



**The Abdus Salam
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Organised and self-organised magnetic confinement.

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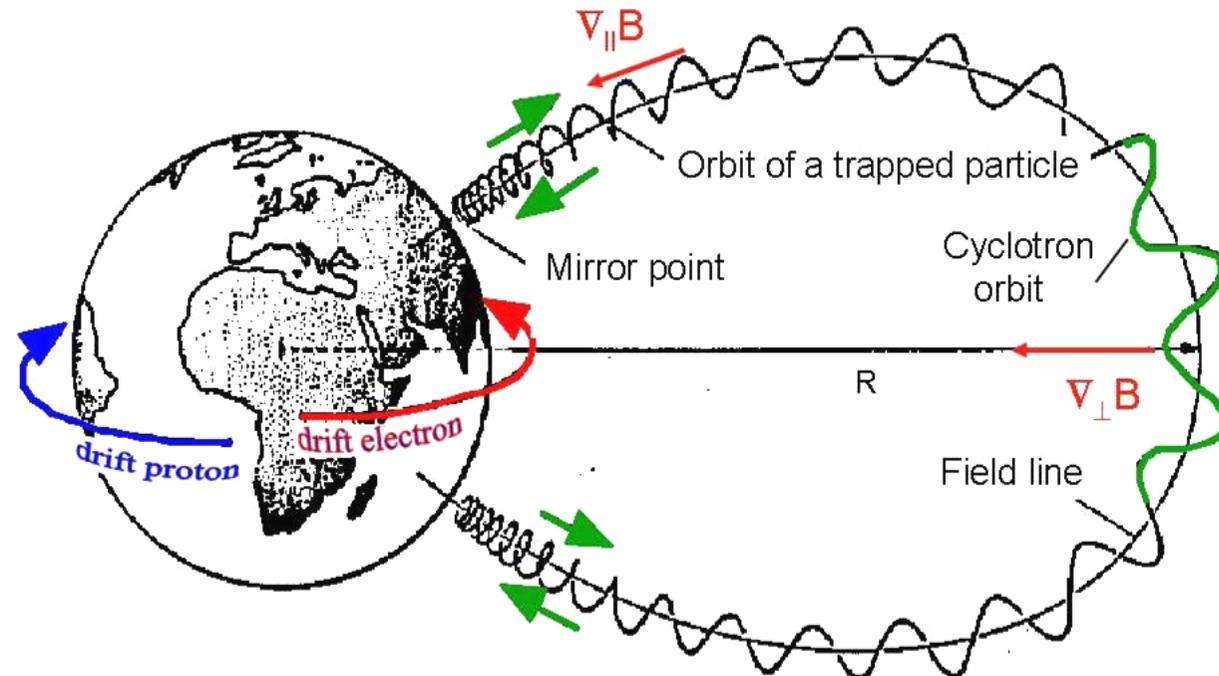


Organised and self-organised magnetic confinement



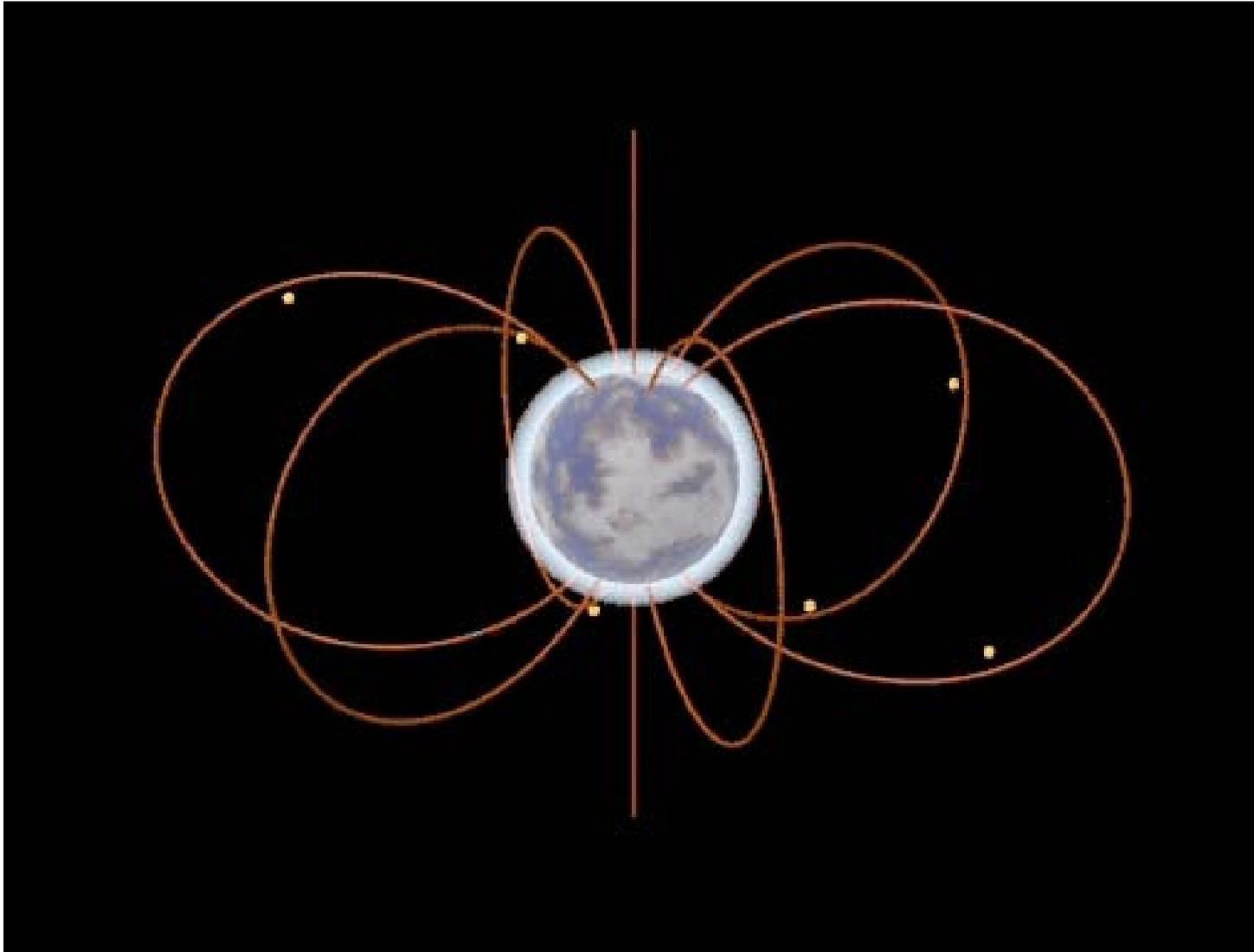
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EURATOM Association

Particle orbits in inhomogeneous field





Particle orbits in inhomogeneous field





Two toroidal confinement systems



Issues of toroidal confinement:

nested flux surfaces

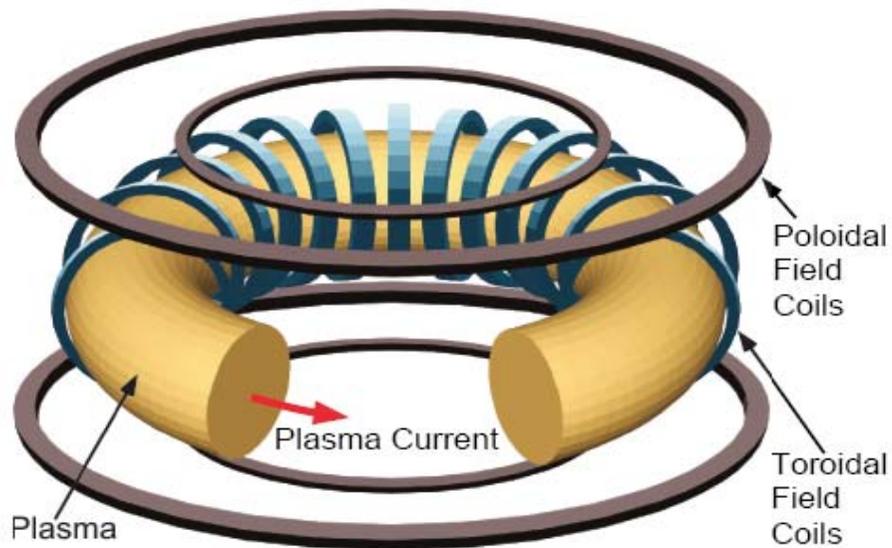
B_θ : rotational transform; confinement

B_ϕ : stability

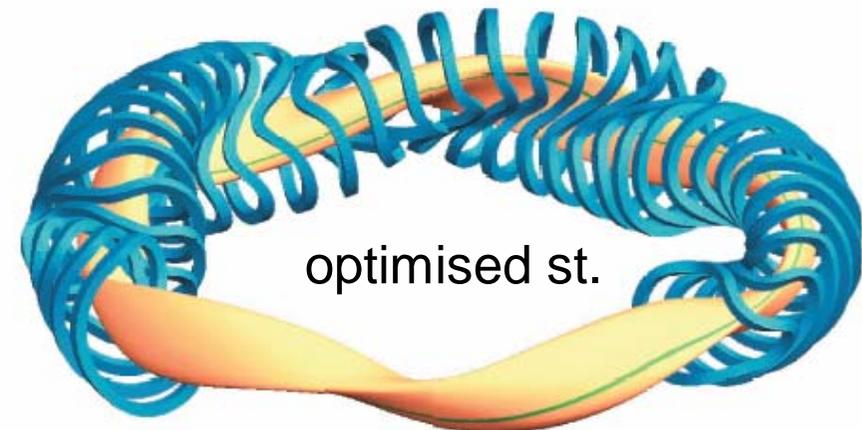
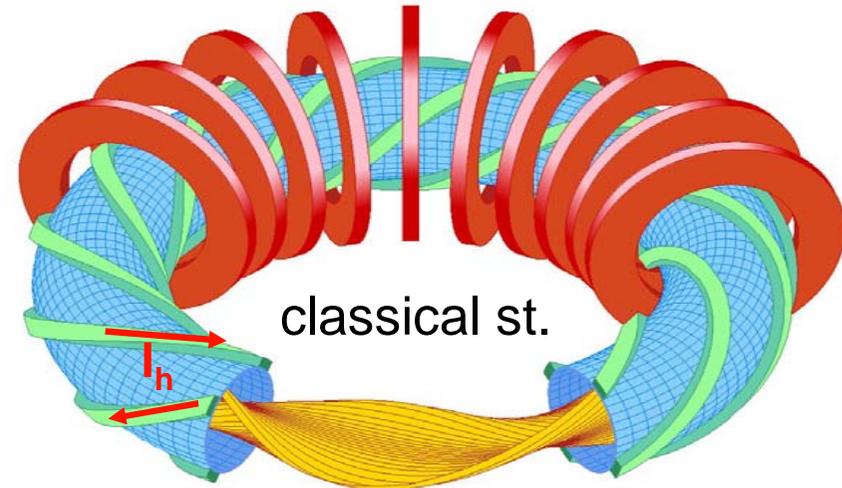
geometry: no endlosses

inhomogeneous field: $B \sim 1/R$

Tokamak



Stellarator



W7-X



Why stellarators ?

