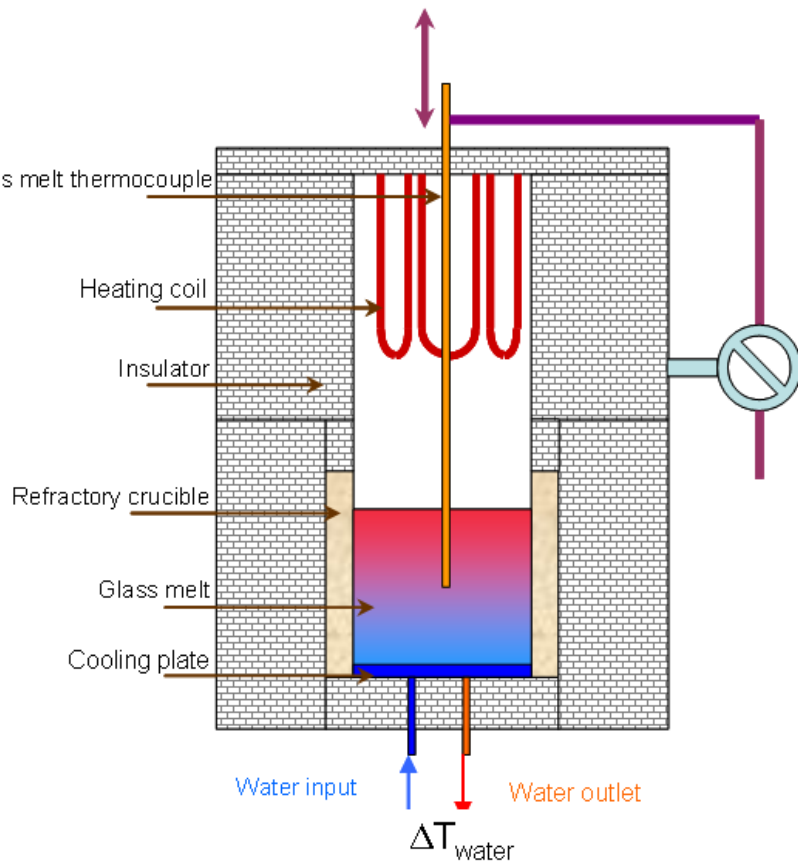


Micro-homogeneity

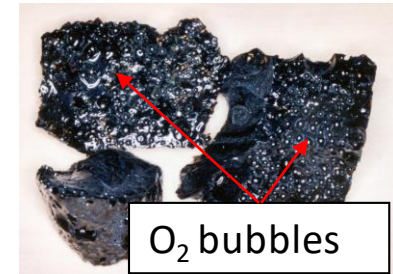
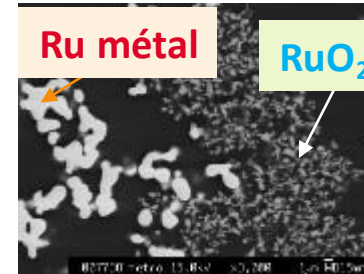


- Chromites
- Palladium -Tellure
- Cerium oxide
- RuO₂
- Silicophosphate
- Ca-molybdate

Thermal conductivity



Redox properties



Process parameters

- Melter atmosphere
- Temperature

Input data

- Glass frit composition
- (Fe²⁺/Fe³⁺) in glass frit
- Waste composition
- Nitrate concentration

Thermodynamic data on redox equilibria in the glass

- Fe²⁺/Fe³⁺
- Ce³⁺/Ce⁴⁺
- Cr³⁺/Cr⁶⁺
- Mn²⁺/Mn³⁺
- Ni²⁺/Ni³⁺
- Ru⁰/Ru⁴⁺

Oxygen fugacity in the final glass

Final redox ratio $M^{m+}/M^{(m+n)+}$ of multivalent elements in the glass

Long term behaviour



Initial Glassy state

Thermal stability
(0-300 years :
90-70 °C)

Self irradiation
(cumulative
dose : 1 dpa)

