



The Abdus Salam
International Centre for Theoretical Physics



Workshop on

**ROLE OF PARTITIONING AND TRANSMUTATION IN THE
MITIGATION OF THE POTENTIAL ENVIRONMENTAL IMPACTS OF
NUCLEAR FUEL CYCLE**

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ICTP - Trieste, Italy

1774/3

Proposed P&T Concepts including *Gen-IV*

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Proposed P&T Concepts including Gen-IV

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- *Concepts 3 + 5 propose a homogeneous recycle of all the An together. The distribution of the An in all the fuel should be preferred for safety reasons*
- *For double strata a separate fuel cycle is needed.*

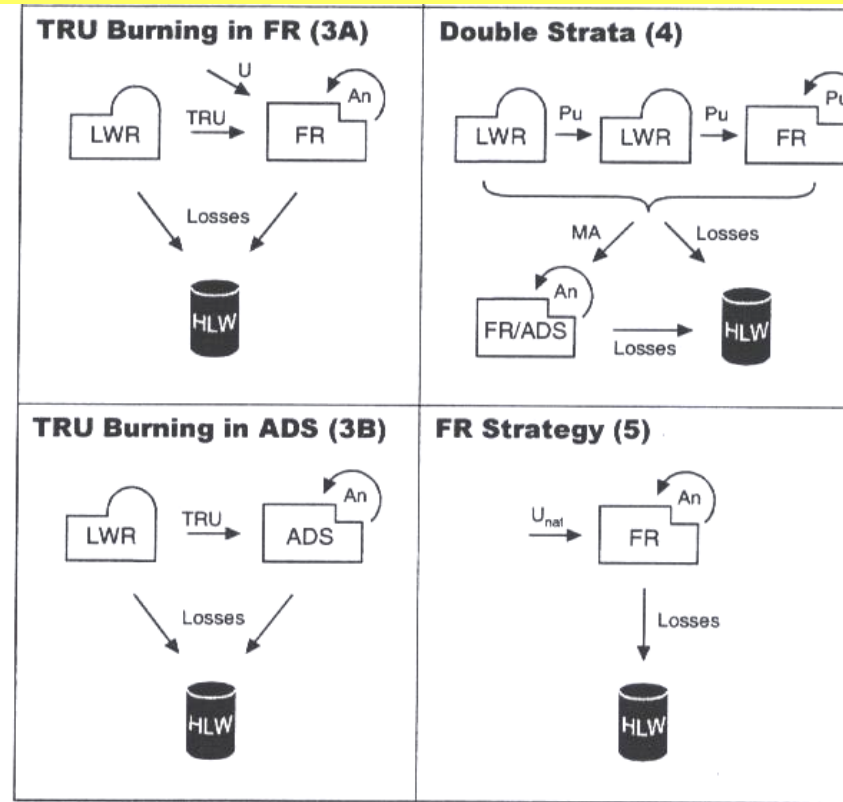
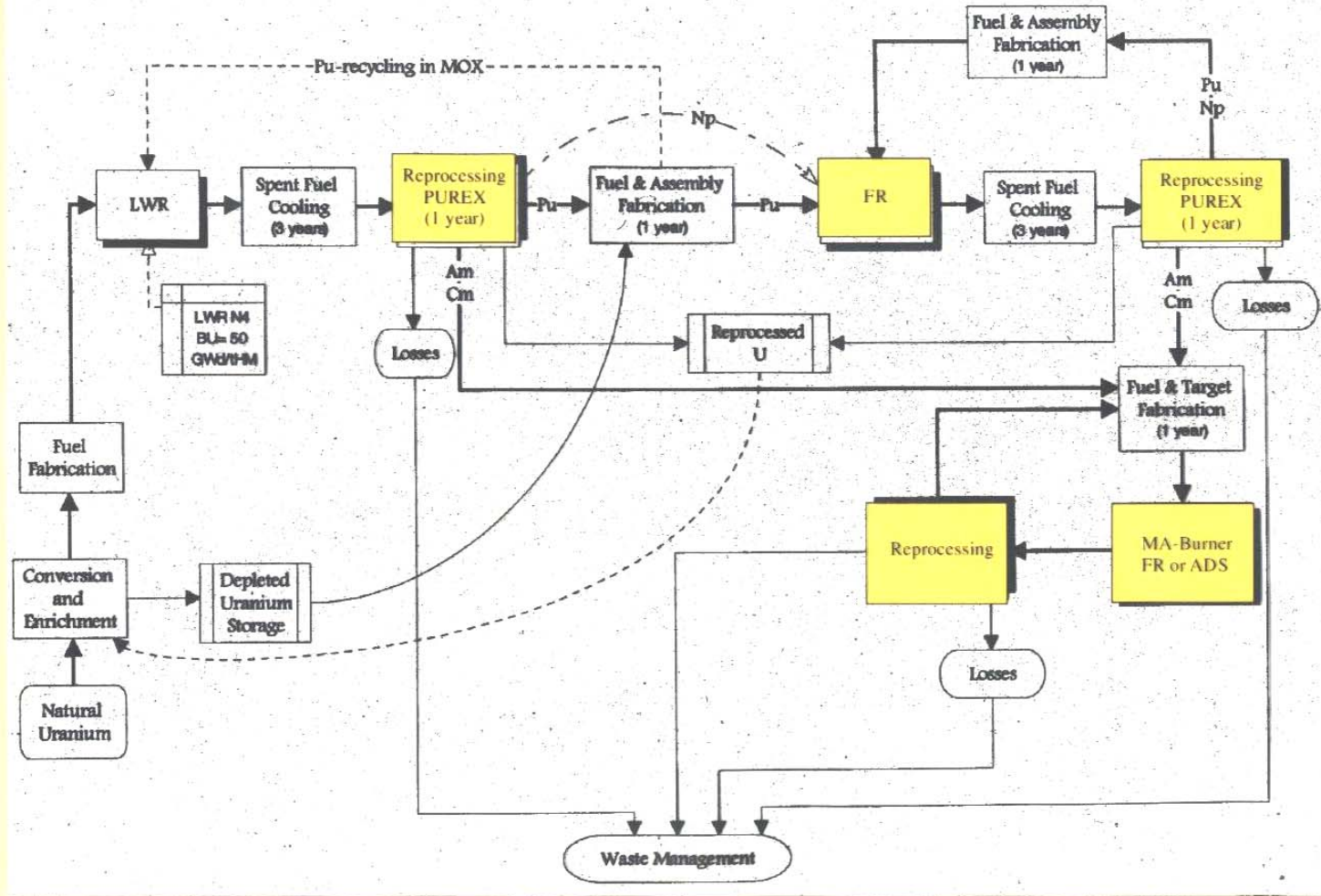
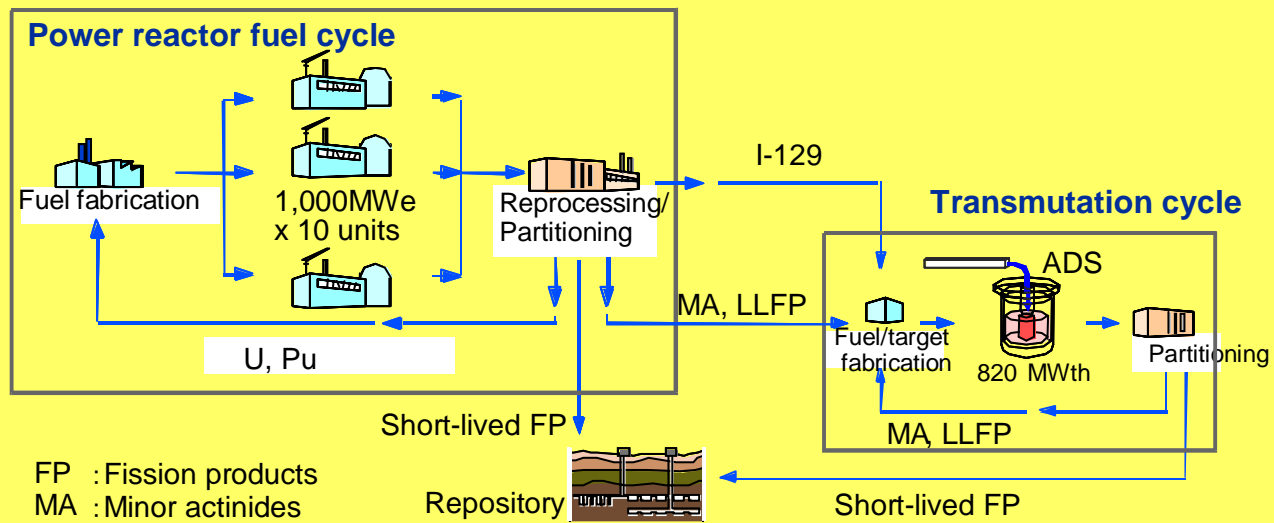