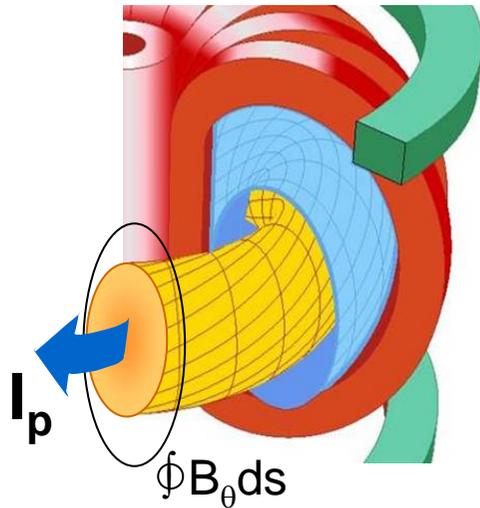


Tokamak and stellarator are complementary

- in the tokamak, the current flows in the plasma
- in the stellarator, it flows in the coils
- the tokamak is pulsed
- the stellarator is for steady-state operation
- the tokamak can develop detrimental instabilities
- the stellarator is not 2-dimensional



Tokamak

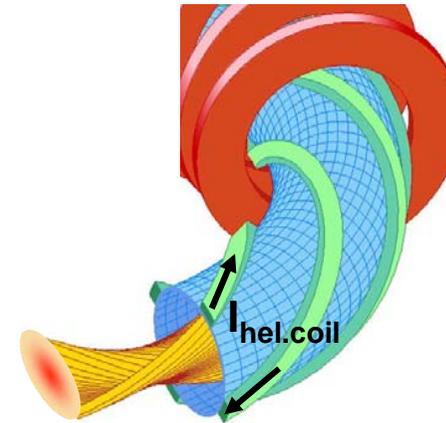


Ampere's law

Tokamak: $\text{curl } \mathbf{B} = \mu_0 \mathbf{j}$

Geometry: 2D: $\mathbf{B} = \mathbf{B}(\psi, \theta); \partial/\partial\phi = 0$

Stellarator



Stellarator: $\text{curl } \mathbf{B} = 0$

3D: $\mathbf{B} = \mathbf{B}(\psi, \phi, \theta)$

no continuous symmetry;
Instead: toroidal periodicity: N



Noether's theorem



Emmy Noether



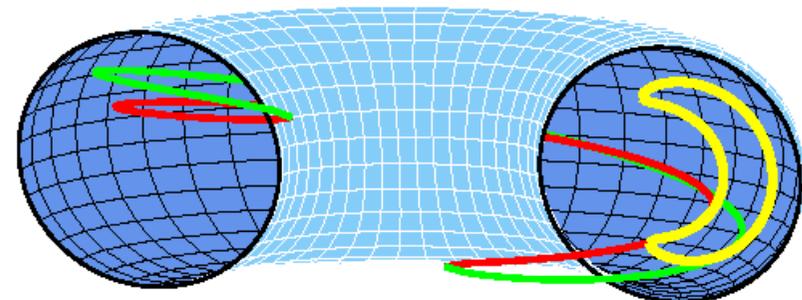
Theorem: in case of continuous symmetry an associated quantity is conserved

Here:
the canonical angular momentum p_ϕ

The consequence for 2D

Orbits are periodic and particles are confined

Banana particles

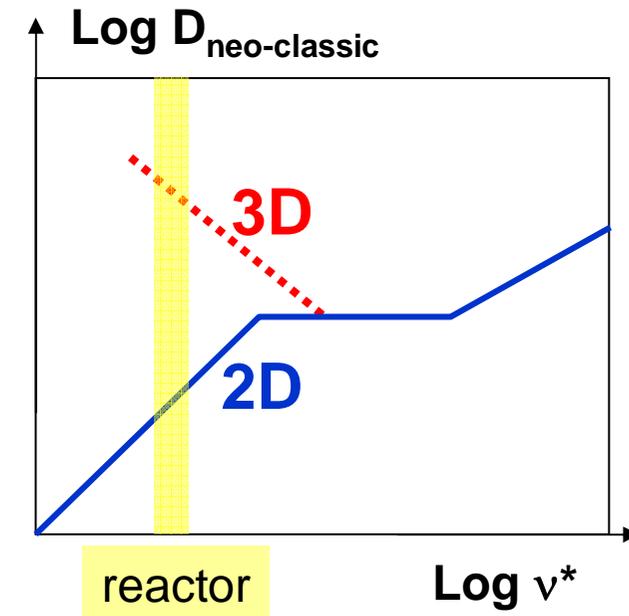
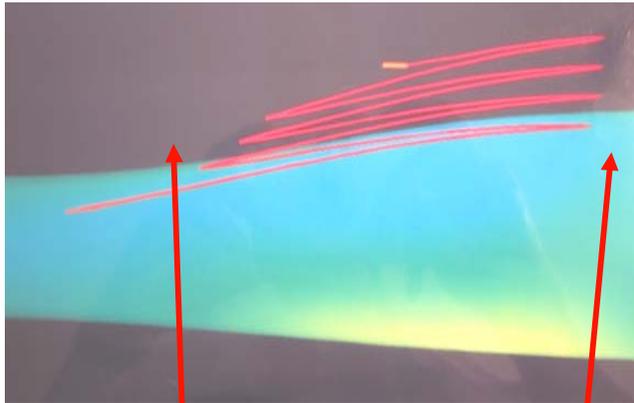




Consequence for 3D



helical ripple leads to collisionless thermal and energetic particle losses



Neo-classical fluxes have bad temperature scaling

**Stellarators
need optimisation**



Principle of stellarator optimisation

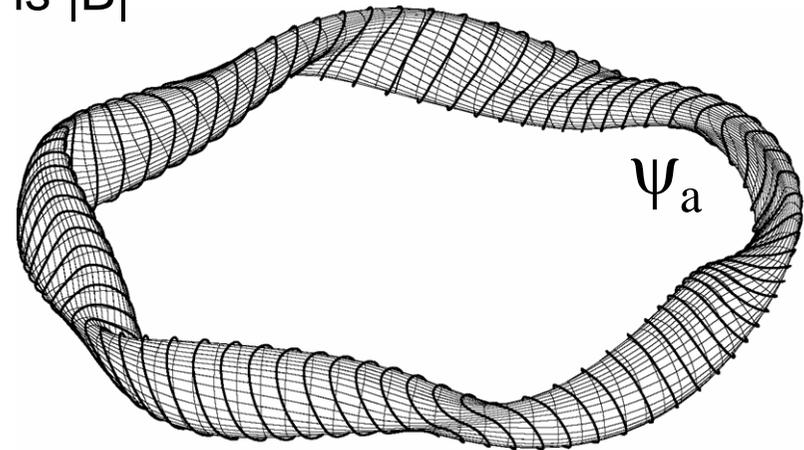
A. Boozer:

the relevant quantity for particle confinement is $|B|$
not $\mathbf{B} = (B_x, B_y, B_z)$

J. Nührenberg:

$|B|$ can be made 2D in (3D) helical systems
(expressed in magnetic coordinates)

⇒ quasi-symmetric systems



The plasma properties inside the volume ψ_a are determined by the geometry of ψ_a

Optimisation by variation of the shape of the flux surface geometry in a high-dimensional configuration space

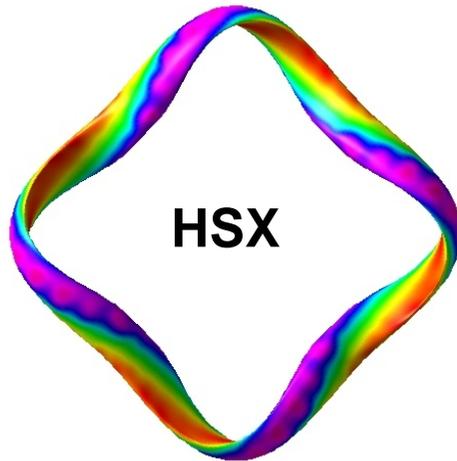
The optimisation procedure fixes geometrical parameters: $A, \kappa, \delta, N, \text{iota}, \dots$