**Chemical
durability**
(performance
assessment : 1
Ma)

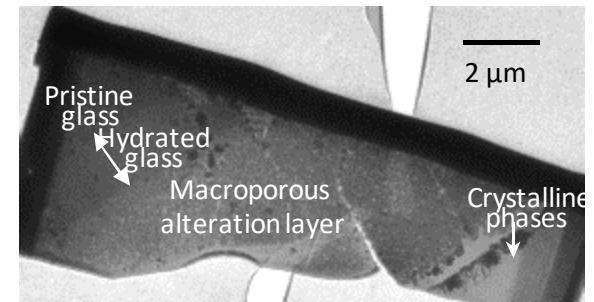
**Phase separation
Crystallization**

**Radiation
damage**
(electronic and
nuclear
interactions,
He production)

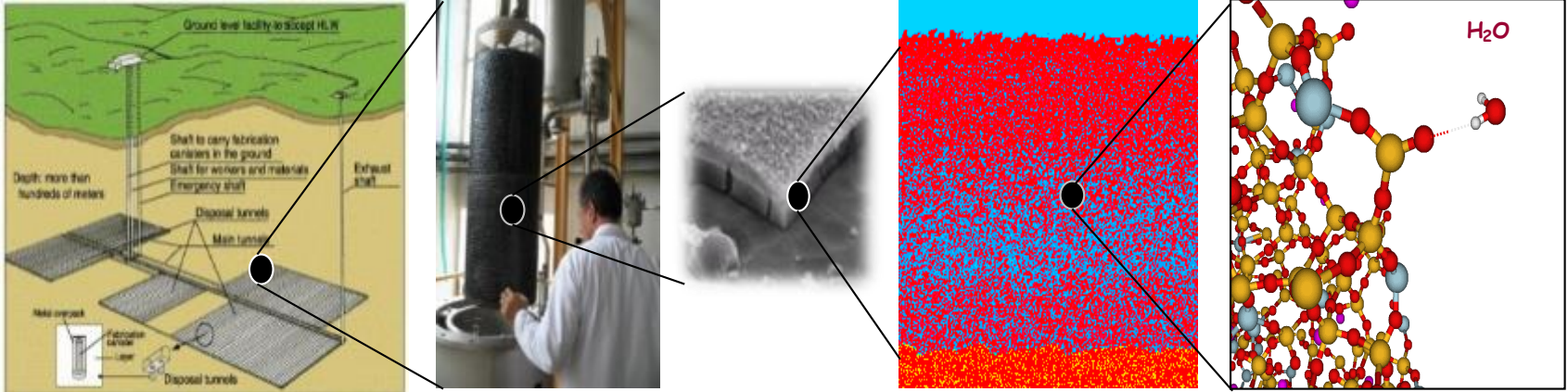
**Aqueous
alteration**
(source term:
radionuclide
release from the
package)

Glassy state structural modifications

Groundwater composition,
Fluid circulation
Interactions with surrounding
materials : clay, iron, cement
Transformation of surrounding
materials : metallic corrosion
products
Self-Radiation



To describe macroscopic properties...



... from atomic to mesoscopic scale

Archeological and natural analogs for validation

Basaltic glass: 1.4 Ma



Obsidian and basaltic glasses (volcanic eruption)



Fractured archeological glass (Embiez), 1800 y. in Sea water

Steel production: Blast furnace slags from iron ore reduction (400 y. in an iron (anoxic burial medium) and clayey environment)

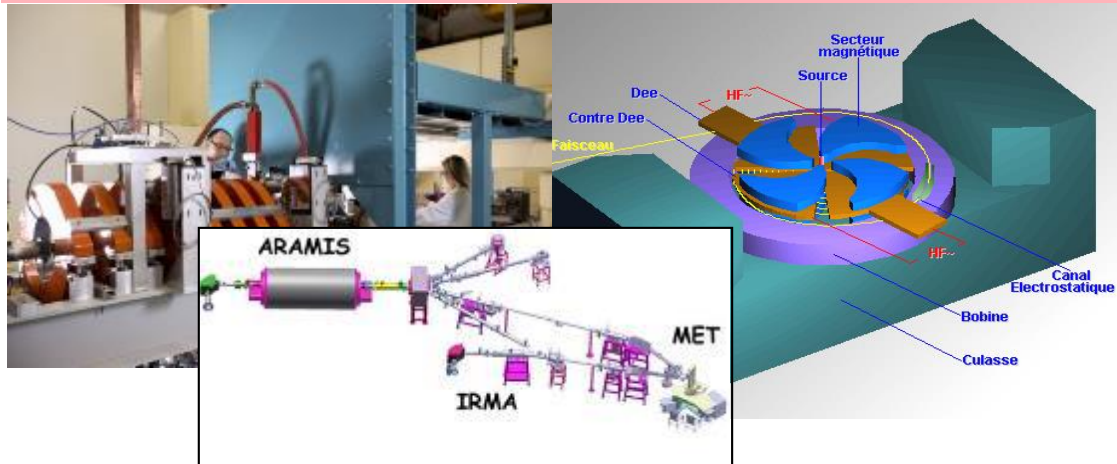


IRRADIATION DAMAGE : EFFECT ON GLASS STRUCTURE AND LONG TIME BEHAVIOUR

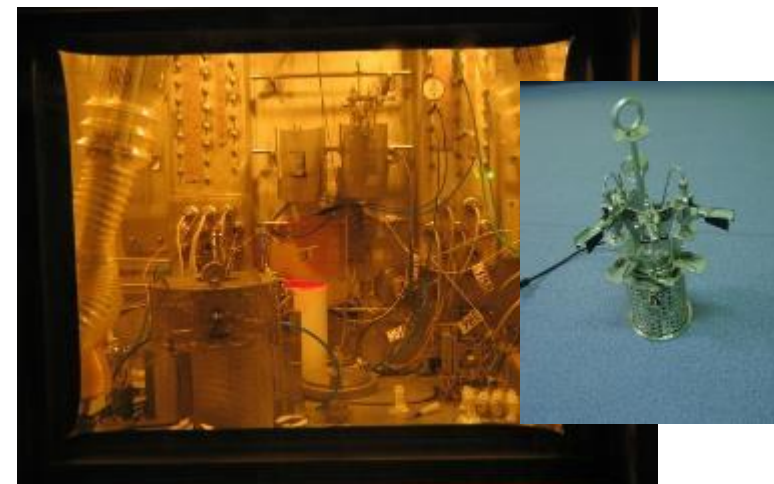
Doped glasses
(^{244}Cm , ^{238}Pu , ^{239}Pu ,....)



