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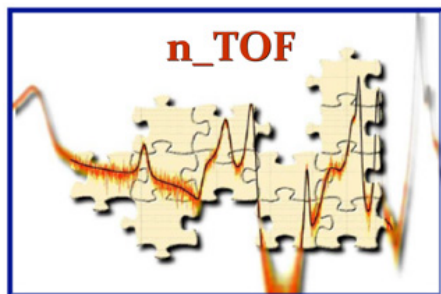
**Joint ICTP-IAEA Workshop on Nuclear Reaction Data for Advanced
Reactor Technologies**

19 - 30 May 2008

**Measurements of capture cross-section of Minor Actinides with a Total Absorption
Calorimeter at n_TOF.**

Nicola Colonna
*Universita' degli Studi di Bari
Istituto Nazionale di Fisica Nucleare-INFN
Via Orabona, 4
70125 Bari
ITALY*

Measurements of capture cross-section of Minor Actinides with a Total Absorption Calorimeter at n_TOF



Nicola Colonna

Istituto Nazionale Fisica Nucleare, Sez. di Bari

V. Orabona 4, 70126 Bari, Italy

nicola.colonna@ba.infn.it

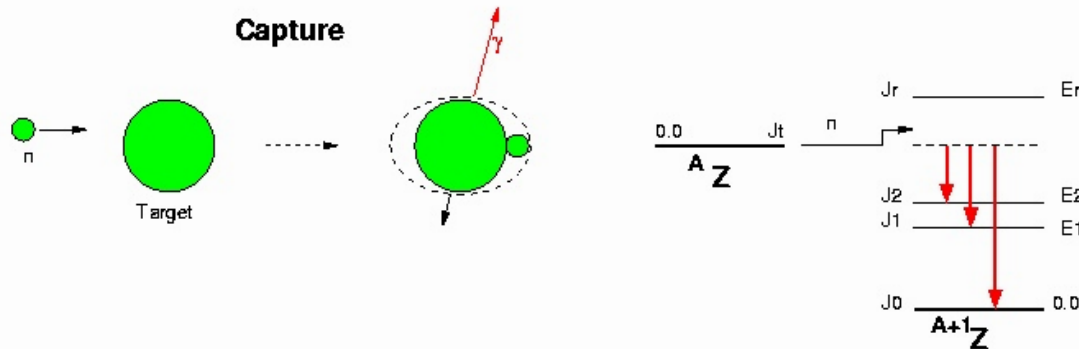


ICTP-IAEA, Trieste, 19-28 May, 2008

Outline

- What is a Total Absorption Calorimeter (and how does it work)
- The n_TOF Calorimeter
- Some preliminary results

The capture reactions



In a neutron capture reaction, excited compound nucleus is formed with $E_x = E_n + S_n$, (neutron energy and neutron separation energy, $5 < S_n < 10$ MeV).

Measurements of neutron capture cross-sections are performed by detecting γ -rays emitted from deexcitation of compound nucleus formed in the capture.

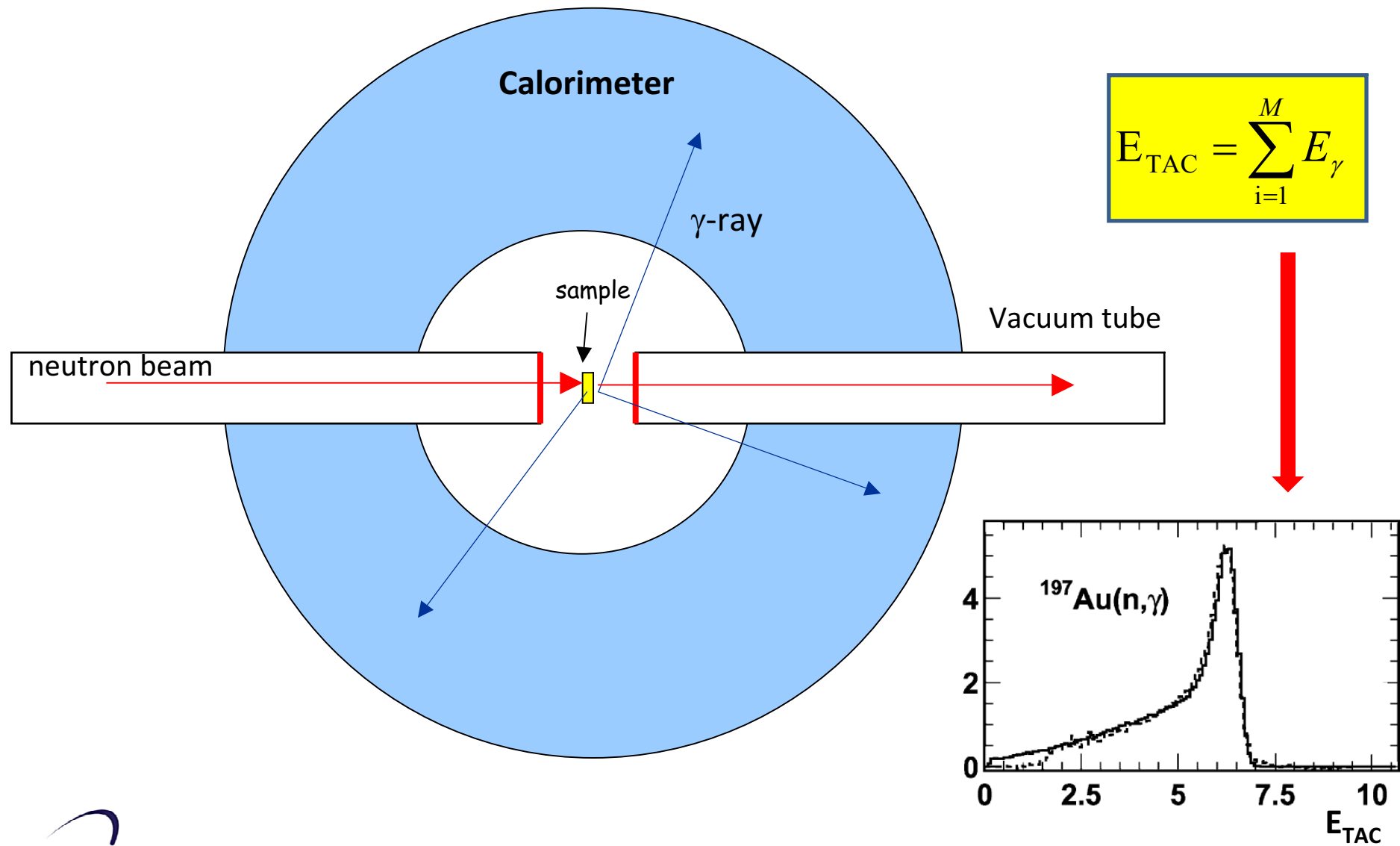
Two methods:

- Single γ -ray detection (shown before) low efficiency, need to correct for efficiency
- Detection of the entire cascade better, but more complicated (and expensive)

Calorimetric method: identify neutron capture reaction on the basis of the total energy of the cascade ($E_{\text{tot}} = E_n + S_n$) and on the **multiplicity** of the cascade.

In order to detect the whole cascade, it is necessary to have a detector that covers the **whole solid angle** (4π detector), and with very **high detection efficiency**.

The calorimetric method



Calorimeter vs single γ -ray detection

Single γ -ray detection

- Choose detectors with **low neutron sensitivity**
- low-cost (few detectors)
- efficiency to capture γ -rays **low and dependent** on cascade
- requires **Pulse Height Weighting Technique** for “efficiency corrections”
- **Difficult to distinguish** between capture and background γ -rays

Calorimetric method

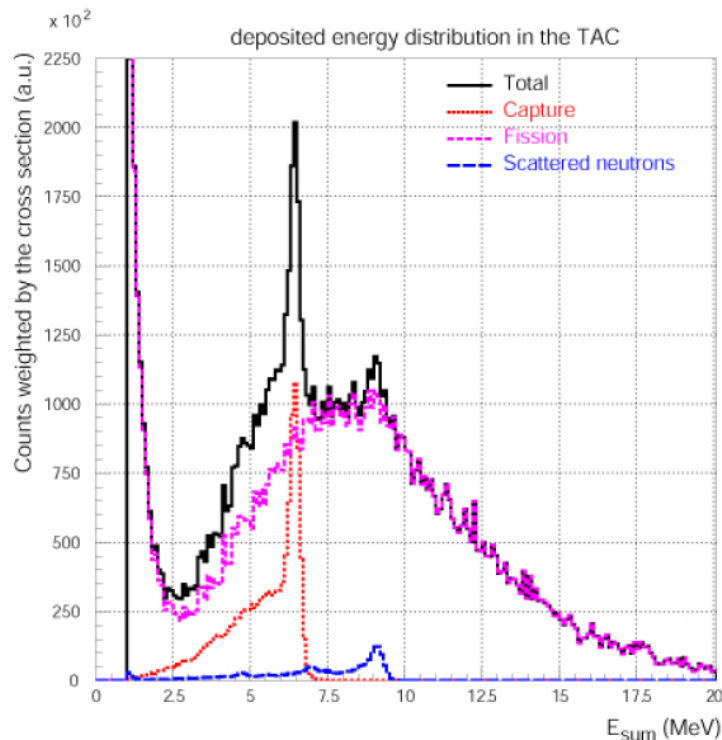
- **high** efficiency
- good **background rejection** capability
- Possibility to distinguish competing reactions (fission)
- **poor** neutron sensitivity
- many detectors
- **high costs and complexity** (many detectors)