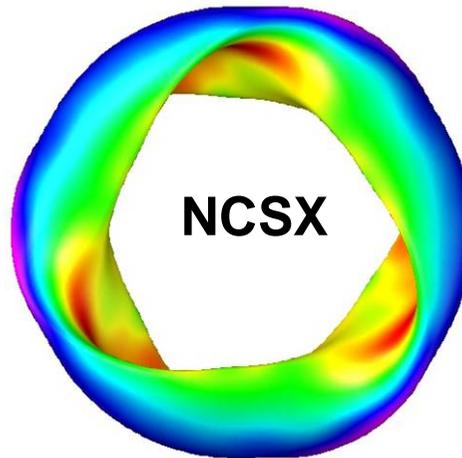
The family of quasi-symmetric systems



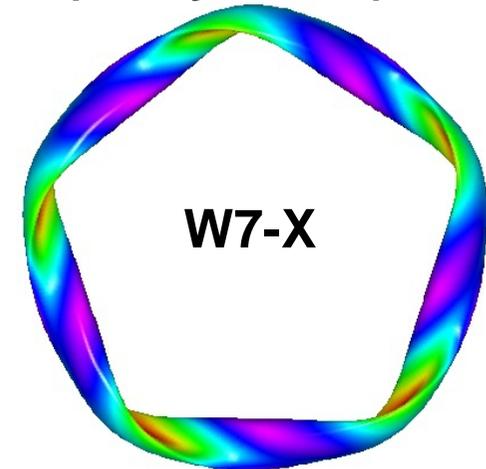
Quasi-helical



axial

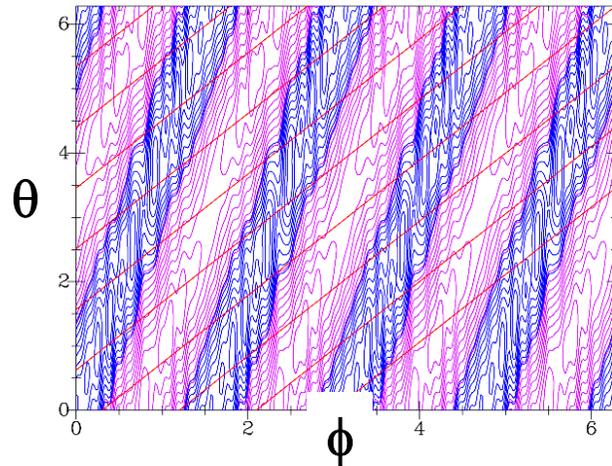


poloidal (isodynamic)

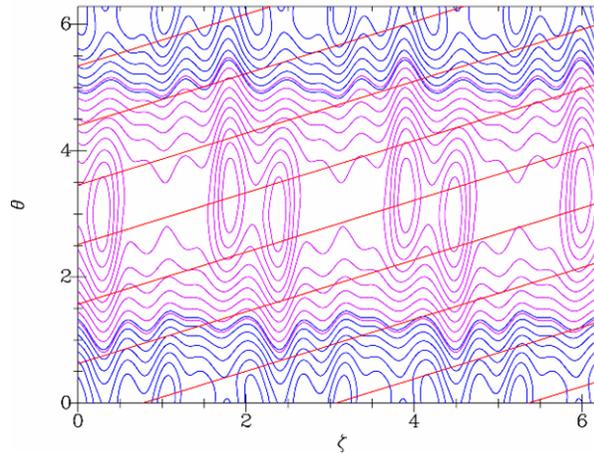


modB surfaces

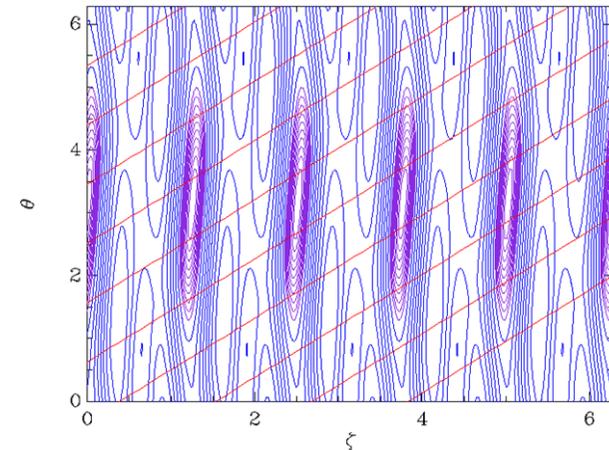
$|B|$ at $r/a = 0.20$ (blue: $B < 1T$, purple: $B > 1T$)



$|B|$ at $r/a = 0.20$ (blue: $B < 1T$, purple: $B > 1T$)



$|B|$ at $r/a = 0.50$ (blue: $B < 1T$, purple: $B > 1T$)





The physics behind optimisation



Principles of optimisation:

- reduction of trapped particle fraction => linked mirror concept
- reduction of radial particle drift
 - => strong elongation => lower curvature
 - => small variation of $|B|$ along \mathbf{B}



Small Pfirsch-Schlüter currents

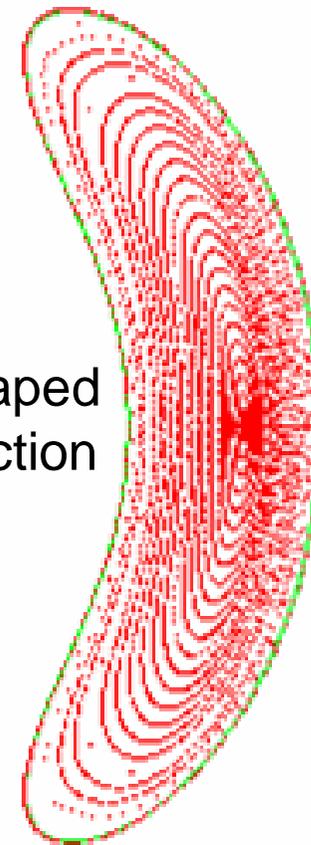
Small Pfirsch-Schlüter diffusion

Small Shafranov shift

Improved stability limit against Mercier and resistive interchange modes

Side conditions: Magnetic well

W7-X
Bean-shaped
cross-section





Ingredients for the optimisation (1)

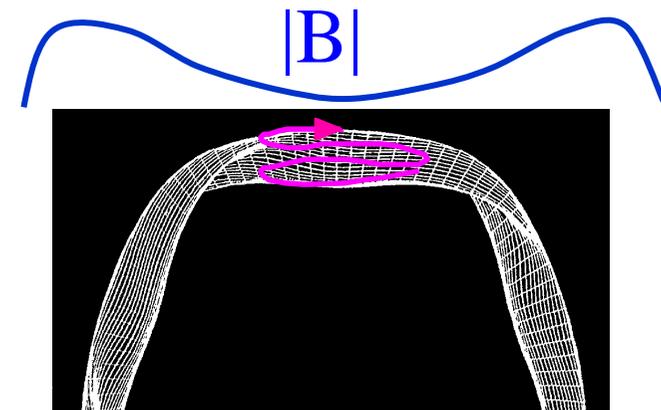
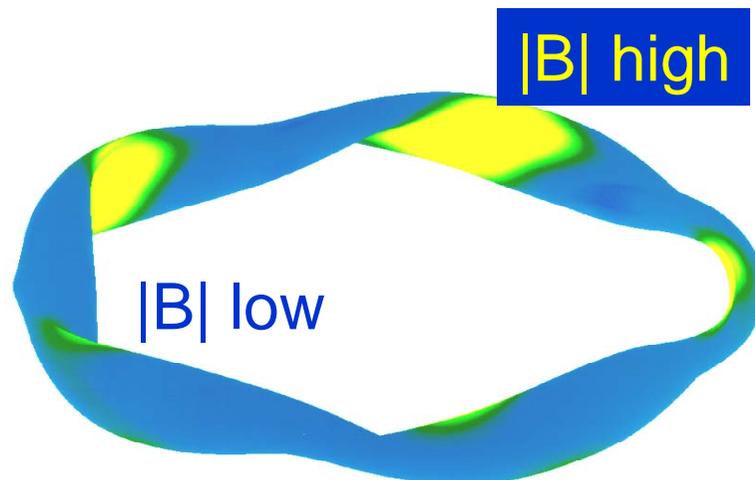


Geometry is toroidally periodic (e.g. $N=5$) with straight sectors between corners

At the corners (high curvature, high drifts): $|B|$ is increased \Rightarrow linked mirrors

Consequence:

the trapped particles are removed from the zones of large vertical drifts.