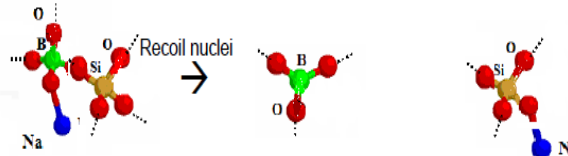
Irradiation facilities



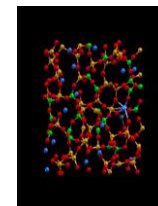
Leaching tests and measurements
(effect of dose and dose rate)



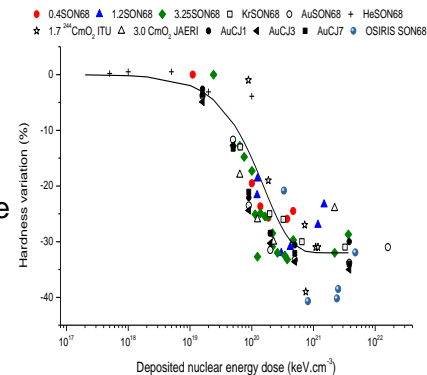
Damage / properties modelling



MD simulation of displacement cascade: accumulation of ballistic disordering



- ❑ **Thermal phase** → **local melting** → **network reorganization**
(rapid thermal quenching)
- ❑ **Stabilization of a new structural state**
when all the volume has been damaged one time ($\sim 4 \times 10^{18} \alpha/g$)
- ❑ **Stabilization of macroscopic properties**
(density, hardness...)



VITRIFICATION PROCESSES

Glass



Glass - Ceramic

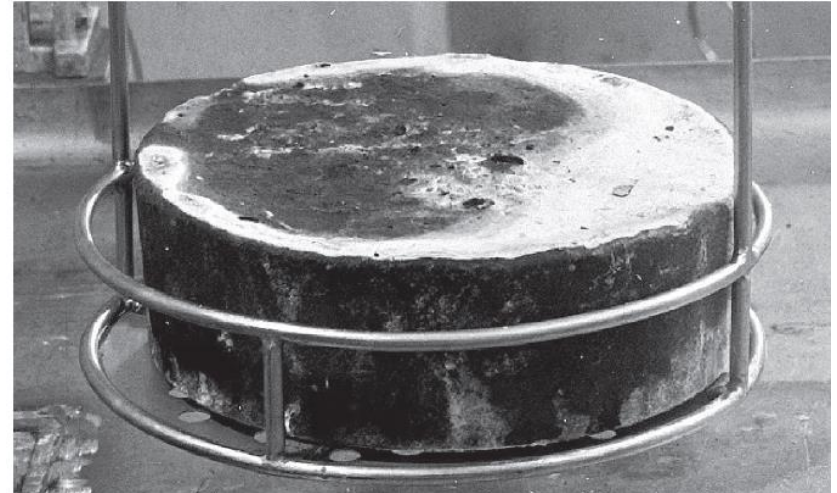


**Glass /Metal
Matrice**



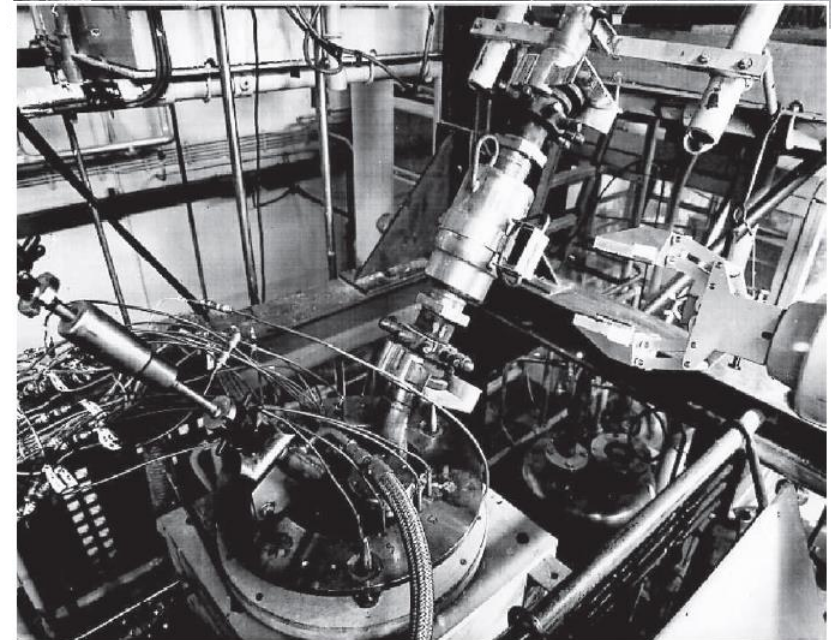
Gulliver (1964 – 1967)

First French Vitrification pilot : heating of a gel, produced by FP impregnation of a clay material, in a refractory pot
→ 170 kg of nuclear glass (10 kg per block)