Figure 6. Flow-sheet of Double Strata Fuel Cycles





The other concepts base on fast reactors/ADS in symbiosis with thermal reactors or have substituted the thermal reactors

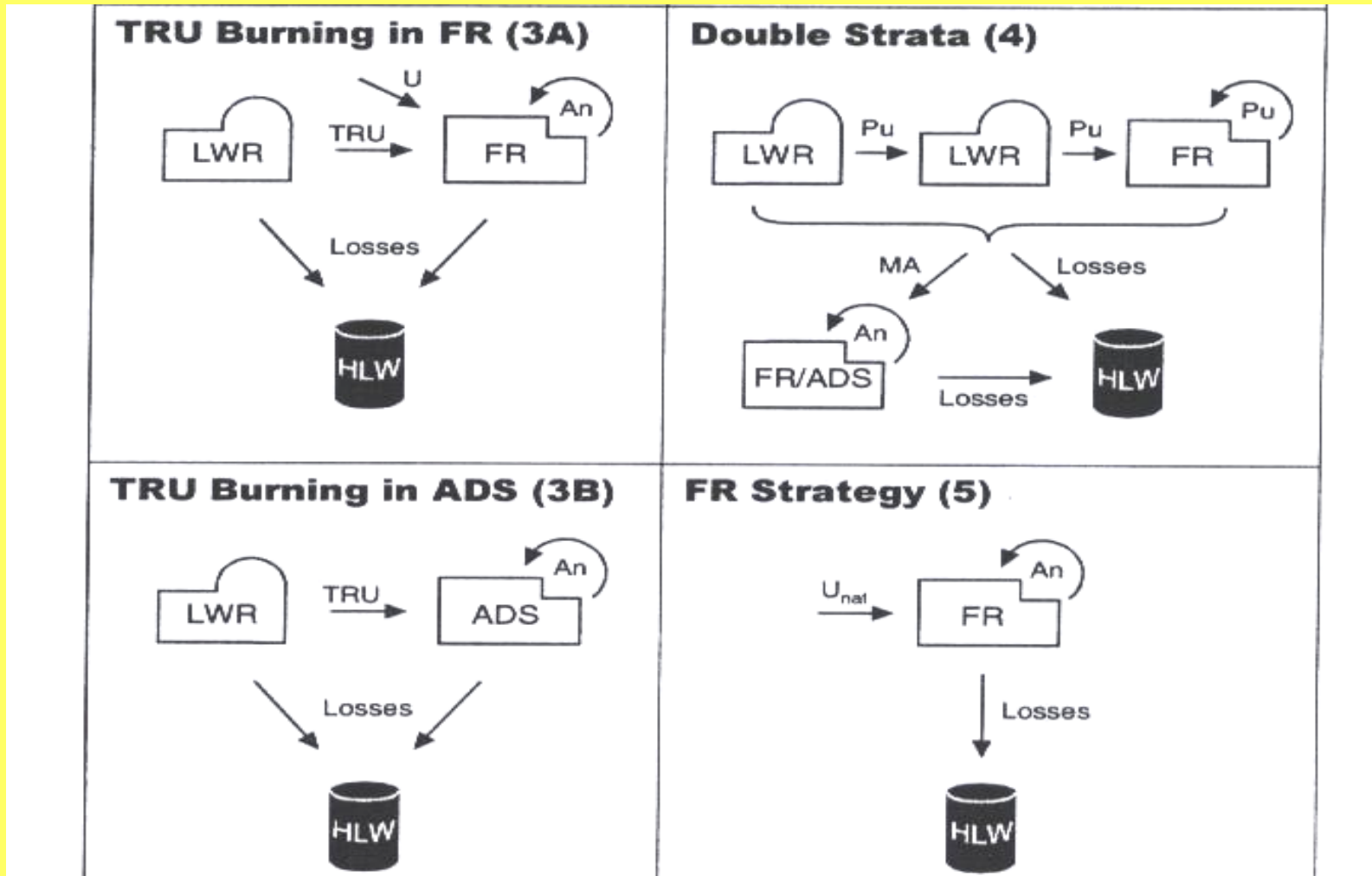
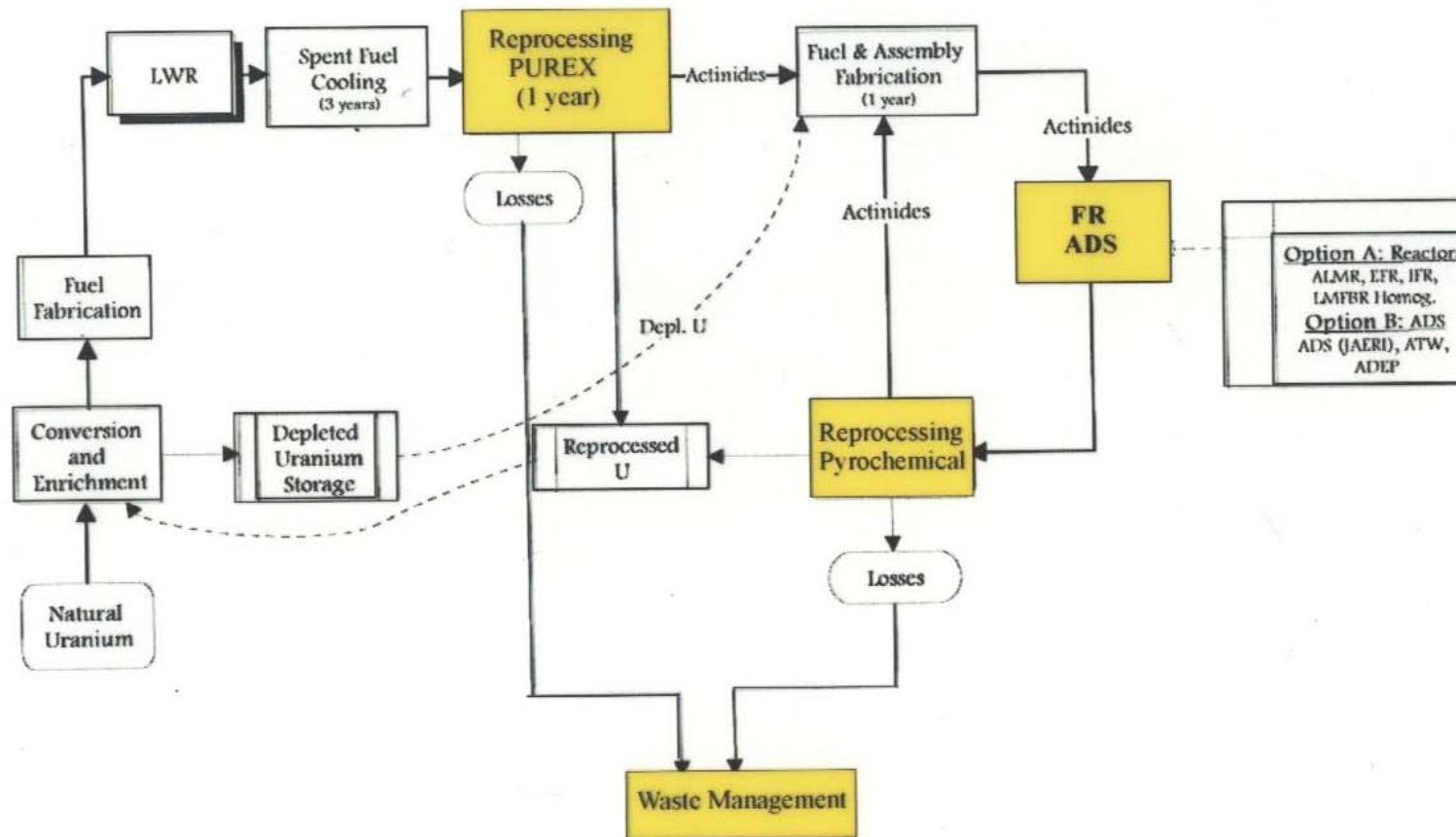
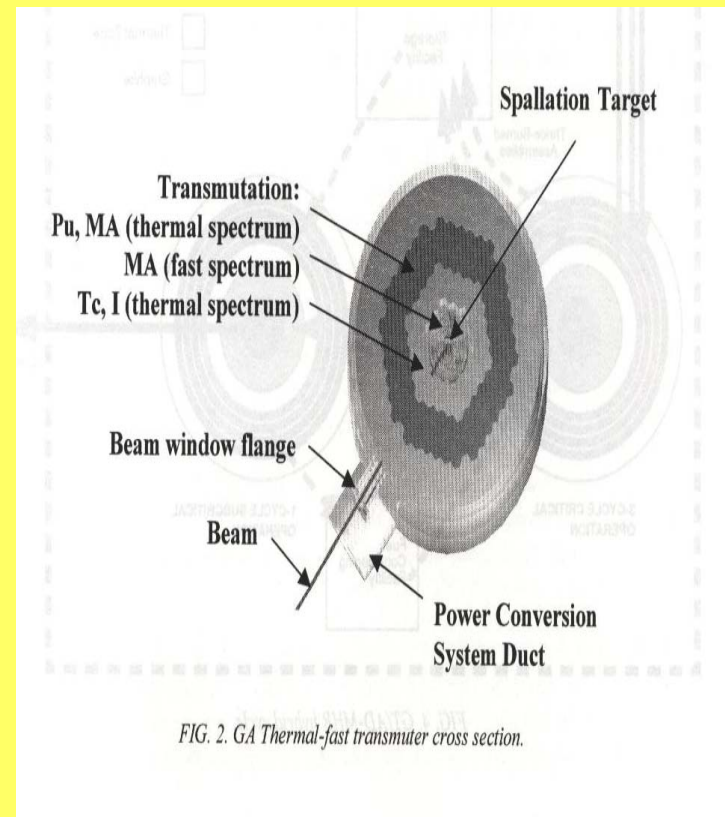
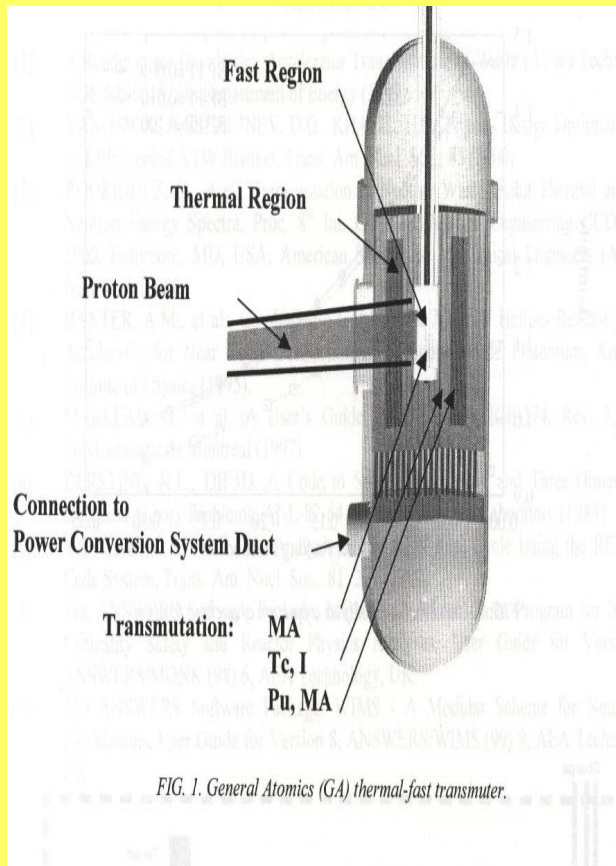