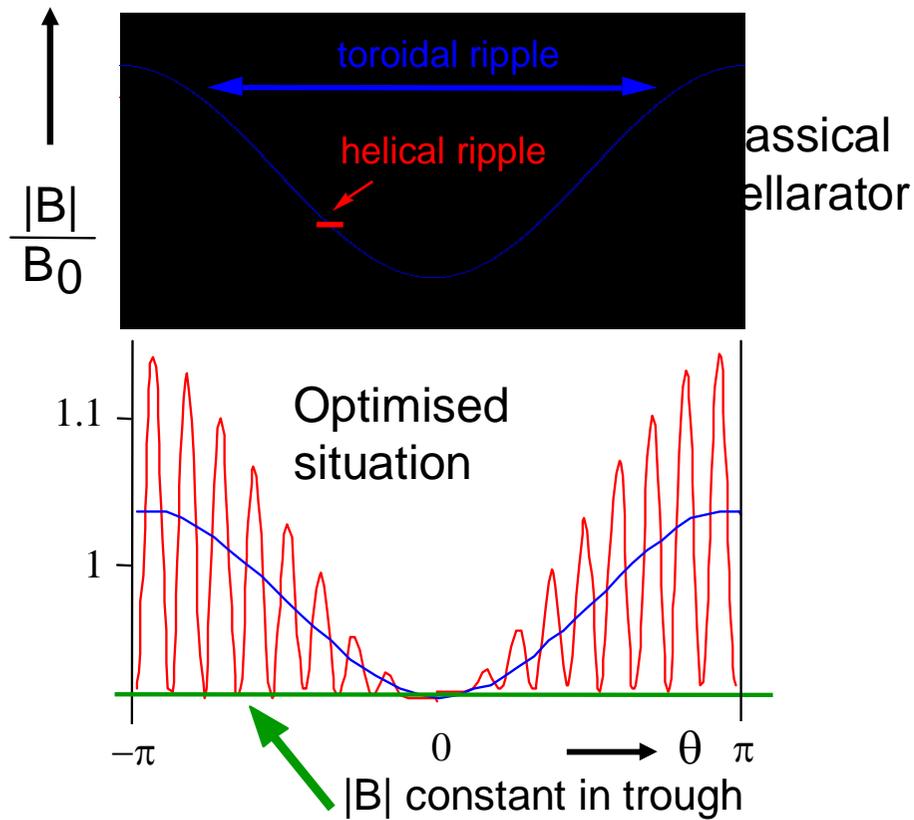




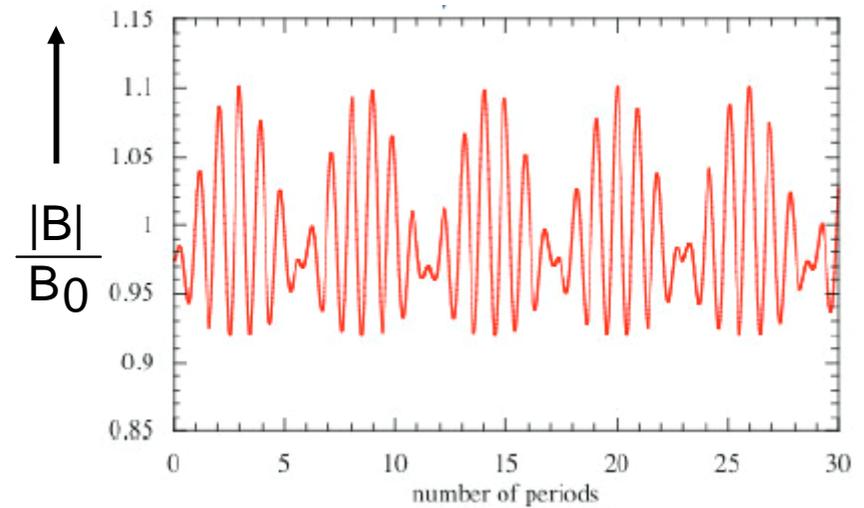
Ingredients for the optimisation (2)



Sigma optimisation



W7-X



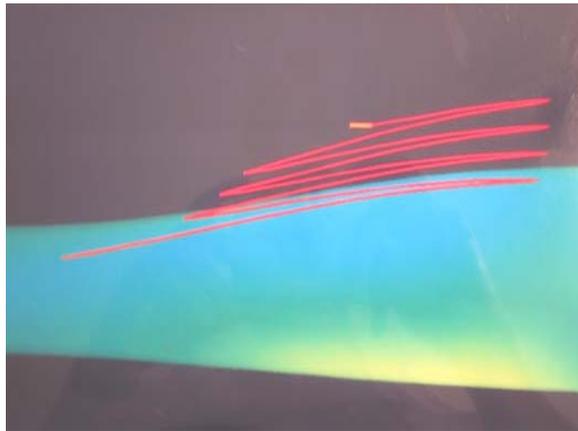


Achievements by optimisation

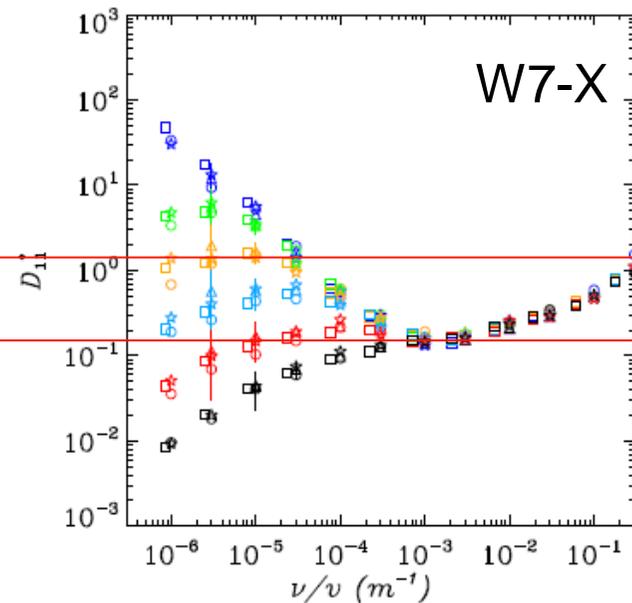
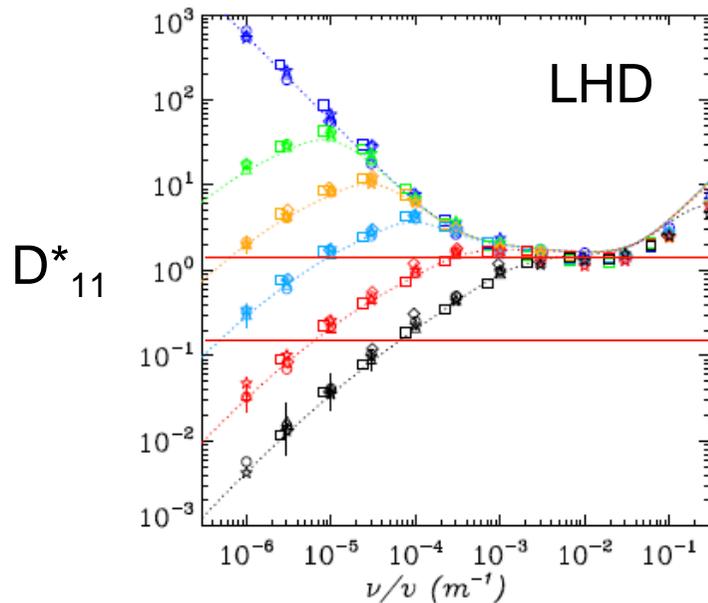
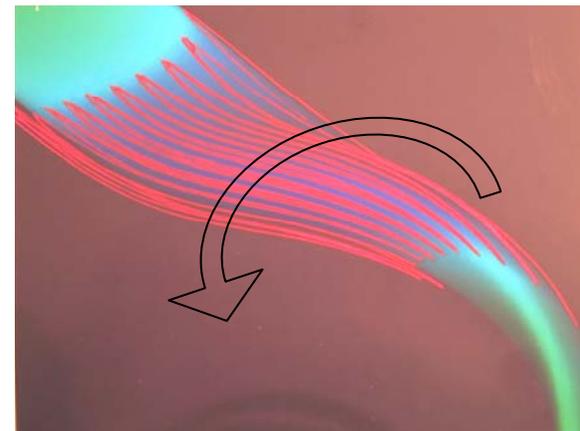


Drift of helically trapped particle

W7-AS (predecessor of W7-X)