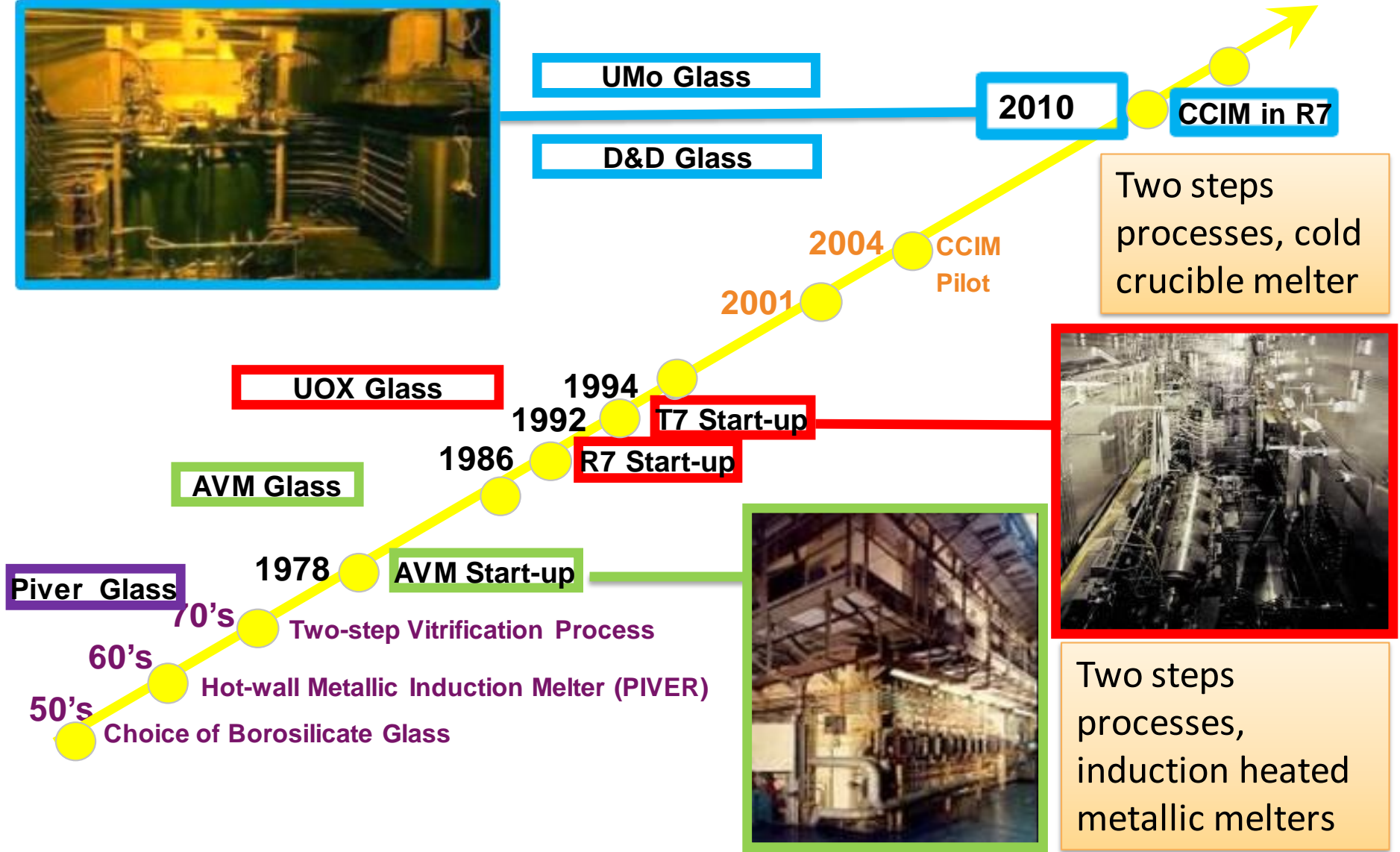


Piver (1969 – 1980)

Semi-industrial process : glass is melted by batch, in a metallic melter, heated by induction, and then poured
→ 13 tons of nuclear glass (25 m³ of HLW FP solution)



cea den Development of the nuclear glass industry



Calcination – Vitrification continuous two-steps process



Glass frit

Surrogates

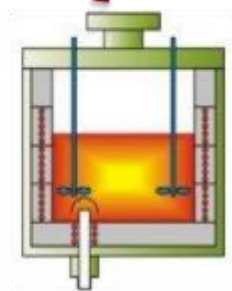
Additives

Calciner

Off-gas treatment



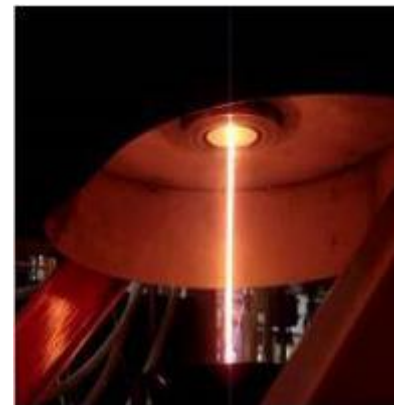
Cold Crucible
Inductive Melter



Hot Metallic
Melter



Glass canister



History of glass specifications implemented at industrial scale



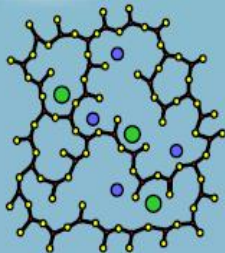
Homogeneous Borosilicate Glasses

Glass-ceramic

Glass



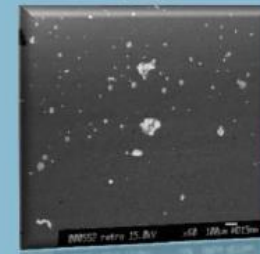
- Silicon
- Oxygen
- Natrium
- Nuclear fission product