Figure 5. Flow-sheet of Nuclear Fuel Cycle Schemes 3



Accelerator driven subcritical fast reactor



ATW Technology Approach

- Superconducting Linac 1 GeV; 45 mA(CW); beam splitter into four beams which drive four 848 MW_{th} fast spectrum cores
- Spallation target Pb-Bi liquid target; neutrons/proton ≈ 30
- Subcritical core PB-Bi coolant
Dispersion (Met-Met) fuel 75%
ZR/25% TRU (no fertile)
K ≈ 0.97 @BOL
Discharge Burnup ≈ 33 a/o

ATW Technology Approach (Contd.)

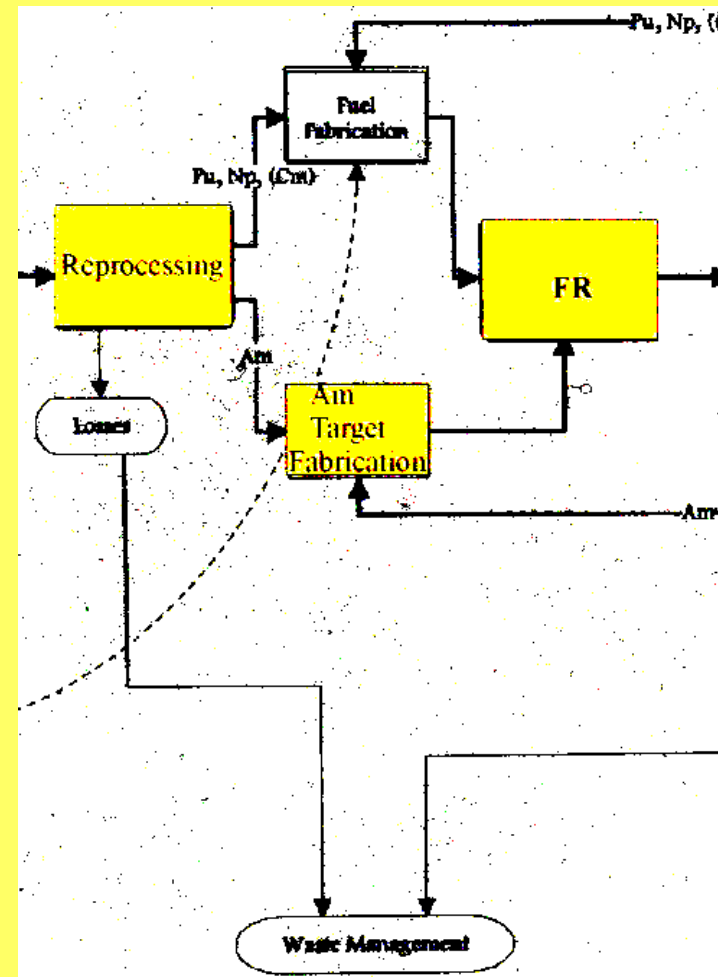
- **ATW Recycle** Chloride volatility to digest Zr; followed by TRU electrowinning fuel refab.: power metallurgy
- **LWR Treatment** One stage PUREX (UREX) to separate very clean U followed by Li reduction and electro-metallurgical
- **Waste Forms** Standard electrometallurgical ceramic and metal alloy forms
- **Process Losses** TRU < 0.1%
Tc, I < 5%

Fuels with inert matrices to avoid Pu-breeding (TRU content 10 - 40 %)

- Solid solutions of actinides in
(stabilised cubic) ZrO_2 , CeO_2 as oxides
 ZrN , CeN as nitrides
- Actinide ceramic phase in a ceramic matrix (cercer)
Actinide oxides in MgO , spinel MgAl_2O_4 , Y_2O_3 , SiC , possibly $\text{Y}_3\text{Al}_5\text{O}_{12}$ a “hybrid” concept is studied using the high radiation stability of ZrO_2 and the good thermal properties of spinel:
actinide-containing small beads of ZrO_2 in a spinel matrix
- Actinide ceramic phase in a metallic matrix (cermet)
Zircaloy, W, Nb, V, Cr, stainless steel

In separated Pu, Am-241 is growing-in during storage. Present Pu-MOX uses „hands-on“ technology. Therefore Am-241 needs to be separated out.

- In thermalised neutron flux Am-241 is faster transmuted*
- In Actinide-group separation we observe always a mixture of Am isotopes (241,242m,243)*
- For aqueous partitioning the critical mass of Am-242m is 19g compared to 3.8 Kg for pyroprocessing*



*The An-group separation has a higher proliferation
resistance*

Three partitioning processes are promising

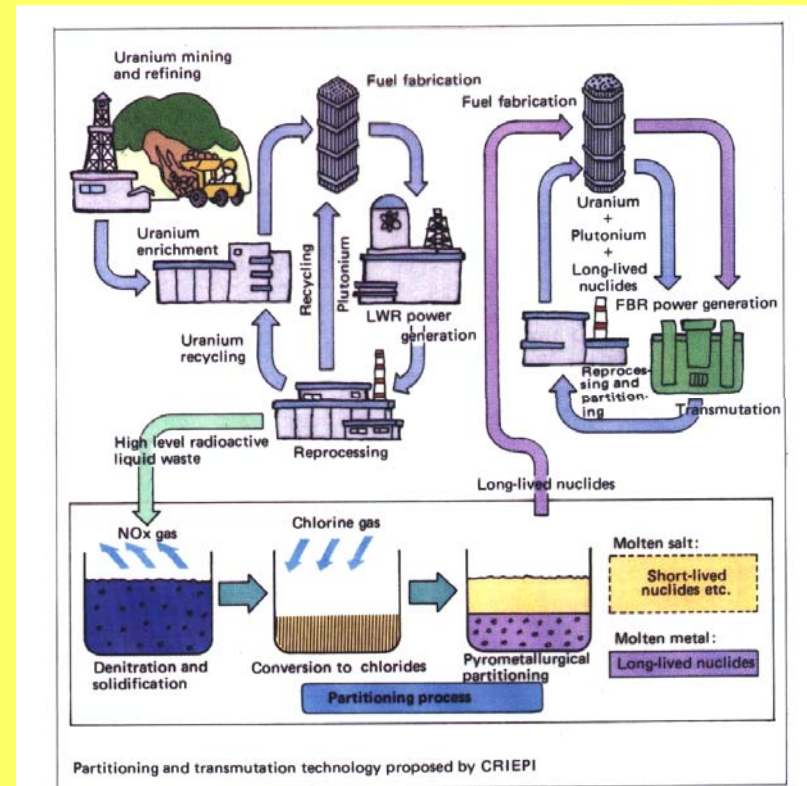
*CRIEPI process originates from IFR Pu
recovery*