For radioactive isotopes, calorimetric method is better since it allows better **rejection of γ -ray background** related to natural radioactivity:

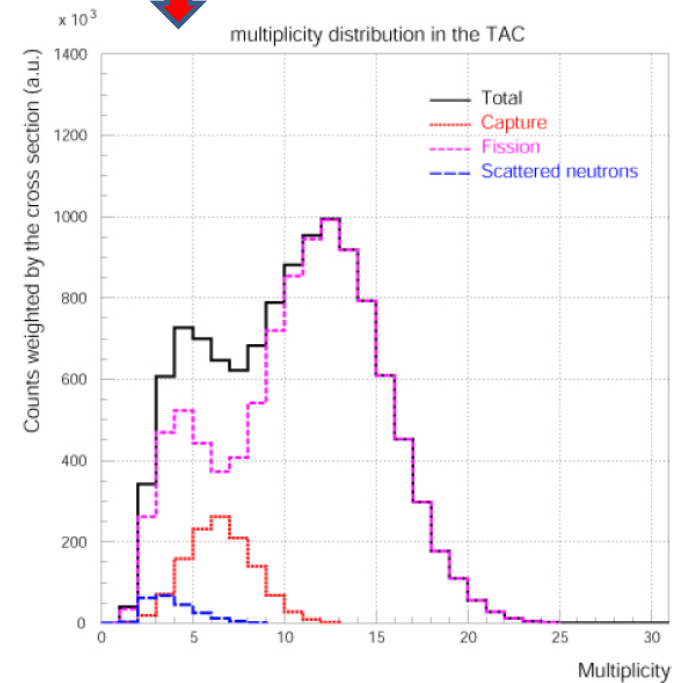
- energy of background γ -rays typically few MeV, well below capture energy;
- multiplicity of background γ -rays $M=1$, while for capture $M\sim 3$

Calorimeter vs single γ -ray detection

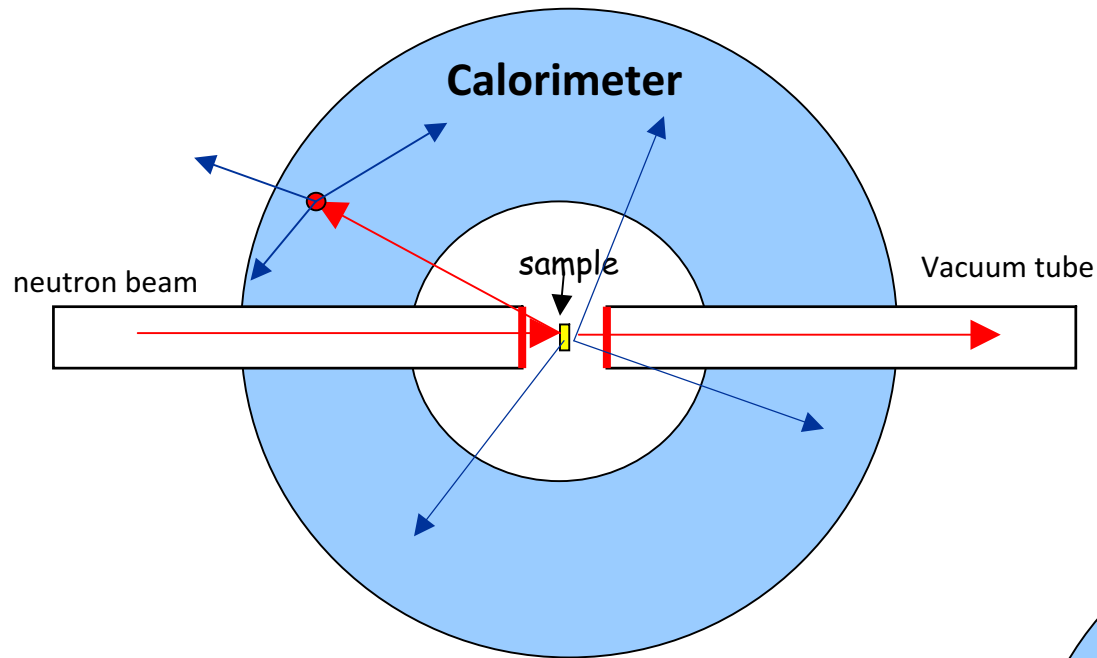
A very important consideration: a calorimeter allows to distinguish capture from other competing reactions that produce γ -rays (for example fission). Very powerful method for measurements of capture cross-sections of **fissile isotopes** (minor actinides)



Simulations of the calorimeter response for the fissile ^{245}Cm



The problem of neutron sensitivity



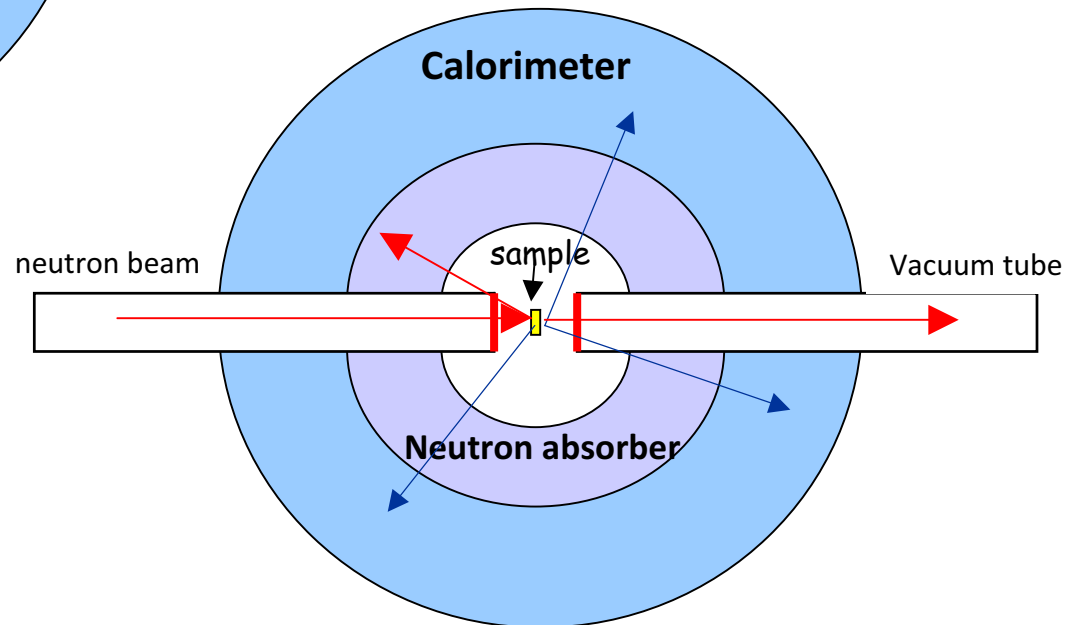
A neutron scattered in the sample can be captured inside the detector.

Cannot be easily distinguished from “capture” events in sample.

Important source of background (depends on elastic cross-section).

A solution consists in using an internal sphere of **neutron moderating and absorbing material**.

Also “**shield**” detectors from thermal neutrons.



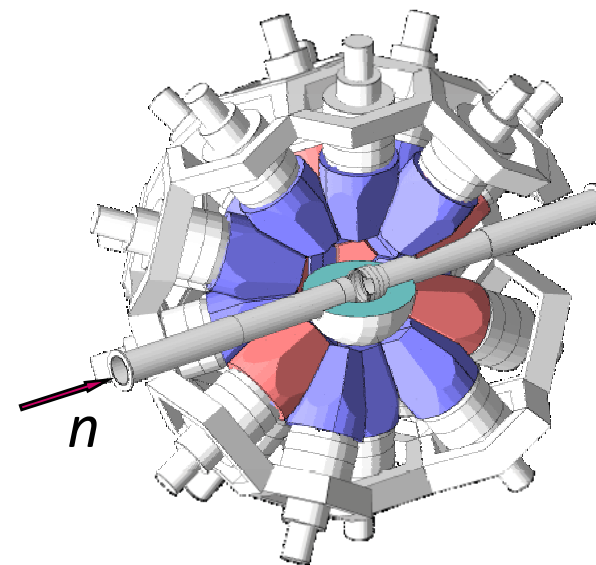
A real calorimeter

How should a calorimeter be made:

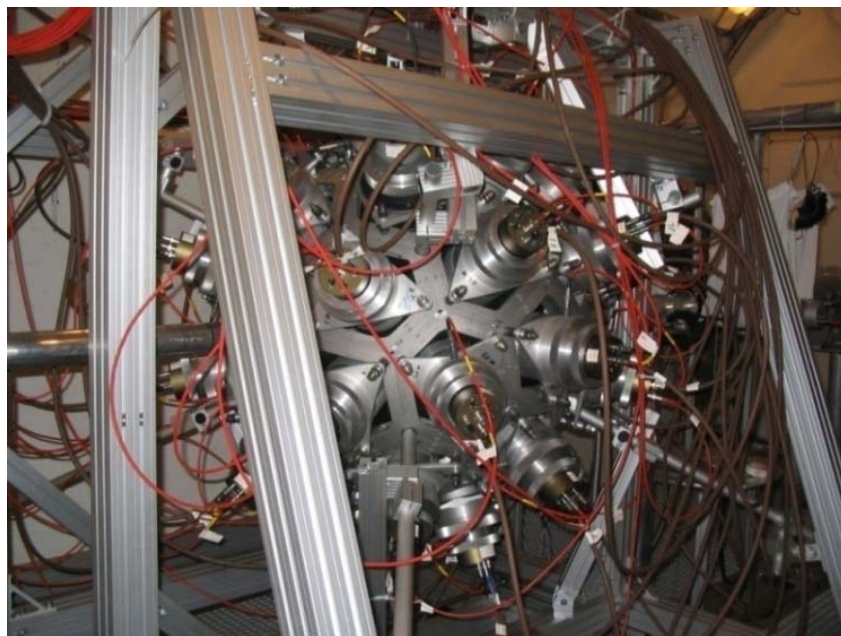
- large number of detectors (for multiplicity reconstruction)
- high-Z material (for high detection efficiency)
- big thickness (for high efficiency)
- fast response (for good timing properties)
- low neutron sensitivity