Legacy waste : Molybdenum-rich fission product solutions (UNGG fuels)

- Highly corrosive ILW glass, low solubility of Mo into BSG
- Designing a glass-ceramic melted material
 - Homogeneous melt (1250°C)
 - Crystallization with cooling
 - Loading factor up to 13 wt%

Glass-ceramic



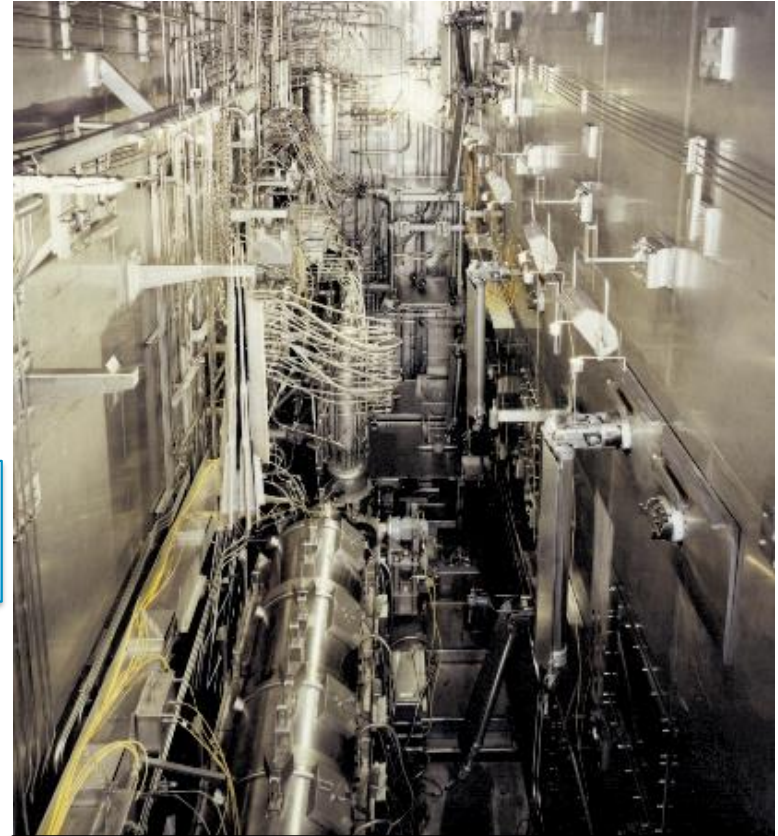
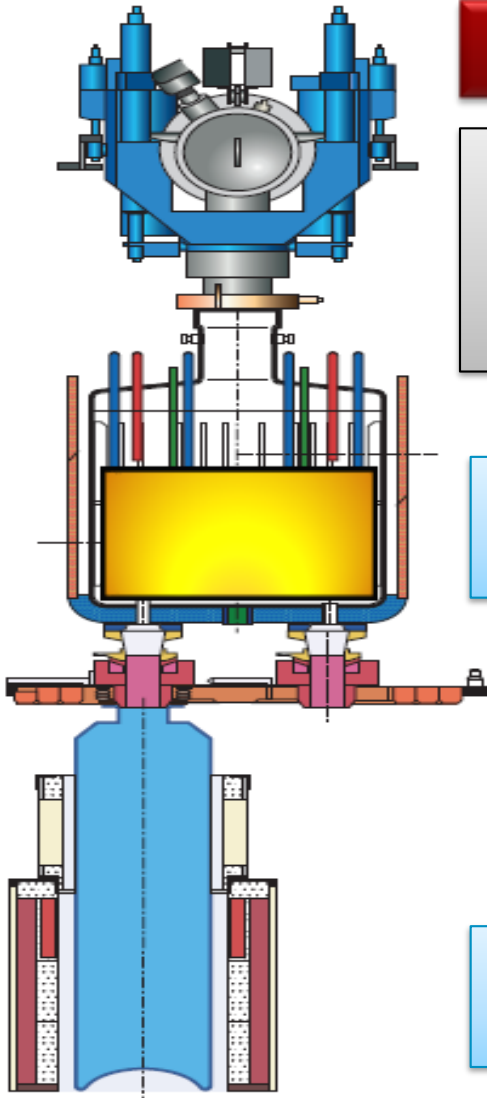
From induction heated metallic melter...