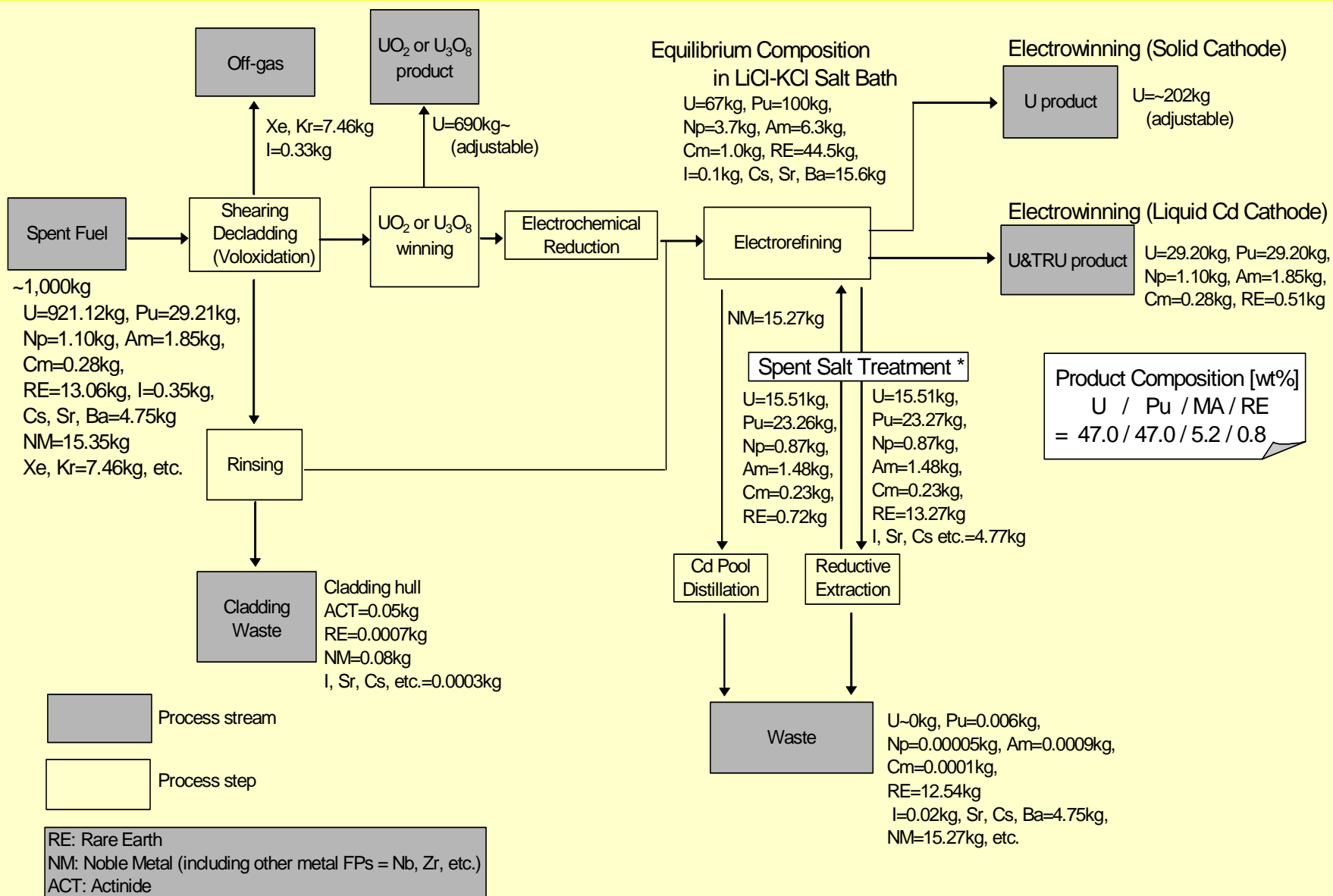
*GANEX proposed by CEA is still under
development*

*DOVITA was developed to separate Pu
(Np). It has to be extended to partition
TPu with high separation yield*