The n_TOF Total Absorption Calorimeter (TAC):

- 40 crystals of BaF_2 , covering 95 % of 4π
- **15 cm** thickness (98 % efficiency to 5 MeV γ -rays)
- Internal sphere of ^6Li -carbonate, to moderate and absorb neutrons scattered from the sample
- capsules in carbon fibre loaded with ^{10}B to minimize neutron sensitivity



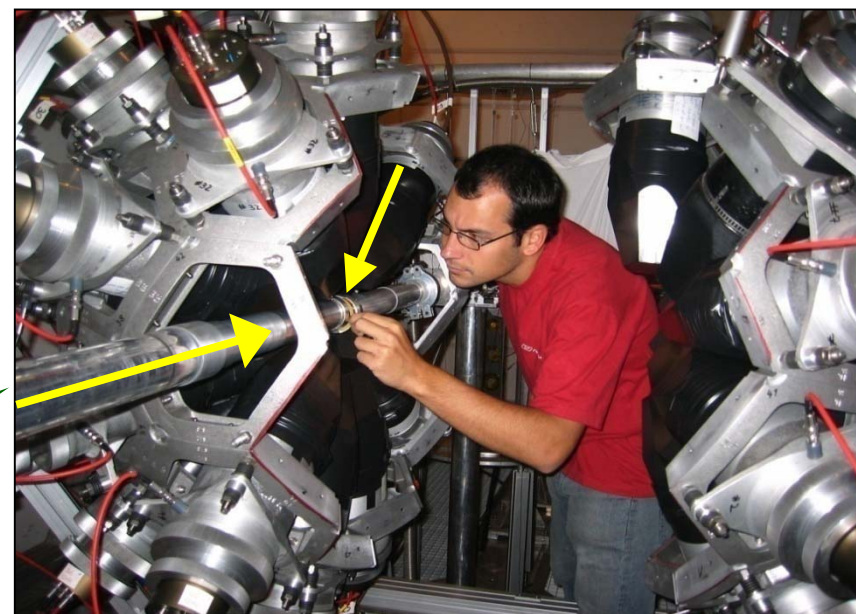
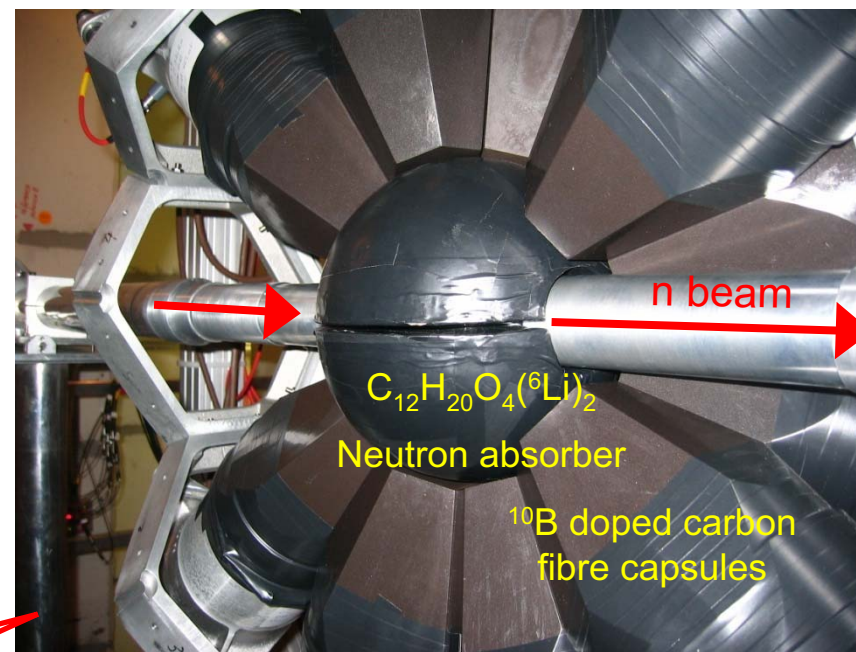
Very high efficiency to a single capture event (close to 100 %)



Inner view of the calorimeter

Calorimeter from the outside (closed)

Mounting of a sample

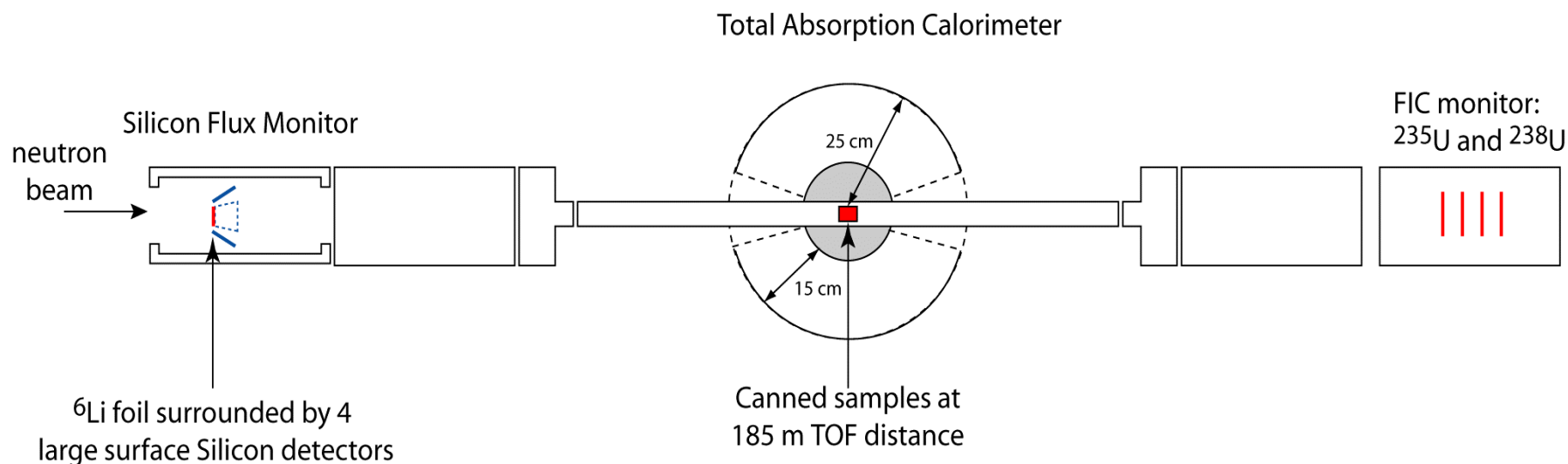


The n_TOF TAC

N.B.: The n_TOF calorimeter is not the only one used in the world:

- TAC at FZK operated since long time
- TAC at LANSCE constructed at the same time of n_TOF

However, better neutron sensitivity, characterization, analysis procedure, etc...



n_TOF TAC mounted inside the experimental area, in conjunction with a flux monitor (need to know the flux in capture measurements) !!

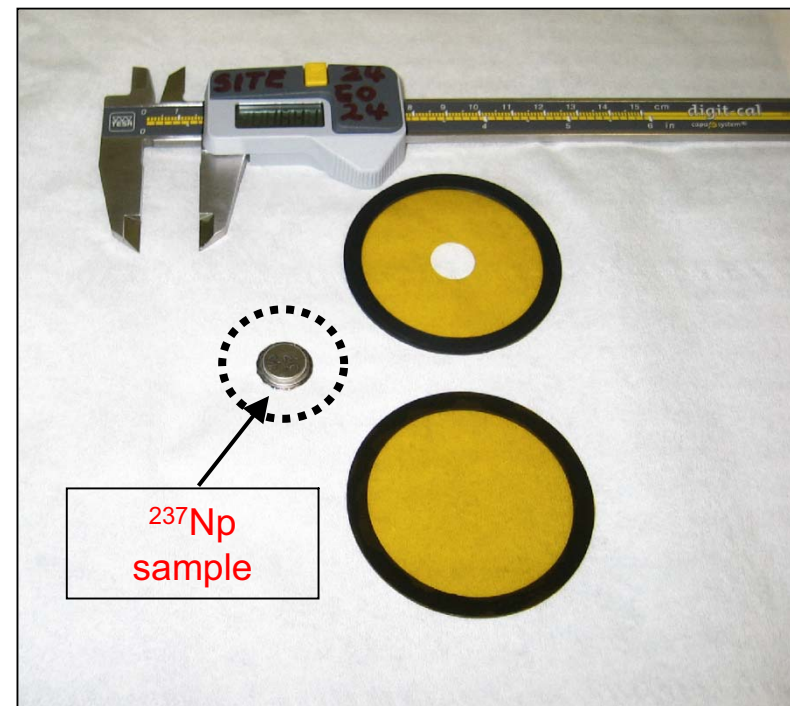
The capture measurements with the TAC at n_TOF

Sample preparation:

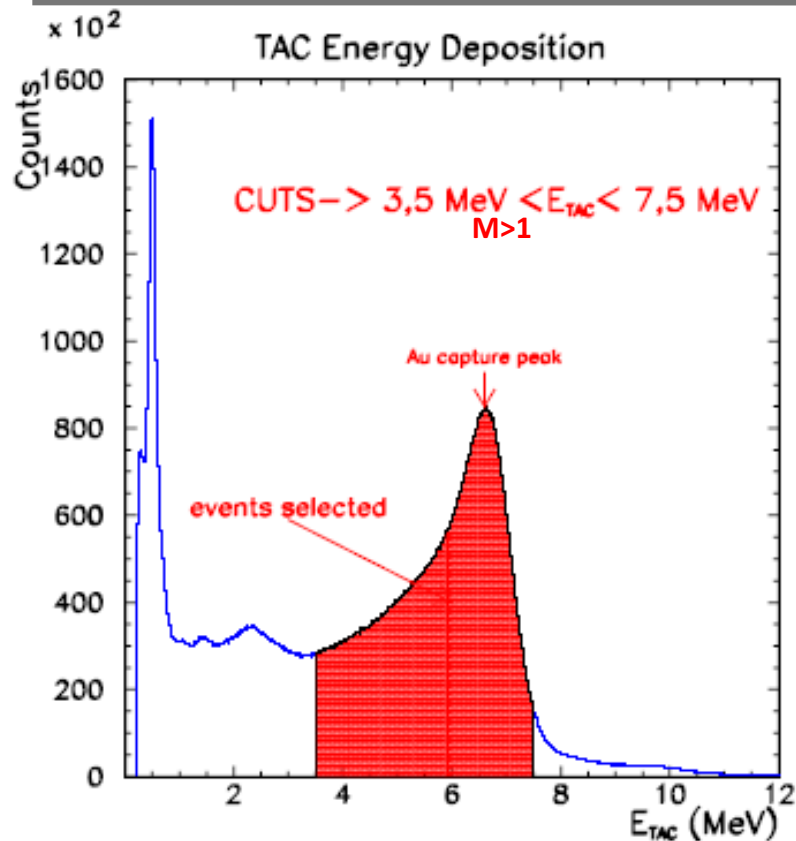
- 1 cm diameter
- sealed in Ti can (for safety reasons)
- precisely aligned in the beam

Measured isotopes (2004):

^{197}Au	paper in
preparation		
^{234}U	in preparation
^{233}U	data analysis
^{237}Np	first draft
^{240}Pu	first draft
^{243}Am	data analysis



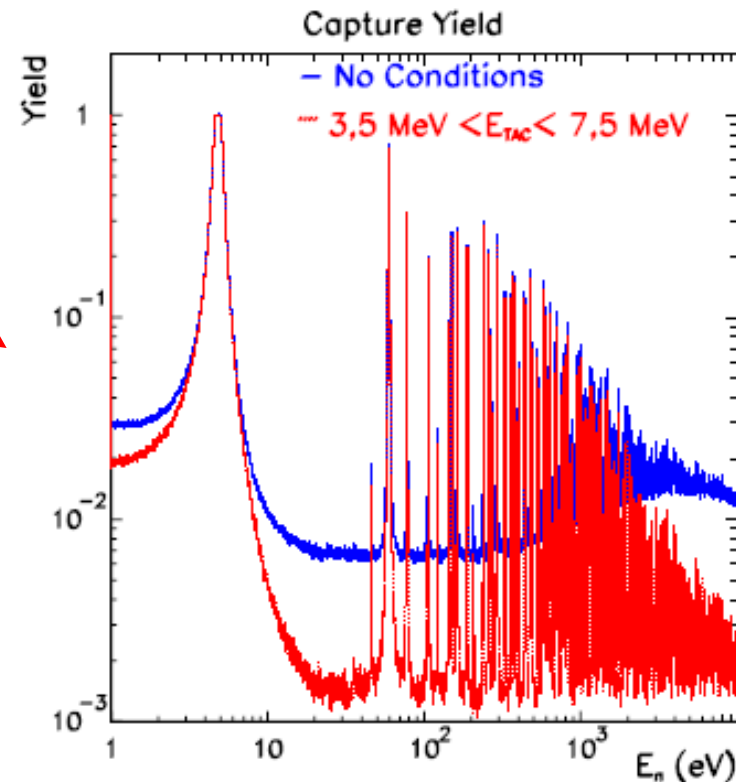
Data analysis: the $\text{Au}(n,\gamma)$ cross-sections



The $\text{Au}(n,\gamma)$ reaction is well known (capture standard):

- useful to understand how the calorimeter work
- can be used for checking Monte Carlo simulations

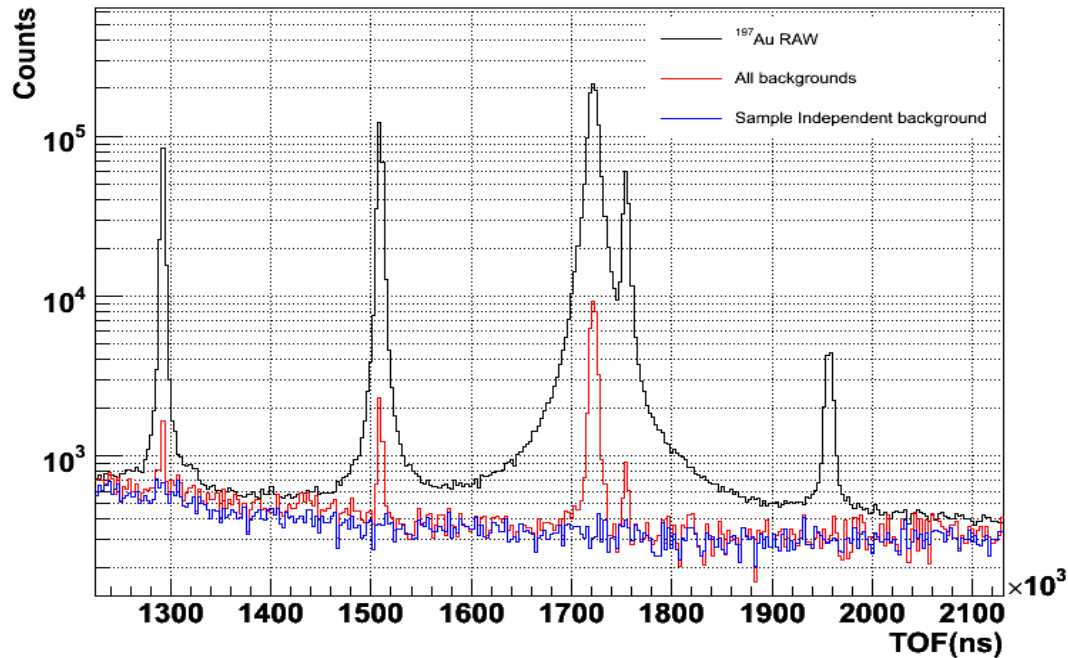
P.S.: The energy deposition for a capture event does not produce a sharp peak at 6.5 MeV because of resolution of detectors.



The condition on the Total Energy in TAC suppresses drastically the background

Data analysis: the $\text{Au}(n,\gamma)$ cross-sections

TAC rate as a function of TimeOfFlight, $m_\gamma > 2$ & $2.5 < E(\text{MeV}) < 7.5$



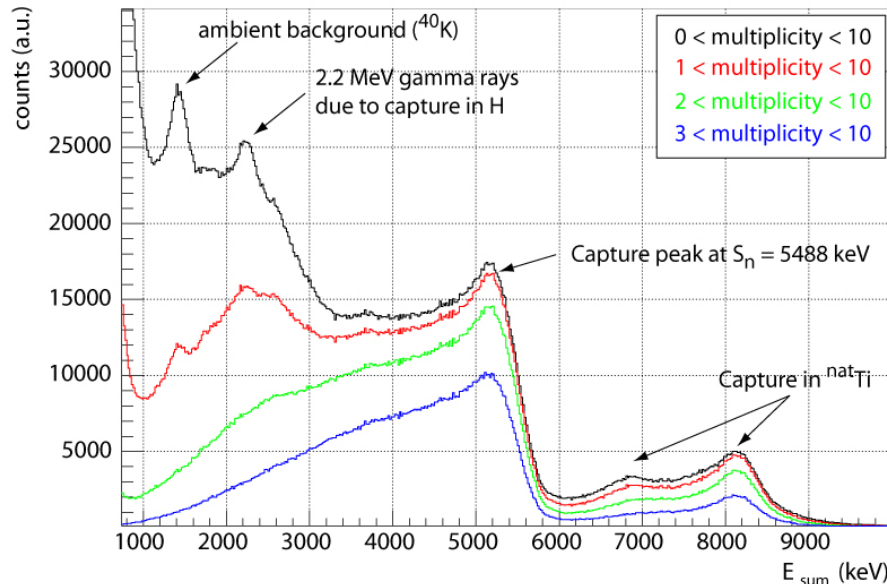
Attention: some neutron sensitivity survives in the calorimeter.

However, contribution of the background caused by neutron scattering is $< 1\%$.