Since 1990

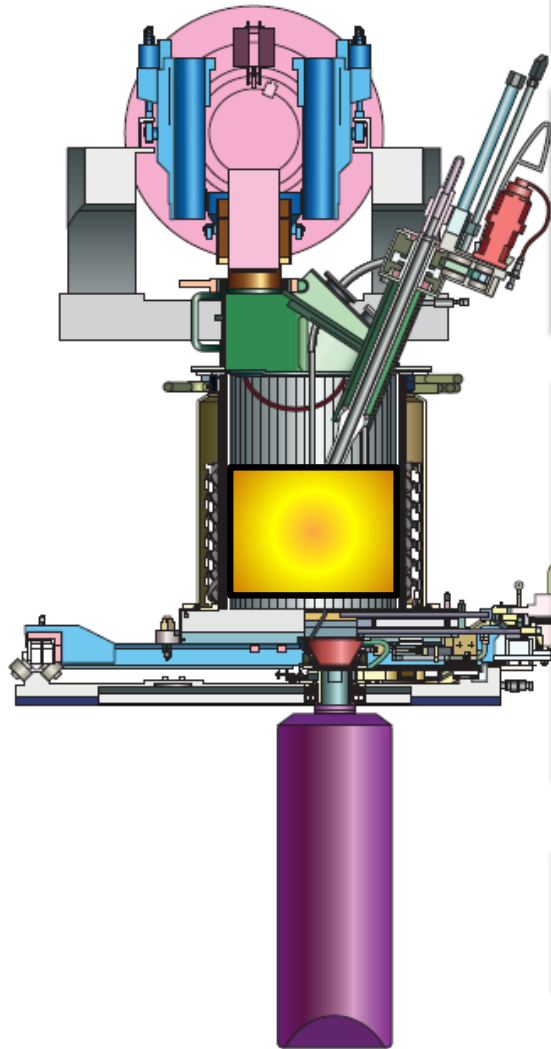
Thermal flux from metallic walls to molten glass

Hot Metallic Crucible
Bubblers, rotary stirrers

Pouring glass into stainless steel canister



5 vitrification lines in operation at AREVA La Hague Facility



Since 2010

Thermal flux from the molten glass to the cooled crucible

Cold Crucible Water cooled metallic structure (higher temperature, no corrosion on the melter)

Pouring into Glass canister



1 CCIM line in operation at ORANO La Hague Facility

**NEW WASTE, NEW VITRIFICATION
PROCESSES :**

IN-CAN TECHNOLOGIES

Marcoule : industrial nuclear site under dismantling

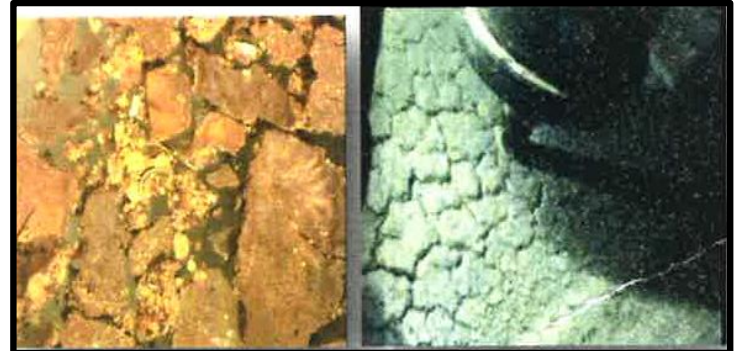


HLW coming from D&D operations

- Small quantities, sludges or solids
 - Compositions are not as precisely defined as for FPS
- Immobilization of TRU and FP into a durable matrix

ILW waste coming from MOX fuel production

- Alpha-bearing waste
 - Organic matter + metals : gloves, power cables, metallic material or tools, dusters...
- Volume reduction
- Organics destruction
- Immobilization of TRU into a durable matrix



High active deposits from fission products evaporators and tanks
Marcoule reprocessing facility



MELOX glove box (<http://www.irsn.fr>)

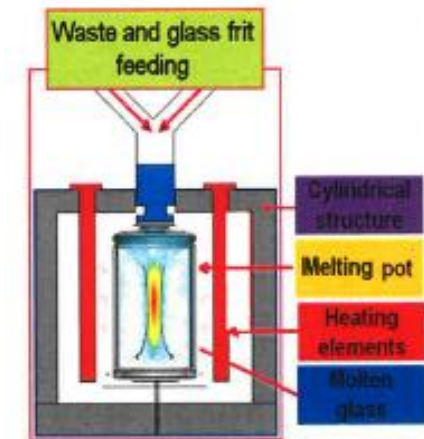
Currently developed by CEA* for its own waste coming from D&D operations, including legacy waste management

Material and process specifications :

- Flexible and adjustable to waste with a composition poorly defined : mixed effluents such as zeolites, co-precipitation sludges, powders of fuel debris (FP and alpha components)
- Final waste package must be suitable with existing routes and/or on-site storage facilities
- Compact size of the process, compliant with existing hot cells under dismantling
- “Dismantling tool” that shall be itself dismantled after use (for re-use)
- Low quantities of secondary waste
- Minimum investment and operation cost

Process development criteria :