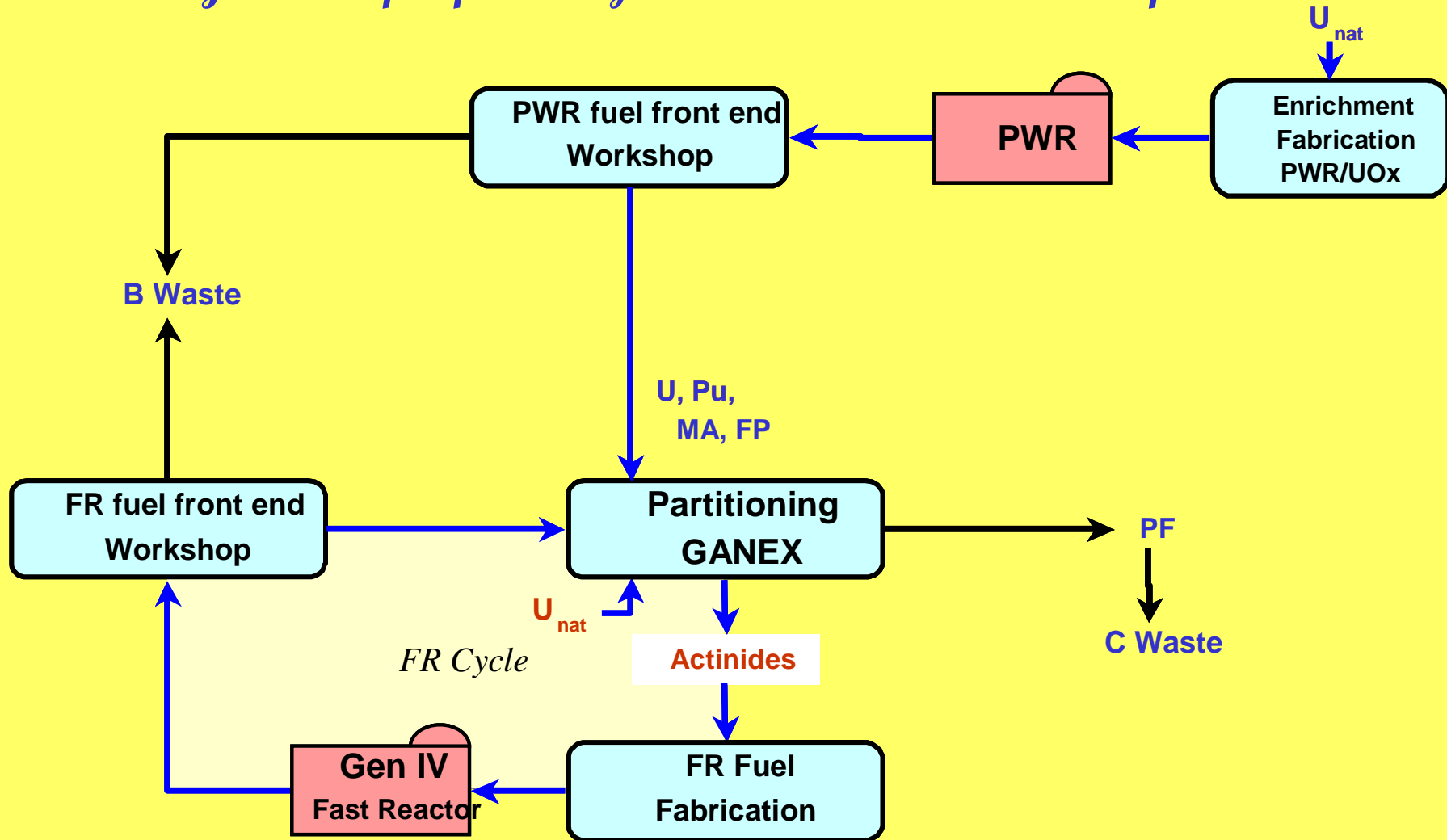
The An group separation has a higher proliferation resistance

- *CRIEPI process originates from JFR Pu recovery.*
- *(Presently only the HEU is separated at the shut-down JFR at Idaho, USA.)*