N.B.: The background due to neutron scattering (neutron sensitivity is estimated with a C sample)

Data analysis: the $^{237}\text{Np}(n,\gamma)$ cross-sections

Energy deposition spectrum for ^{237}Np in the neutron energy range 1 eV - 10 eV



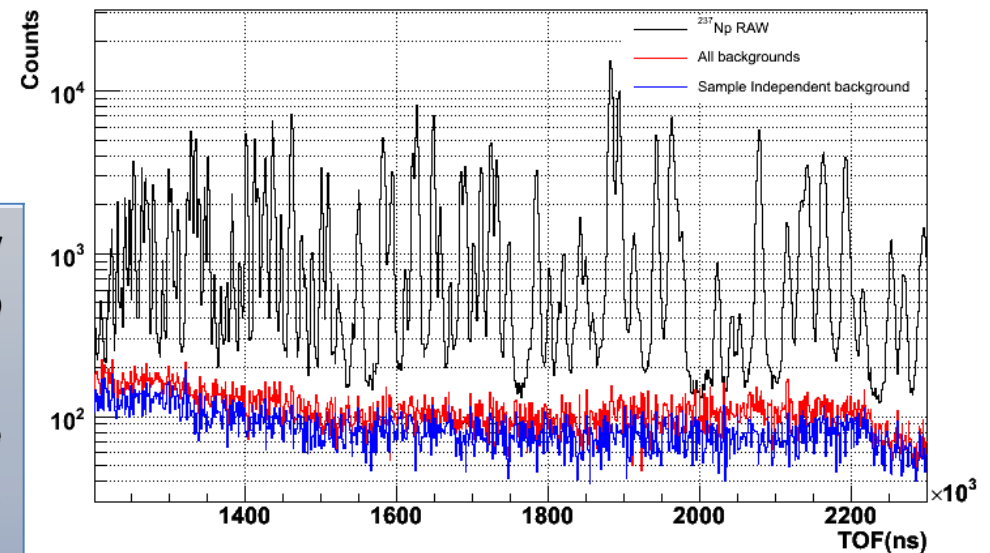
The condition on the multiplicity suppresses the background, but also changes the efficiency (decreases).

Efficiency corrections require extensive Monte Carlo simulations: Geant 4, MCNP-X, Fluka, etc...

The background can be rejected in two ways:

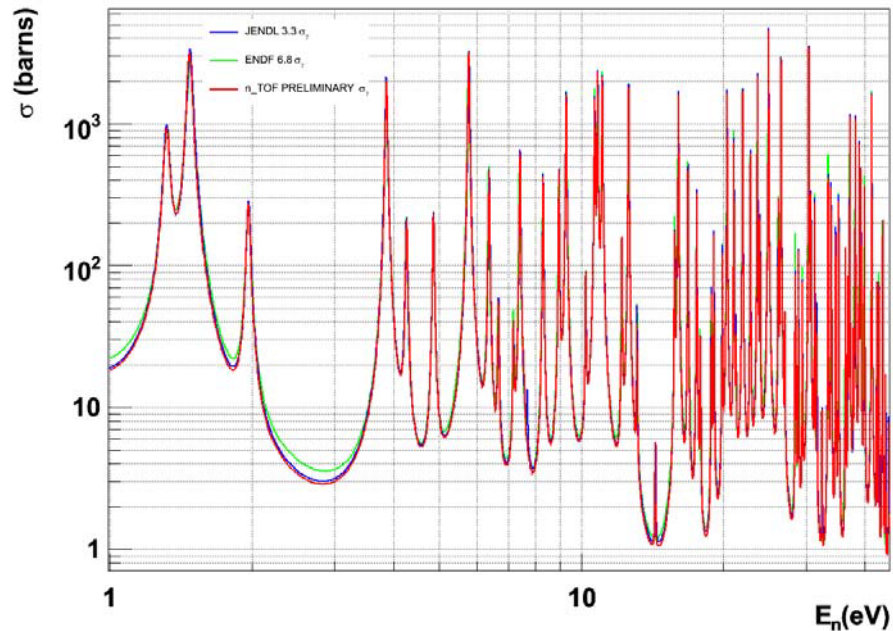
- condition on the deposited energy
- condition on the multiplicity of γ -rays

TAC rate as a function of TimeOfFlight, $m_\gamma > 2$ $2.5 < E(\text{MeV}) < 6$



Resonance analysis of $^{237}\text{Np}(n,\gamma)$

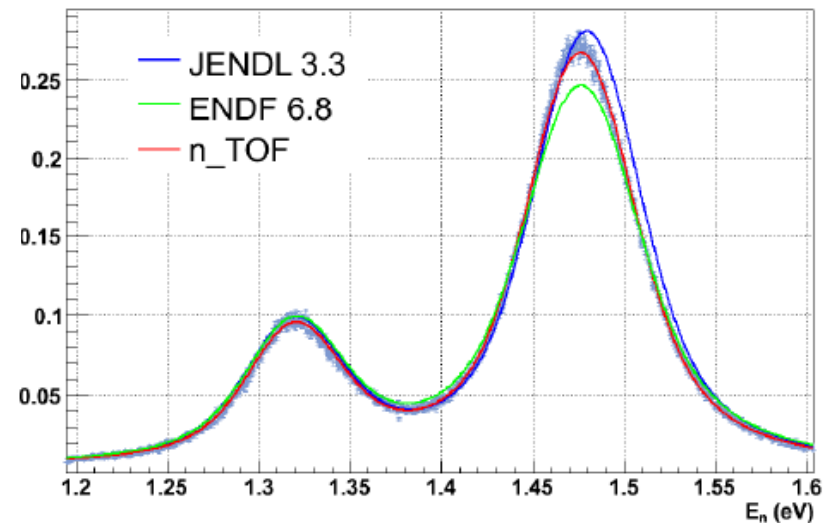
$n_{\text{TOF}}^{237}\text{Np } \sigma(n,\gamma)$ compared to Evaluated Data Libraries



Typically done with the code SAMMY
(see talk and exercises by S. Marrone)

The measured resonances have to be “fitted” (R-matrix analysis) to extract accurate resonance parameters.

^{237}Np experimental Yield fitted with SAMMY



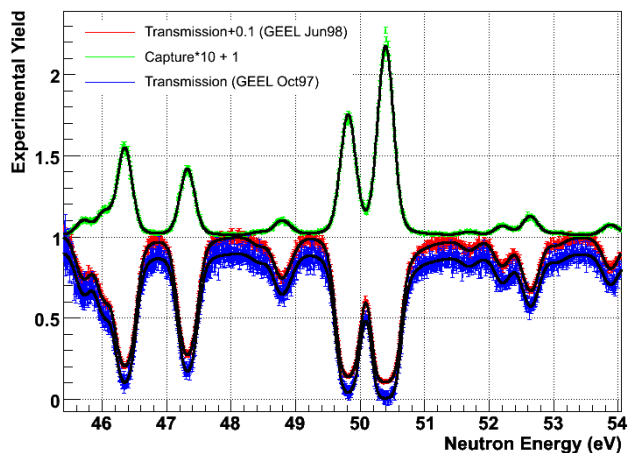
As discussed by P. Schillennebecks, accurate resonance parameters (partial widths Γ_n and Γ_γ) can be obtained by combining capture and transmission measurements.

^{237}Np (n, γ) measurement with the TAC

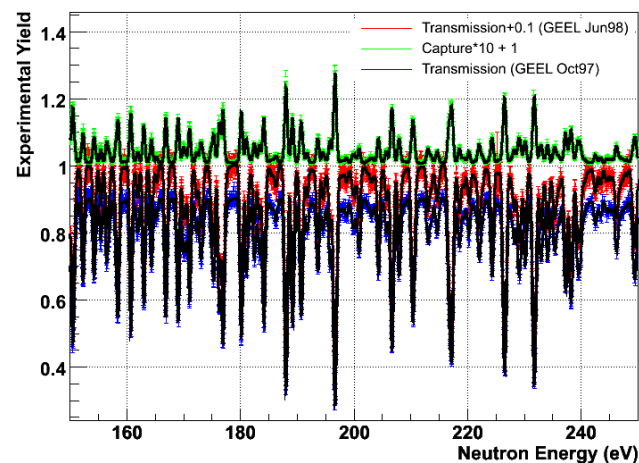
43.3 mg, 1.29 MBq



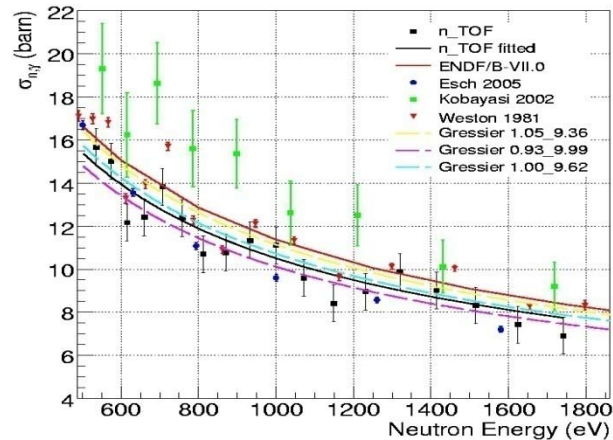
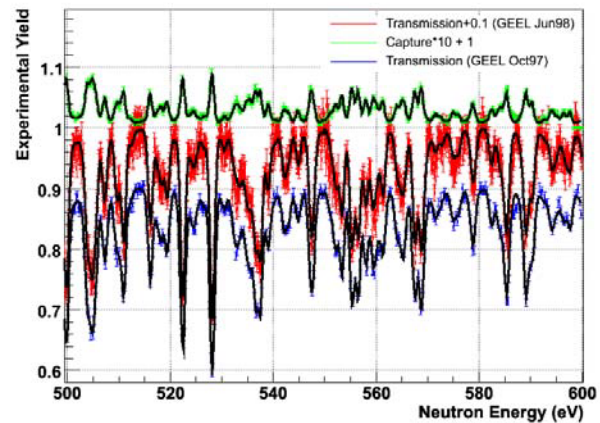
^{237}Np Capture (nTOF) + Transmission (GEEL) Yield