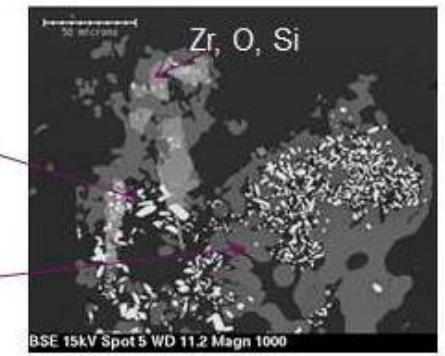
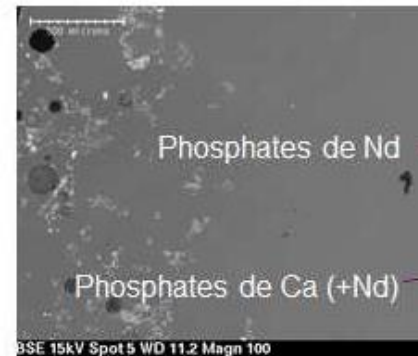
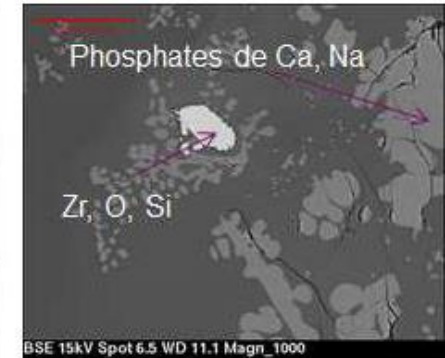
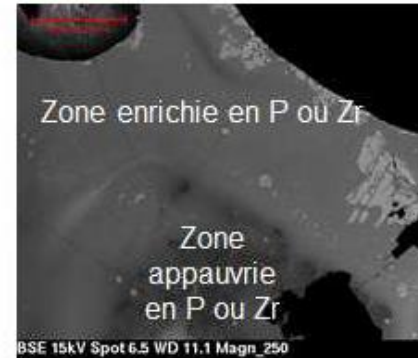
- ❑ One step IN CAN vitrification (no calciner)
- ❑ Container is used as a crucible renewed for each batch (no pouring)
- ❑ Resistance heating, thermal homogeneization (no stirrer)
- ❑ Design for liquid or solid feeding in a melting pot
- ❑ Operating temperature $< 1100^{\circ}\text{C}$



Principle of process

Formulation criteria :

- ❑ Minimization of FP volatilization (Cs)
- ❑ Adjustable to accommodate composition uncertainties and variabilities
- ❑ High content for P, Zr, Mo (a few wt%),
- ❑ Low viscosity melts to ensure homogeneization thanks to thermal convection



Microstructure of a simulated borosilicate glass enriched with P and Zr oxides showing numerous crystallizations

- ❑ To develop flexible glass formulations :
 - ❑ At relatively low elaboration temperature to avoid Cs volatilization
 - ❑ Suitable for P, Zr and Mo, elements that have a low solubility in borosilicate glasses
 - ❑ Compliant with variations of the feeding stream, characteristic of old deposits remaining in facilities that have been shut down, currently under dismantling
- ❑ To develop final package description :
 - ❑ Source terms are needed, since these packages are designed for deep disposal

LCV

Joint Vitrification Lab

cea AREVA

INCINERATION AND VITRIFICATION PROCESS : IN CAN MELTER

- ❑ **Intermediate Level Waste contaminated with alpha emitters:**
 - ❑ Mainly arising from glove boxes used for MOX production (Melox facility)
 - ❑ Mixed waste made of 30% organic matter/70% metallic content
- ❑ **Original conditioning option (compaction) not suitable for disposal because of organic matter radiolysis and hydrolysis that may result in**
 - ❑ Hydrogen release → overpressure, explosion issues
 - ❑ Corrosive species release → waste package corrosion issues
 - ❑ Complexing species release → potential increase in RN mobility in deep disposal
- ❑ **Alternative conditioning option is under study, with the following requirements :**
 - ❑ Full destruction of organic matter
 - ❑ RN conditioned in a mineral matrix

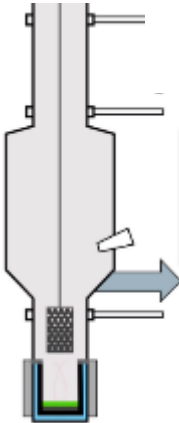
1



Introduction of the waste

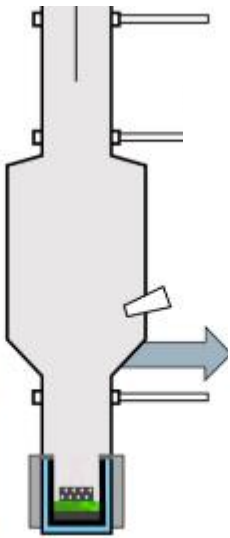
2

Organic matter is burning in the plasma

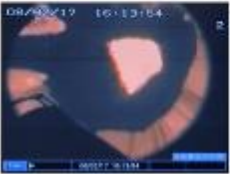


Gaz is released into gaz treatment

3



Metallic waste is heated thanks to induction
Glass fraction (green) is trapping actinides



- ❑ **Process developed as a combination of already pre-existing technologies**
 - ❑ Plasma torchs for incineration of solids
 - ❑ Cold crucible technology for metal waste melting
- ❑ **Innovations**
 - ❑ In Can melting of a biphasic melt
 - ❑ Metallic fraction at the bottom of the can
 - ❑ Glass fraction at the top of the can
 - ❑ New kind of waste package

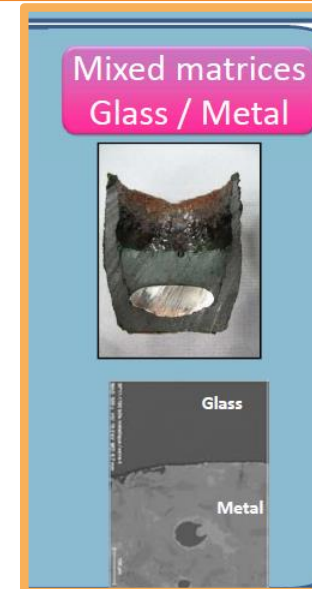
❑ Formulation of a new glass

- ❑ Suitable for actinide incorporation :
RN shall be confined in the glassy phase, not in the metallic part