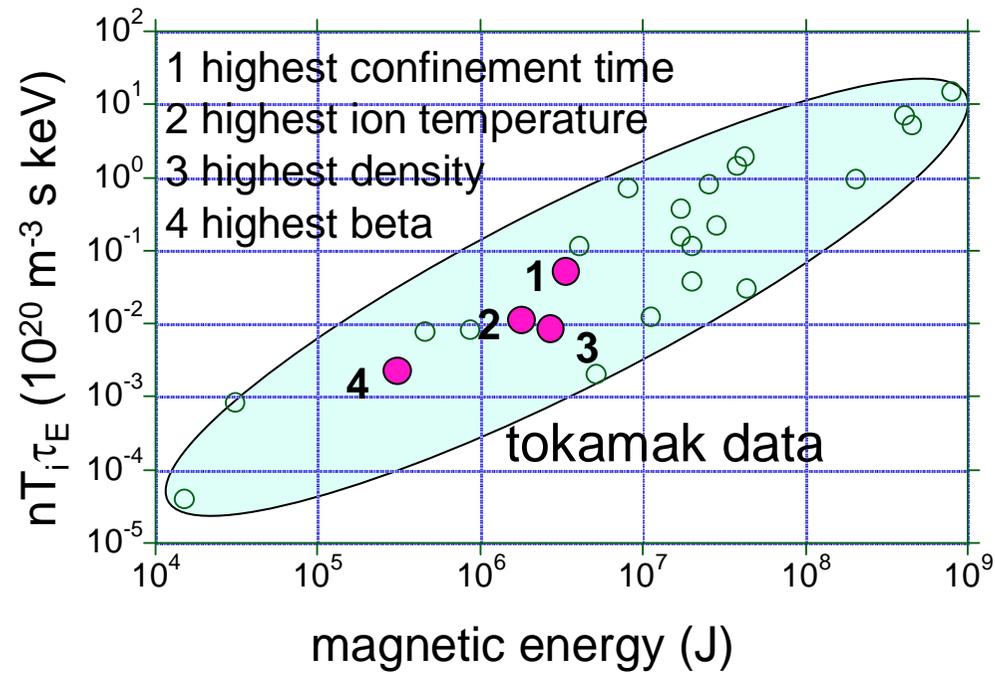
W7-X



Neo-classical diffusion

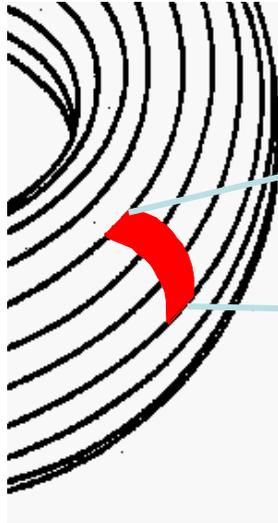


Comparison of tokamak and stellarator triple product

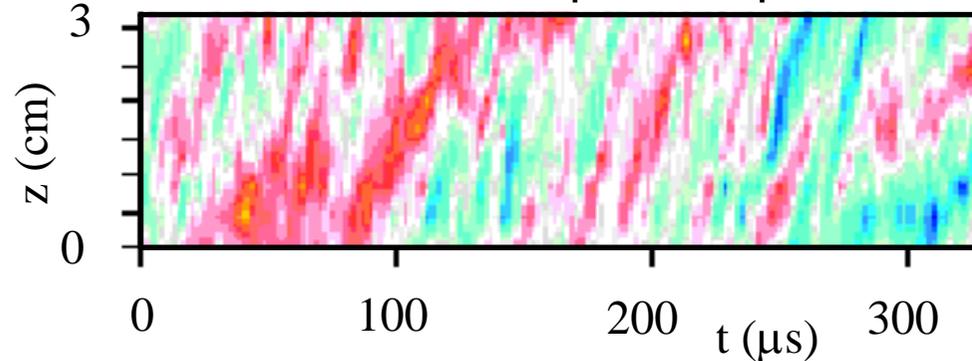




Turbulent transport

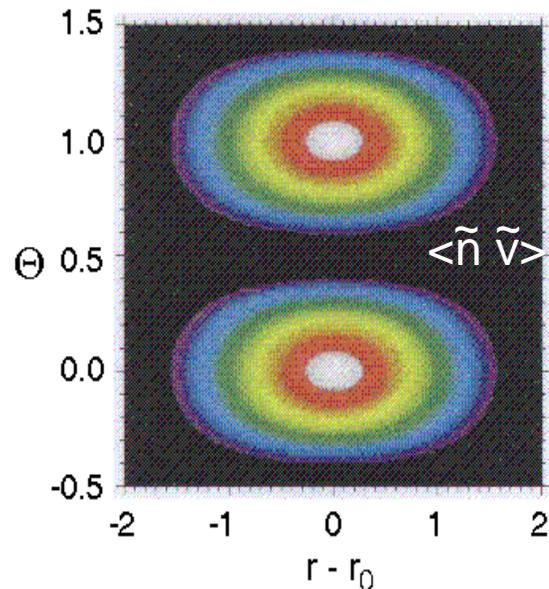


Fluctuations in plasma potential



Space scales:

Gradient length L_p : 1m,
perp. correlation length: $k_{\perp} \sim 1 \text{ cm}$
parallel correlation length: $k_{\parallel} \ll k_{\perp}$



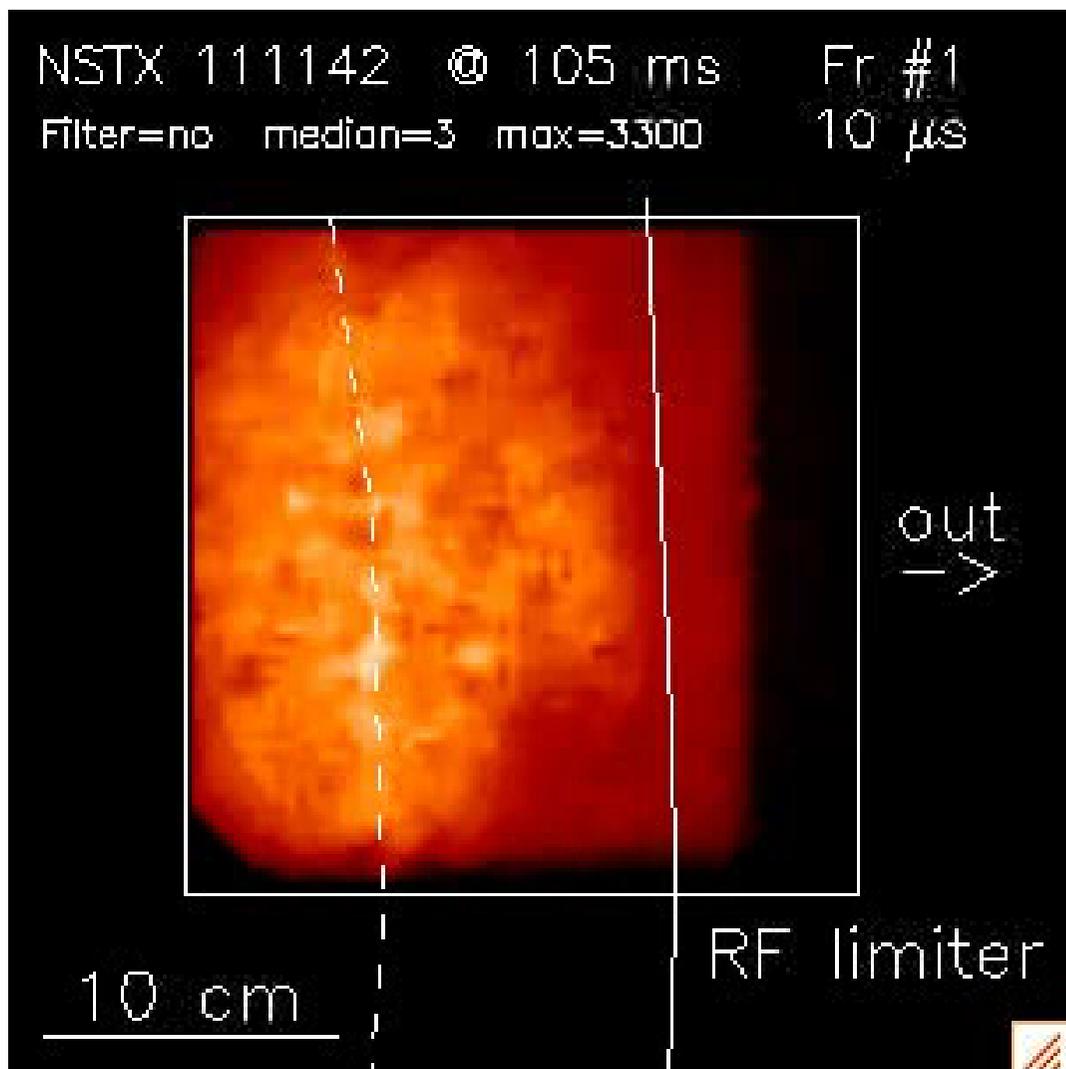
Time scales:

Drift frequency: $c_s/L_p = 10^6 \text{ s}^{-1}$

$$D_{turb} \approx \frac{\gamma}{k_{\perp}^2} \sim 1 \text{ m}^2/\text{s} \Rightarrow \tau_E \sim 1 \text{ s}$$