^{237}Np Capture (nTOF) + Transmission (GEEL) Yield

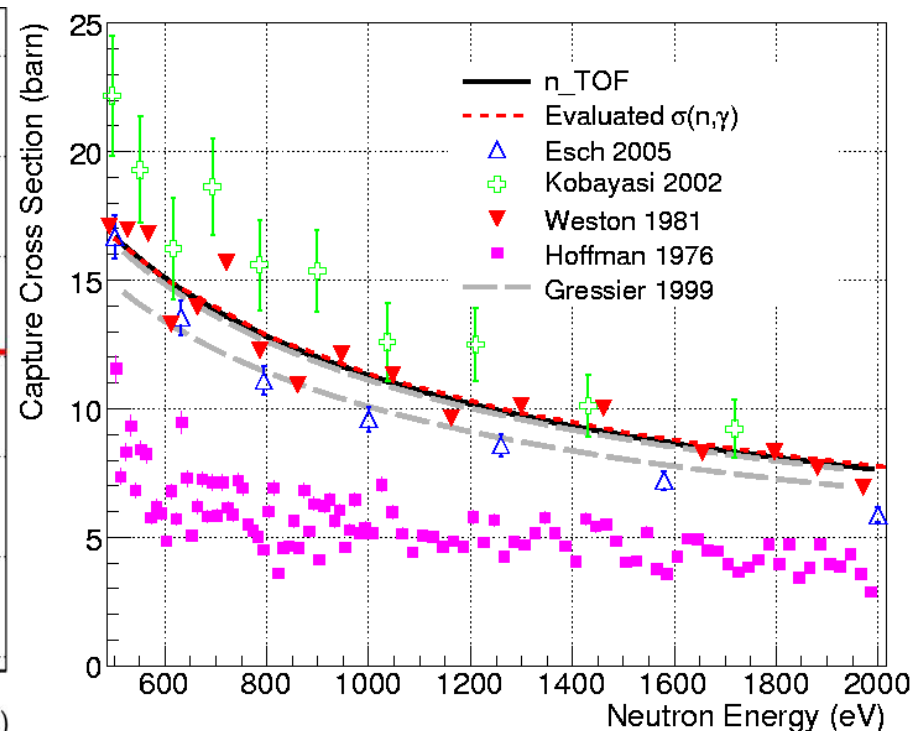
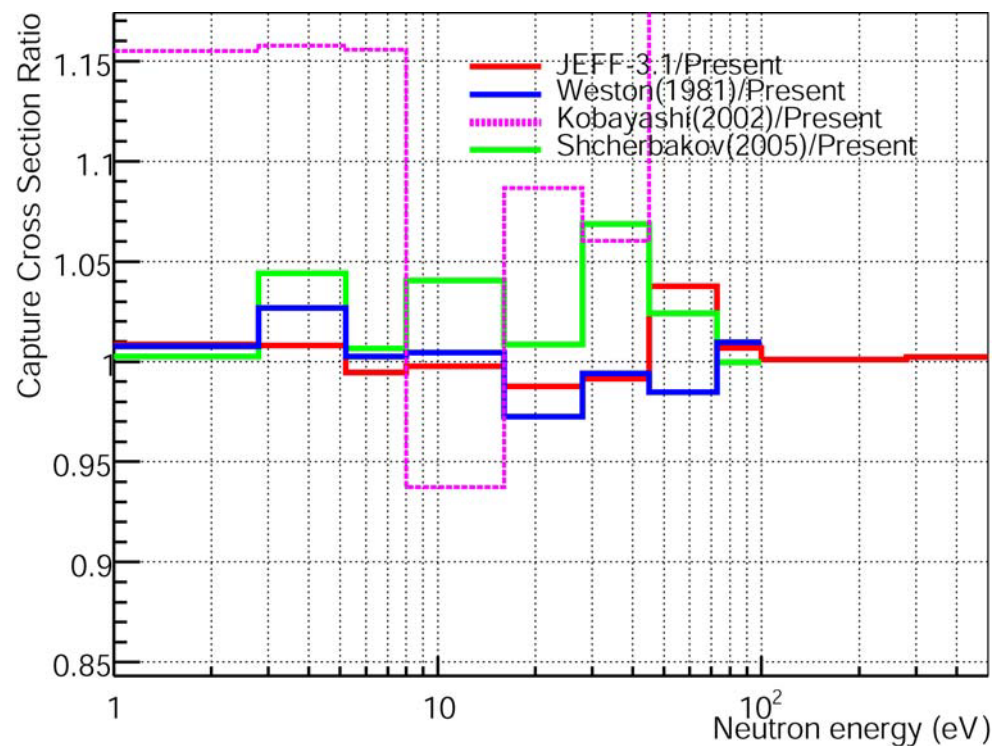


^{237}Np Capture (nTOF) + Transmission (GEEL) Yield



C. Guerrero et al. (n_TOF Collaboration), Proc. Int. Conf. Nuc. Data for Sci. and Tech. 2007, Nice.

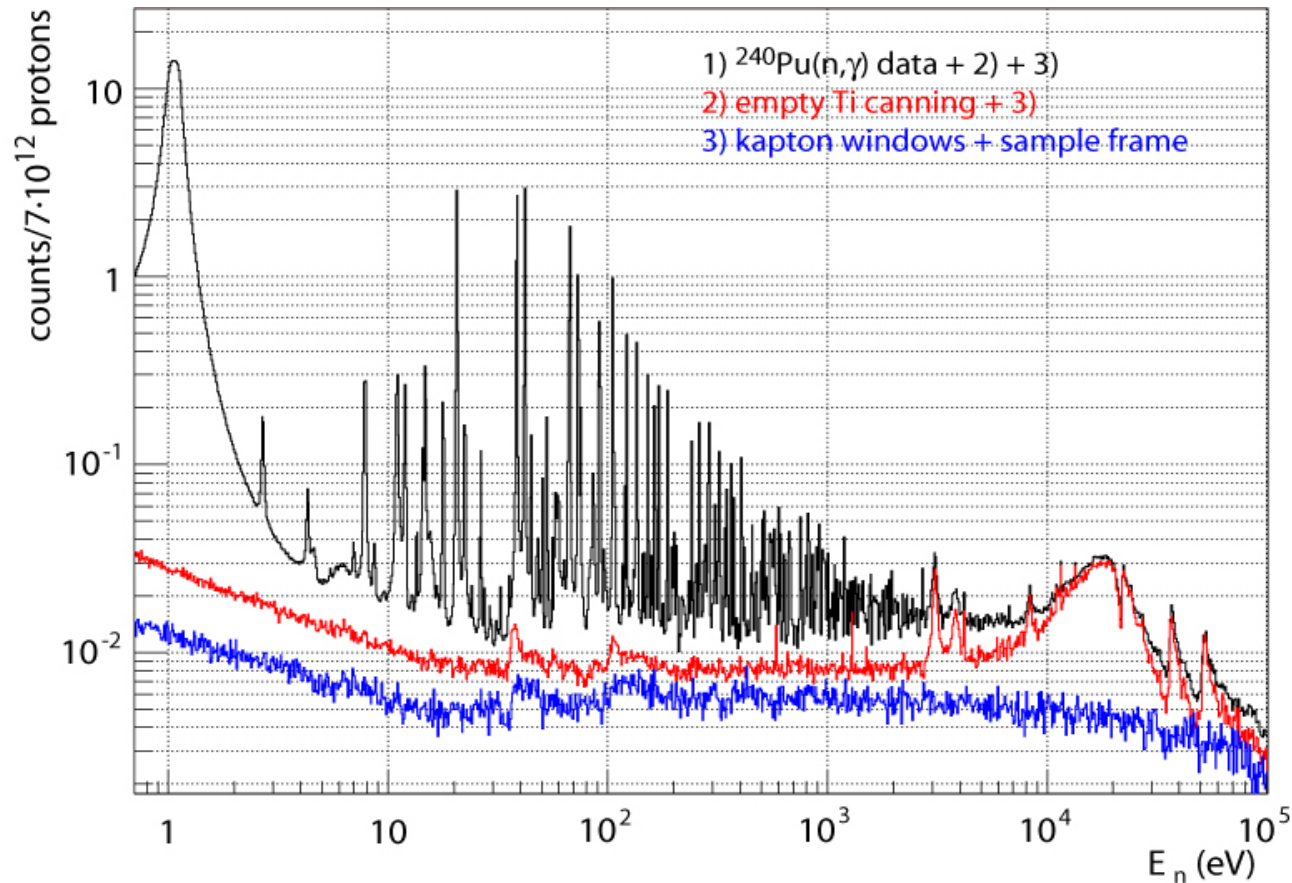
The $^{237}\text{Np}(n,\gamma)$ in the URR



JEFF-3.1 = JENDL-3.3 = ENDF/B-VII from (n,tot) at GELINA (V. Gressier et al.)