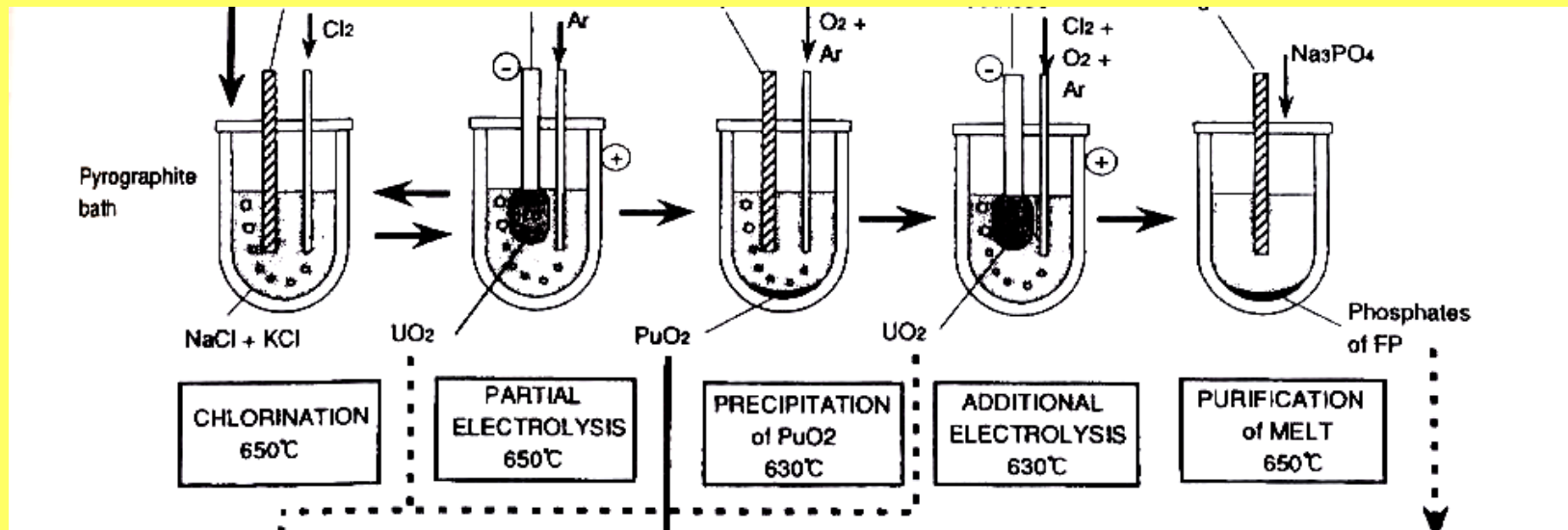




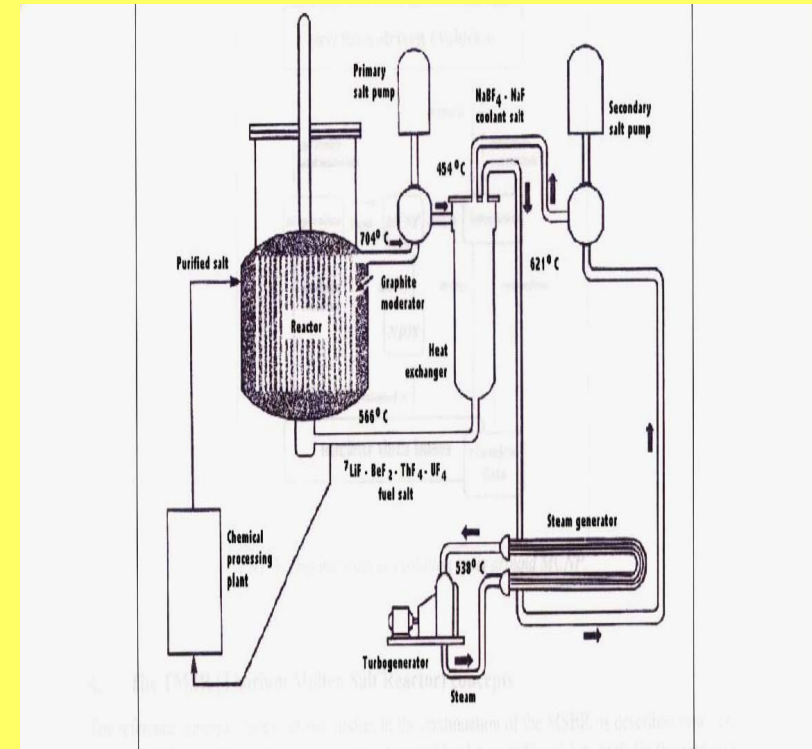
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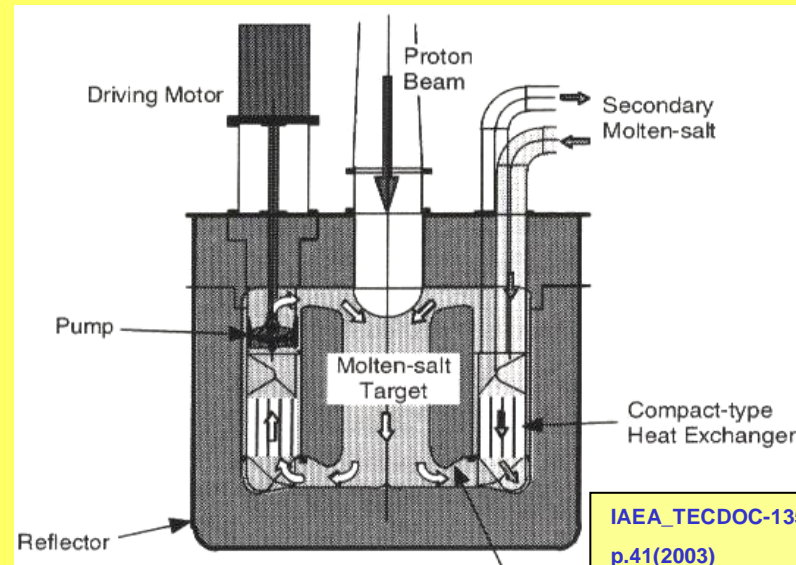
DOVITA was developed to separate Pu (Np). It has to be extended to partition I Pu with high separation yield



- *The Molten Salt Breeder Reactor, MSBR, once operating in ORNL, USA is based on the Th-U-233 cycle-*
- *To reduce the parasitic neutron capture of the fission products, they are separated out in-line*



- *The principle of the MSBR is extended to an ADS concept.*
- *Different salt mixtures are proposed. Fluorides must be preferred over chlorides.*



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