→ Partition coefficients are under study, depending of compositions

❑ Description of a new ILW waste package

- ❑ Leaching behaviour of the vitreous phase
- ❑ Corrosion mechanisms of the metallic part of the package
- ❑ Combination of both parts in expected disposal conditions



TO CONCLUDE



Specification = to produce durable glass

→ Material : performance demonstration

→ Process : large quantities to be produced, half-continuous process, including pouring of the melt into containers

Chemical compatibility with the waste

Solubility (Cr, Ru, Rh, Pd, Ce, Pu, SO₄, Cl)

No phase separation (Mo, SO₄, Cl, P)

No devitrification (Mo, P, F, Mg, ...)

Maximize the waste loading

Process / Technology

Melting temperature
Viscosity, reactivity,
residence time,
Electrical cond.



Glass performance storage/disposal

Thermal stability
Chemical durability
Resistance to self-
irradiation

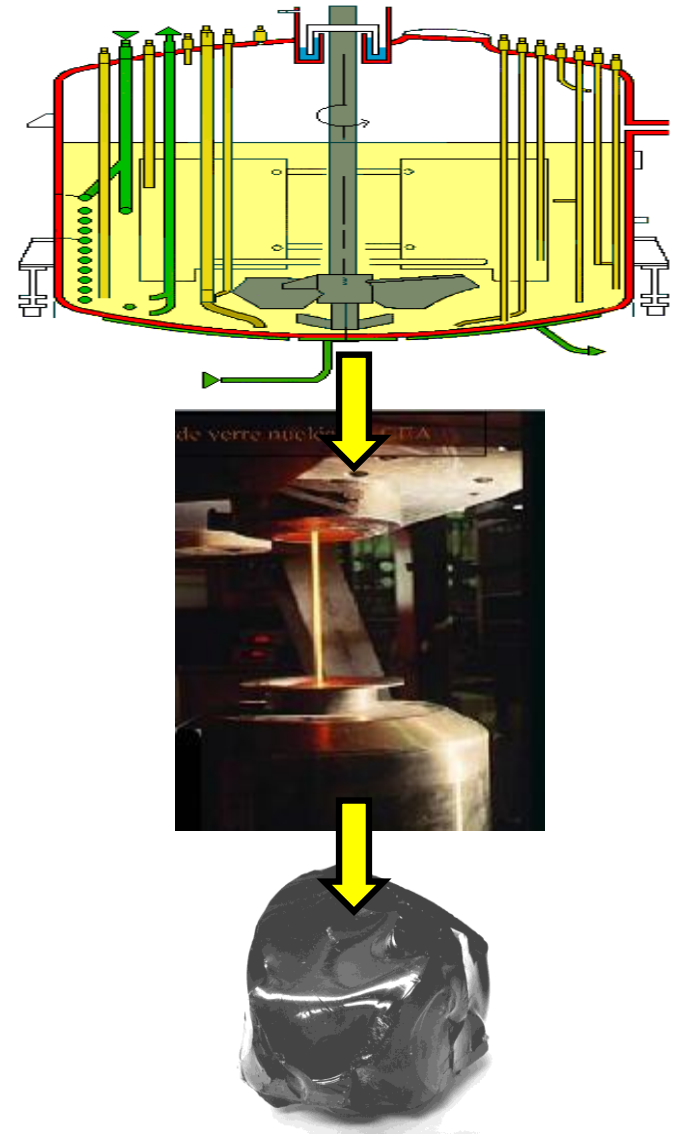
- ❑ **Vitrification of fission products solution is a mature industrial technology characterized by :**
 - ❑ Large capacities of production (20 to 50 kg/h of glass produced per melter, continuously)
 - ❑ Small variations of the incoming streams

- ❑ **New processes/glasses are needed for new High and Intermediate Level Waste**
 - ❑ New waste coming from dismantling operations of old facilities, larger range of chemical compositions
 - ❑ New specifications : smaller quantities of waste to be treated, geographically dispersed, need for lower cost vitrification processes
 - ❑ New glasses → New glass material science challenges to face

Thank you for your attention



- ❑ Fission Products solutions coming from spent fuel reprocessing (PF) were produced by PUREX process
- ❑ It was not possible to store them in the liquid state for a long time : acidic stream, needed to be cooled and agitated → Solidification required
- ❑ First ideas were to transform the FP solutions into a synthetic rock, such as naturally occurring silicates minerals
- ❑ At the end of the 50', vitrification has been developed



Fission Products				Metallic species				
Se	Rb	Sb	Sr	Ru	Mo	Sb	Rh	Tc
Te	Y	Cs	Zr		Pd	Sn		
Ba	Nb	La	Mo	Actinides				
Ce	Tc	Pr	Ru	U	Np	Am	Pu	Cm
Rh	Pd	Nd	Pm	Corrosion and addition species				
Sm	Eu	Gd	Ag					
Cd	In	Sn	Tb					
	Dy			Fe	Cr	Ni	P	Na

- Chemically complex (more than 30 chemicals)
- Precisely defined and nearly constant for given spent fuels (slowly evolving with increased burn-ups)