The new data with high accuracy (5 %), help solve discrepancies between databases

Data analysis: the $^{240}\text{Pu}(n,\gamma)$ cross-sections



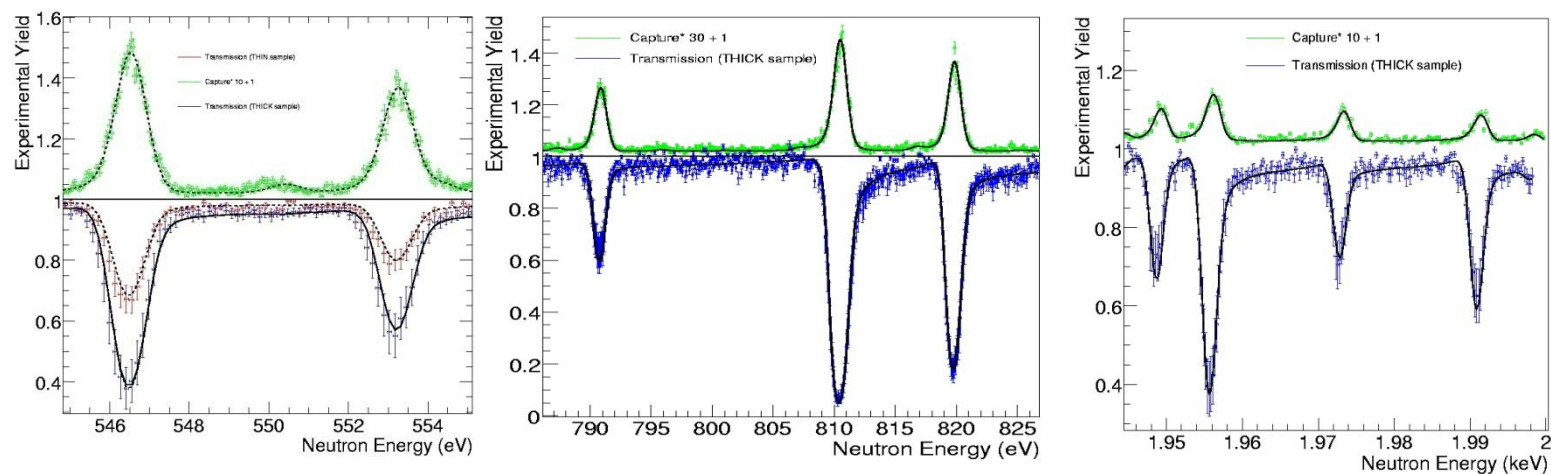
Below 10 keV, low background. Accurate measurement of the Resolved Resonance Region.

Above 10 keV, background dominated by neutron capture in ... Ti



^{240}Pu (n, γ) measurement

51.2 mg, 458 MBq

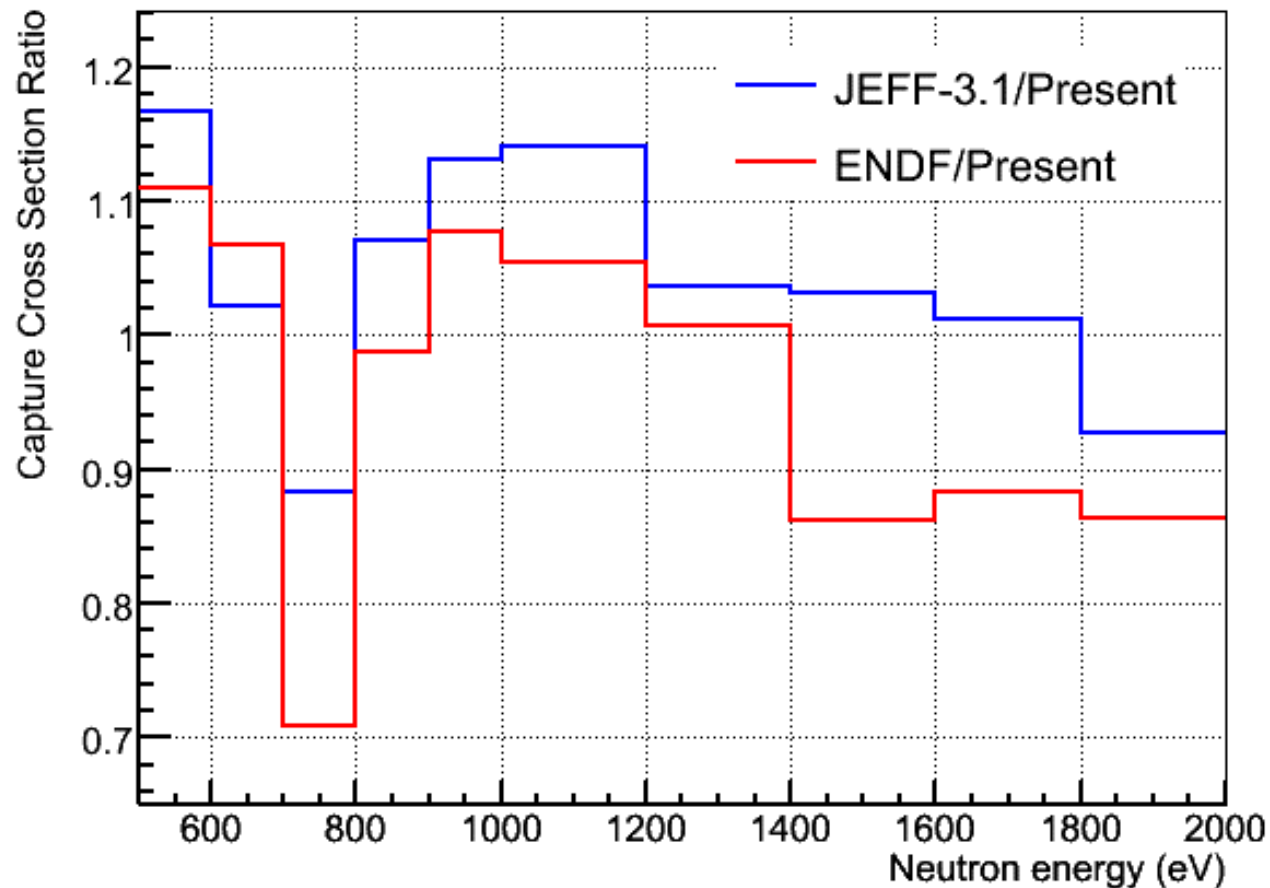


	ENDF/B-VII	n_TOF	Bouland et al.
$\langle D_0 \rangle$ (eV)	12.1	12.1 ± 0.1	12.06
$\langle \Gamma_\gamma \rangle$ (meV)	30.6	32.4 ± 0.8	31.9
$S_0 (10^{-4})$	1.13	1.04 ± 0.08	1.07

C. Guerrero et al. (n_TOF Collaboration), Proc. Int. Conf. Nuc. Data for Sci. and Tech. 2007, Nice.

The $^{240}\text{Pu}(n,\gamma)$ reaction

Salvatores *et al.*, Nucl. Sci. Eng. 146 (2004) $\rightarrow \text{UNC}(\sigma_\gamma) = 10\%$

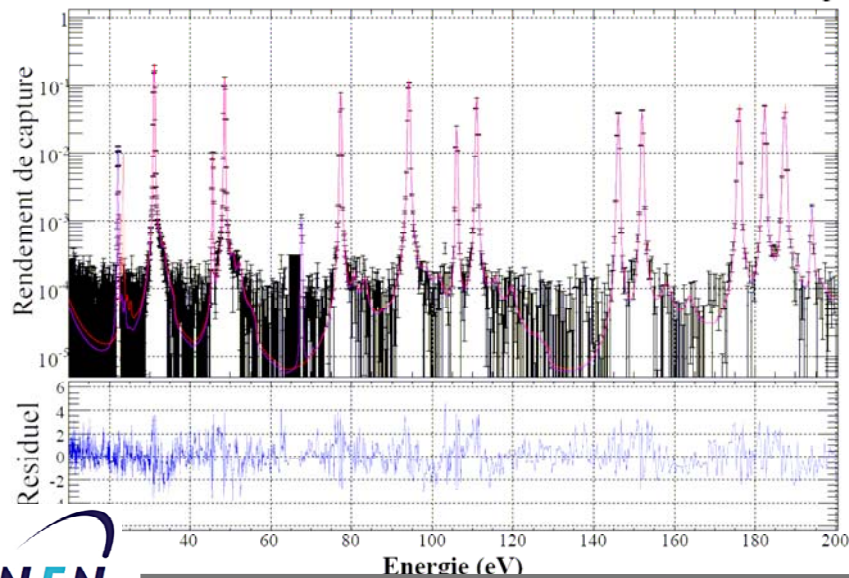
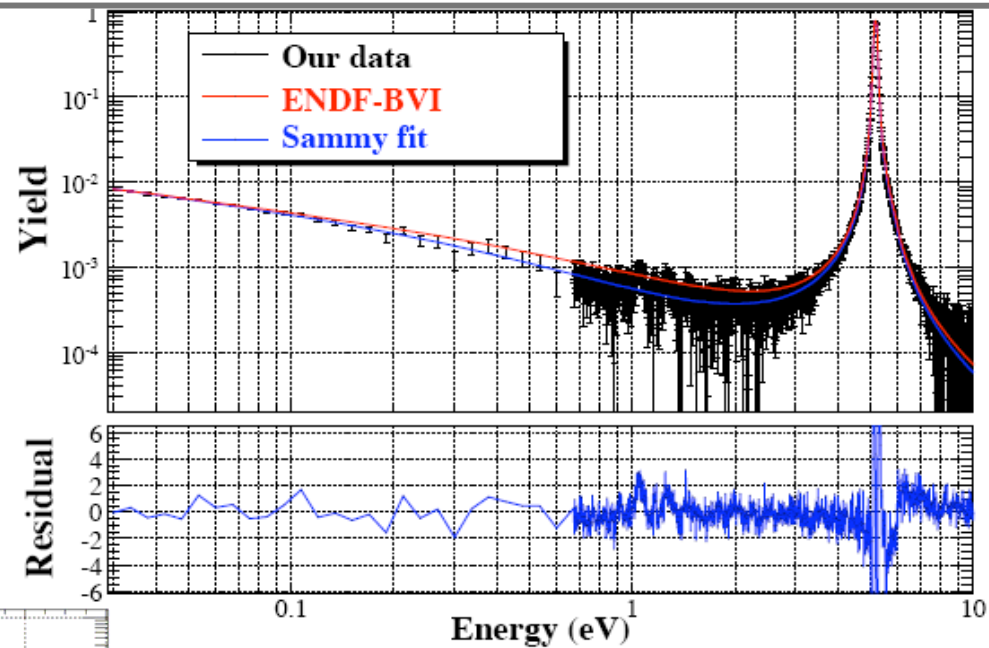


ENDF/B-VII based on (n,tot) at GELINA by Kölar *et al.* (1968)

JEFF-3.1 = JENDL-3.3 based on (n,tot) at GELINA by Kölar (Bouland *et al.* 1998)

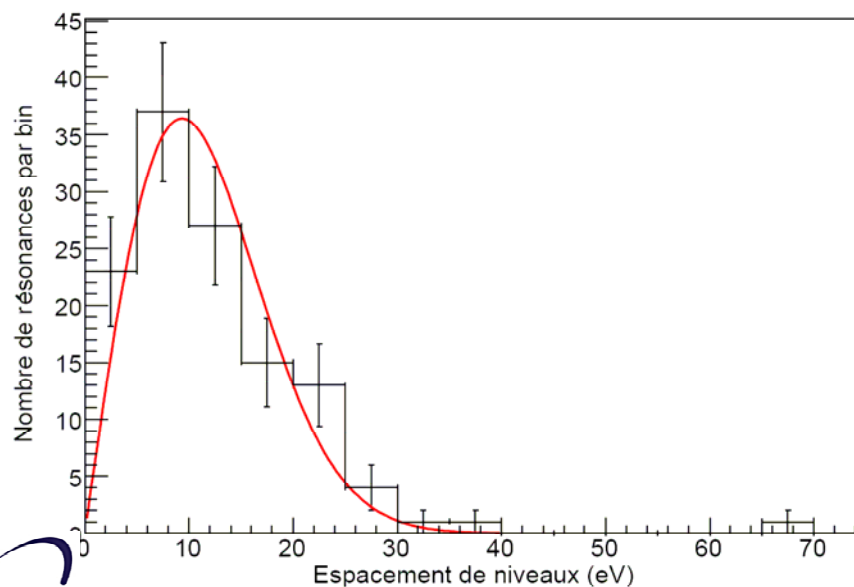
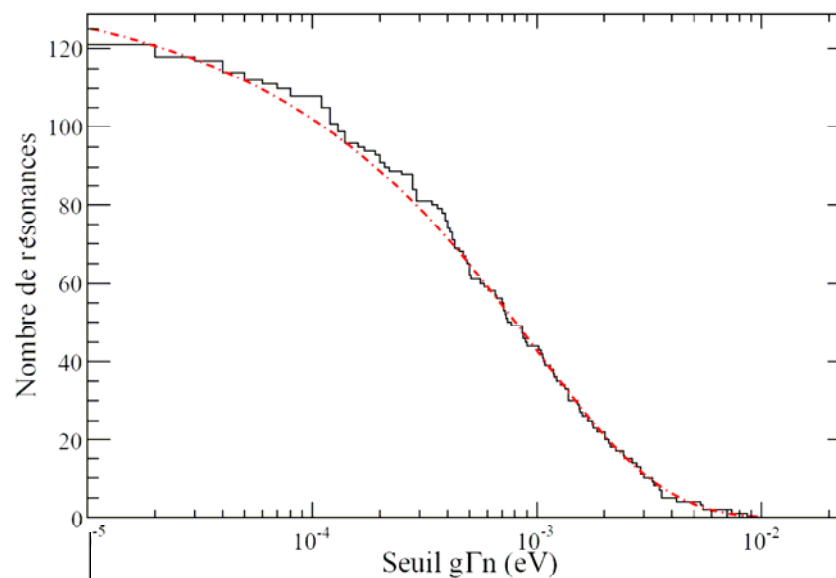
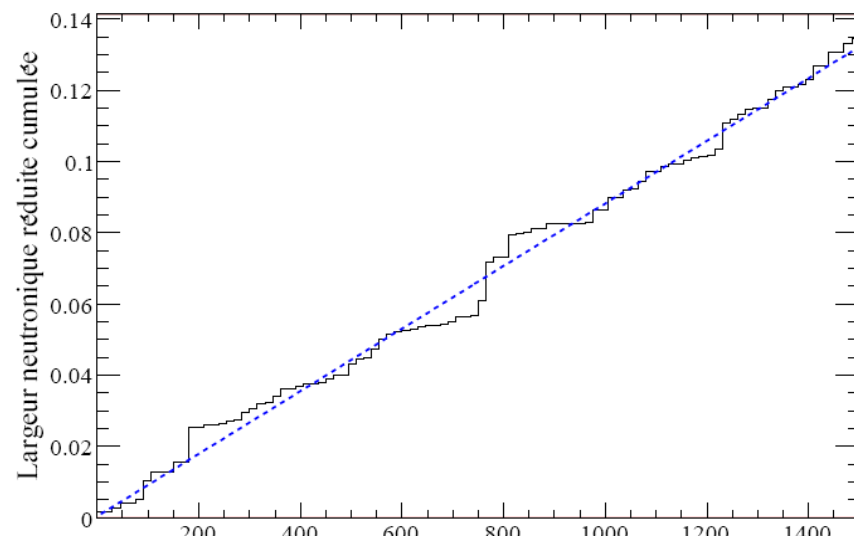
The $^{234}\text{U}(n,\gamma)$ reaction

33 mg, 0.2 MBq
(piece of cake)



High quality data (very accurate)
Waiting only to be published

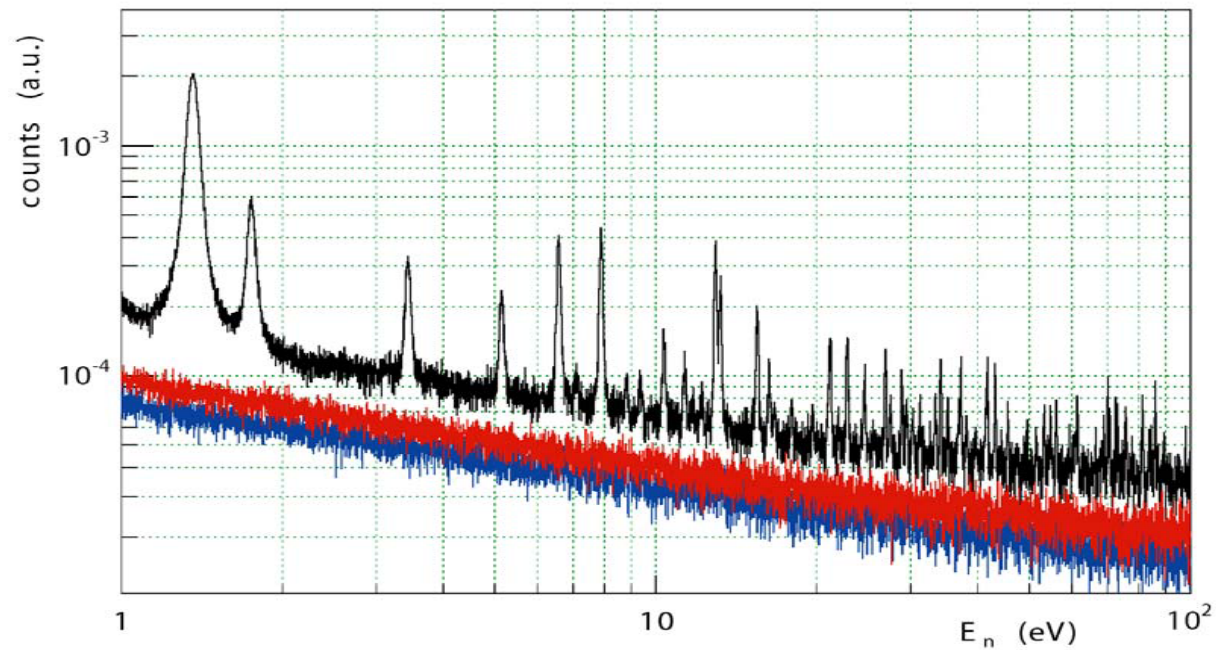
Statistical analysis of $^{234}\text{U}(n,\gamma)$



Allows to obtain information on average level spacing, neutron strength function, and other nuclear properties

Very important for model calculations

^{243}Am (n, γ) measurement with the TAC



First (n, γ) measurement **EVER**. 10 mg, 75 MBq

D. Cano-Ott et al. (n_TOF Collaboration), Proc. Int. Conf. Capture Gamma-Ray Spec. 2005, Santa Fe.

Conclusions

The detection of the entire γ -ray cascade allows to discriminate capture events from background (in particular the one related to natural radioactivity of the sample or to competing reactions).

Background rejection based on multiplicity and total energy reconstructed.

A fundamental device for measuring capture cross-sections of fissile elements (fission reactions rejected on the basis of total energy of γ -rays)

Thanks to its features, the TAC allows to measure with high accuracy capture cross-sections for Pu isotopes and minor actinides involved in advanced nuclear reactors.