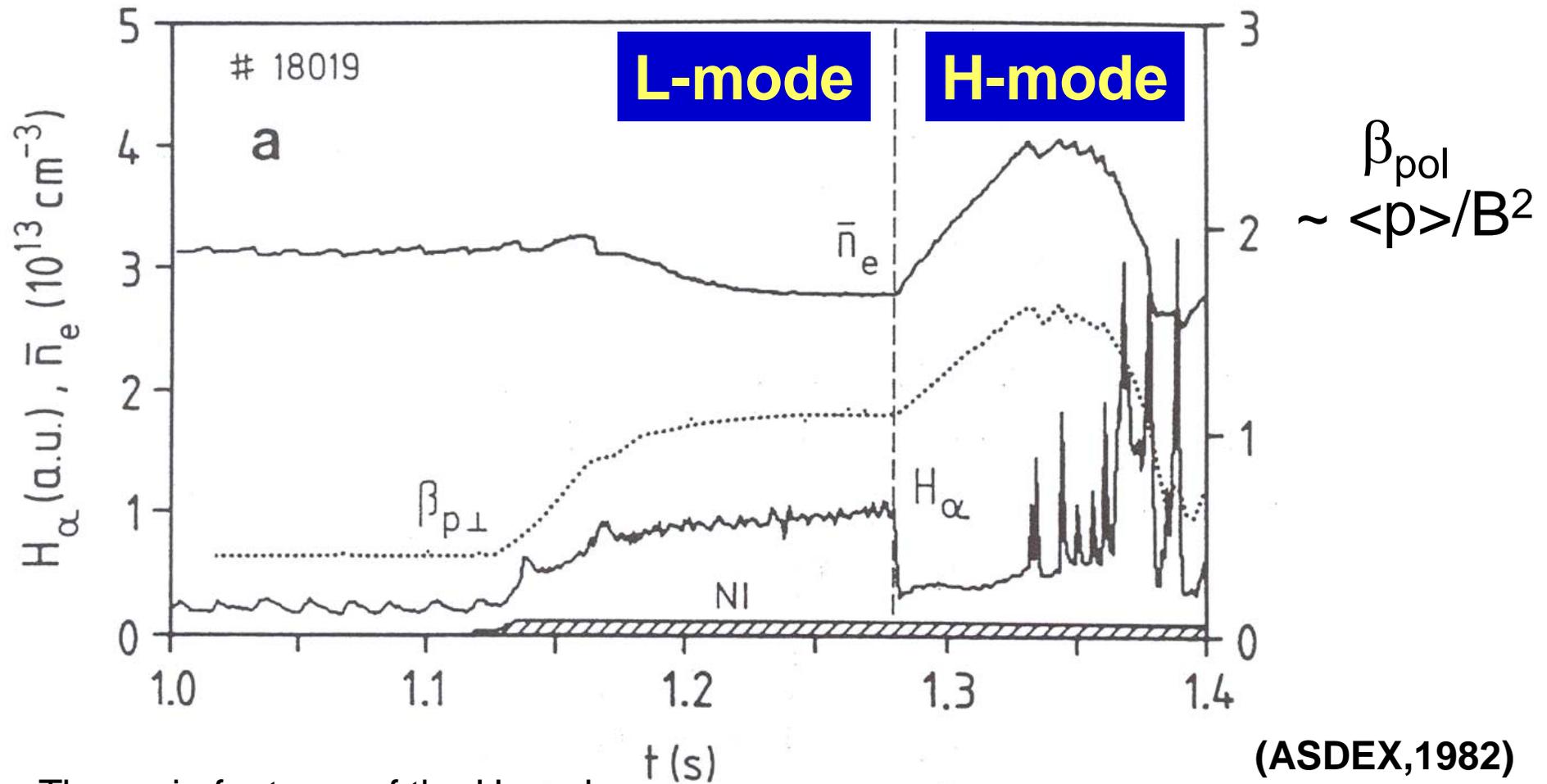
Movie of edge turbulence



S.J. Zweben *et al.*,
Phys. Plasmas **9** (2002) 1981



Self-organisation of toroidal fusion plasmas



The main features of the H-mode

- a spontaneous and distinct transition during the heating phase

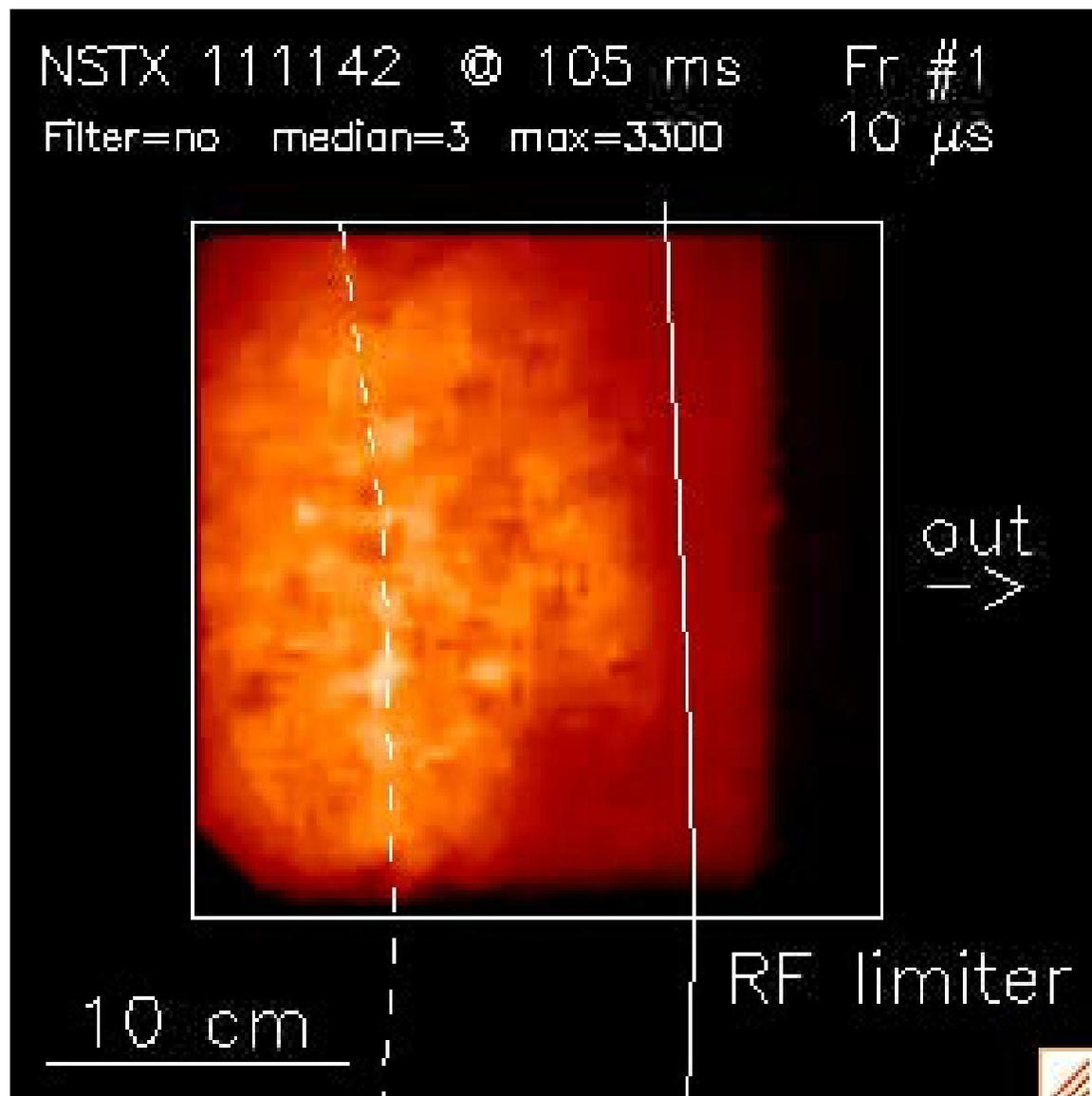
- both energy- and particle confinement time increase

- the tracer for the transition is the H_α -radiation

- new instabilities appear in the H-phase: ELMs, edge-localised modes



Macroscopically visible H-mode transition

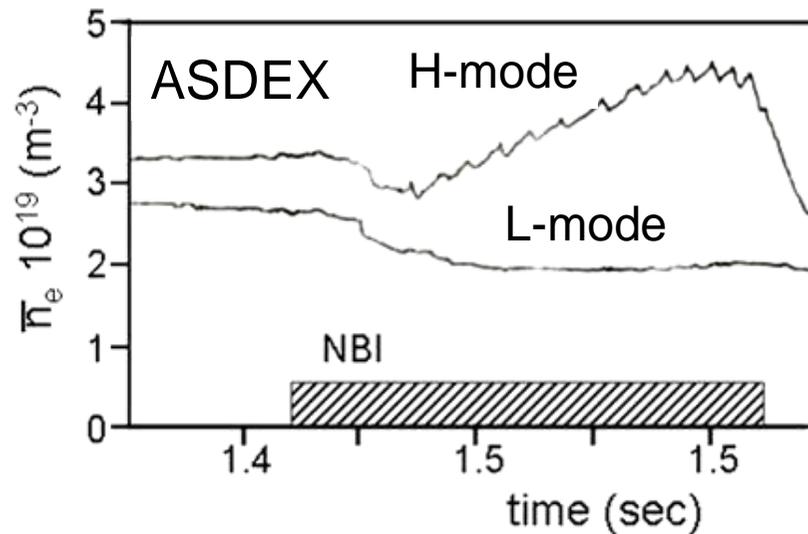




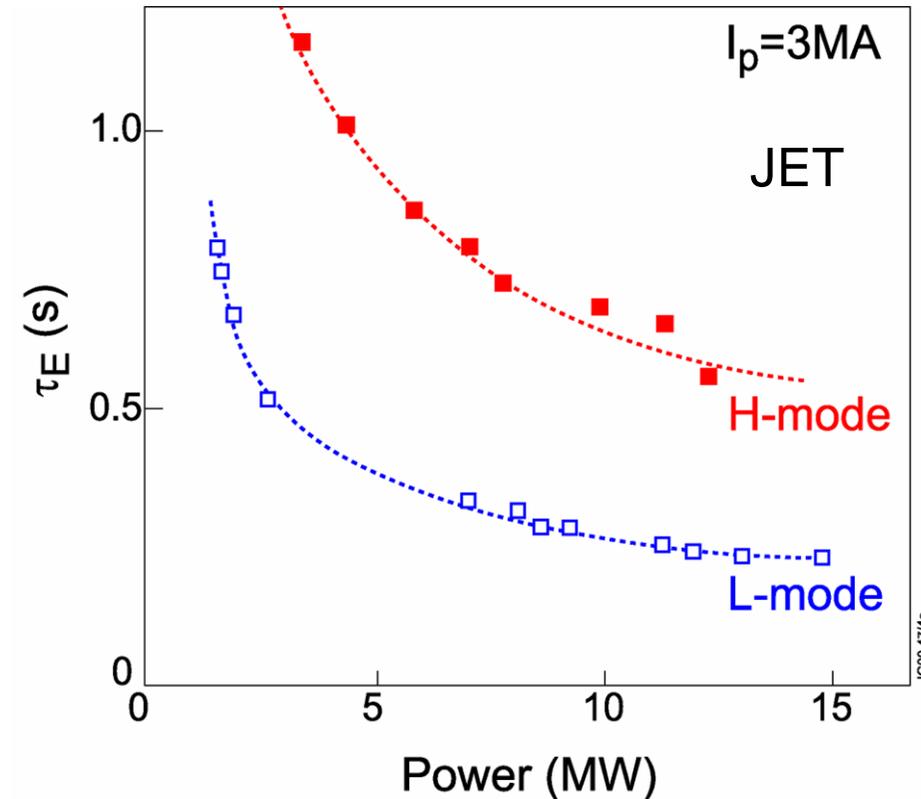
L- and H-mode branches



Particle confinement



Energy confinement



Two well separated branches
Space inbetween not accessible
(at given plasma setting)

$$\text{Def. } H = \tau_E^H / \tau_E^L$$



Benefit of improved confinement



The importance of improved confinement:

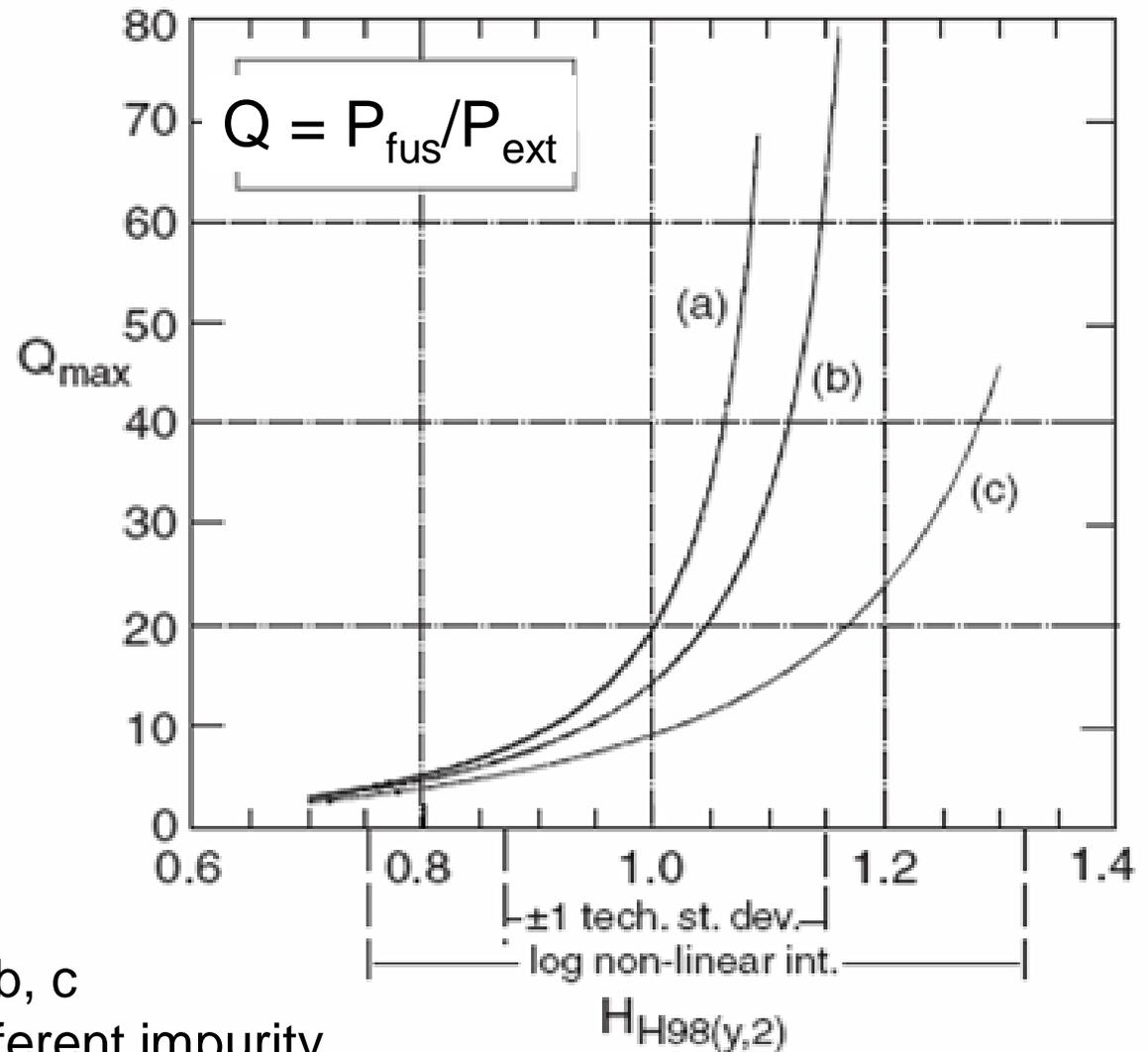
Improvement factor: $\tau_E \Rightarrow H\tau_E$

Ignition:

$$\frac{\langle p \rangle \tau_E}{a^2 B_t^2} \sim H^2$$

Triple product:

$$nT\tau_E \propto H^2$$



a, b, c
different impurity
confinement

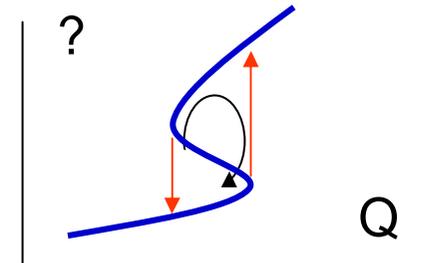
V. Mukhovatov



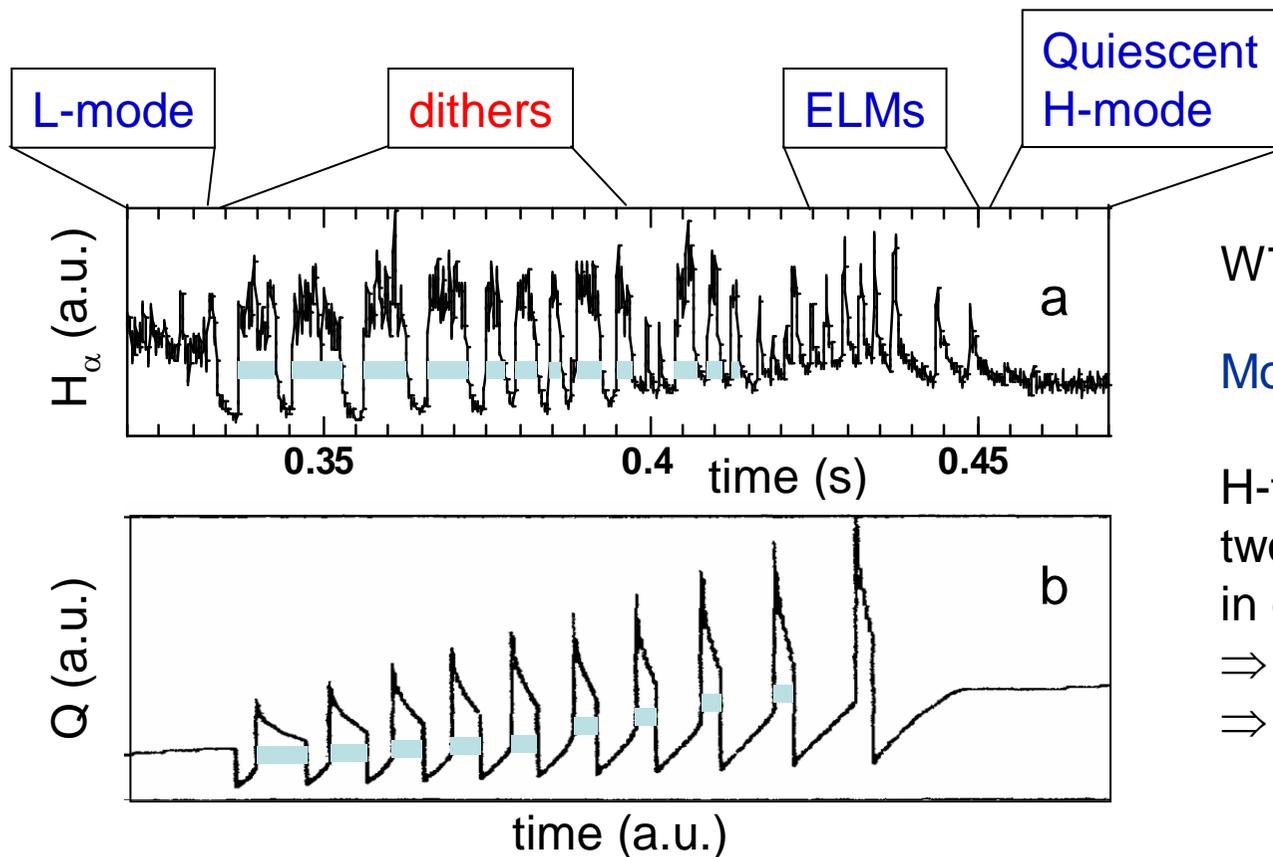
The H-mode as bifurcation phenomenon



Theory: Development of bifurcation models



A feature of bifurcations: Limit-cycle oscillations (dithers)



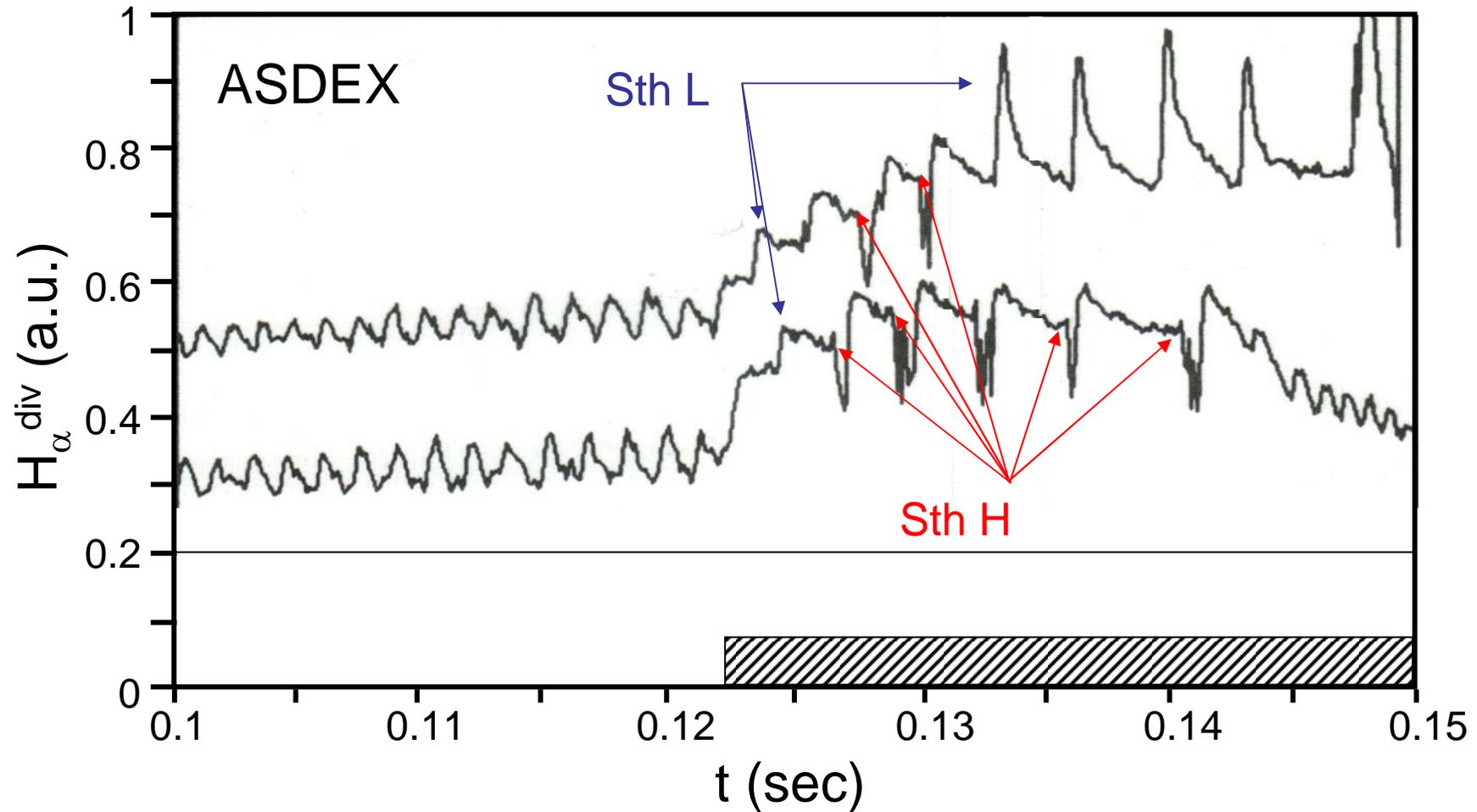
W7-AS

Model by H. Zohm:

H-transition initiates two processes going in opposite direction
⇒ deeper into H
⇒ back to L

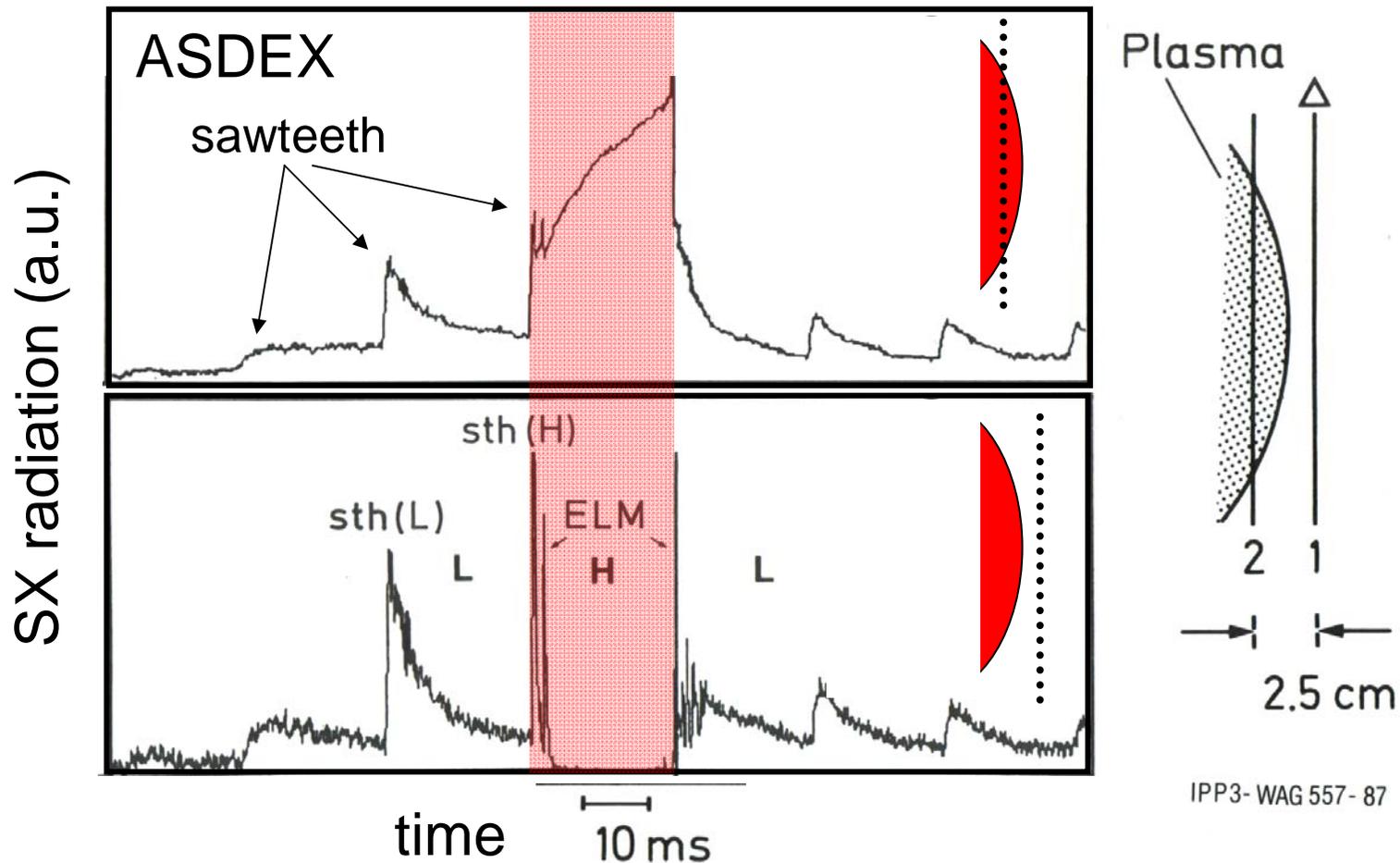


Sawteeth trigger short H-phases





Edge and SOL probed with sawteeth after NBI switch-on

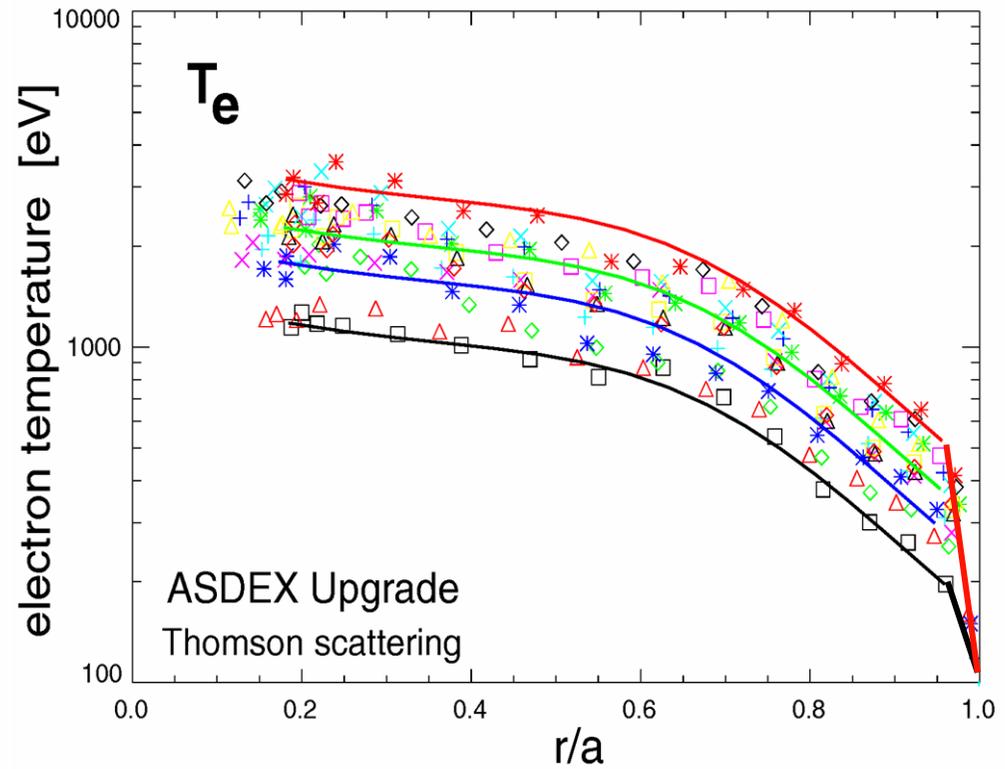
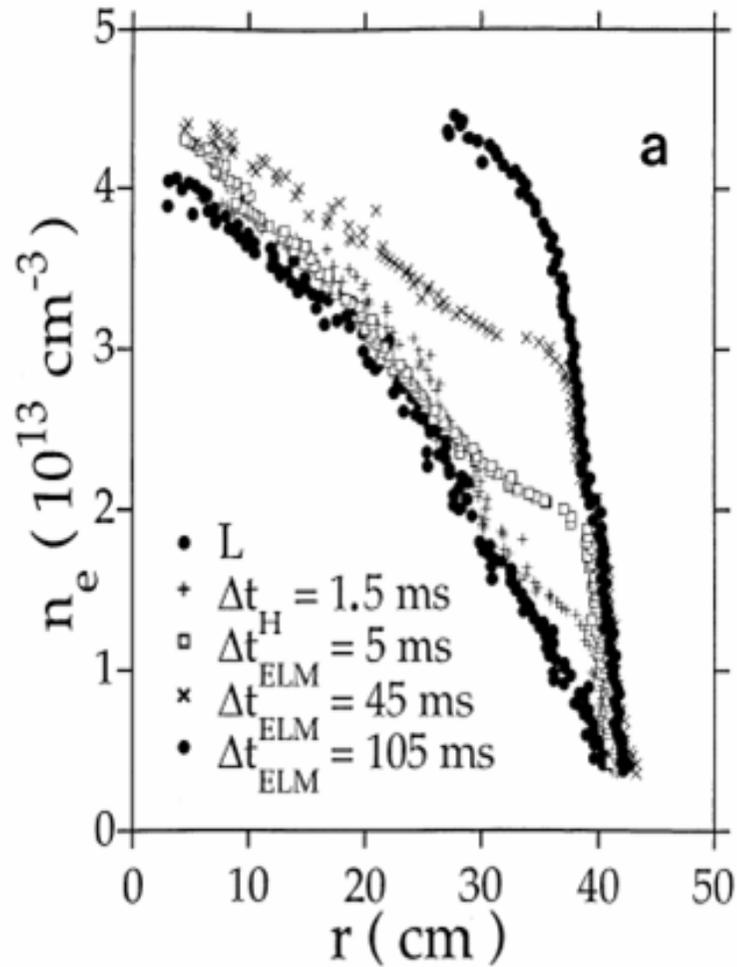




Edge Transport Barrier in density and temperature

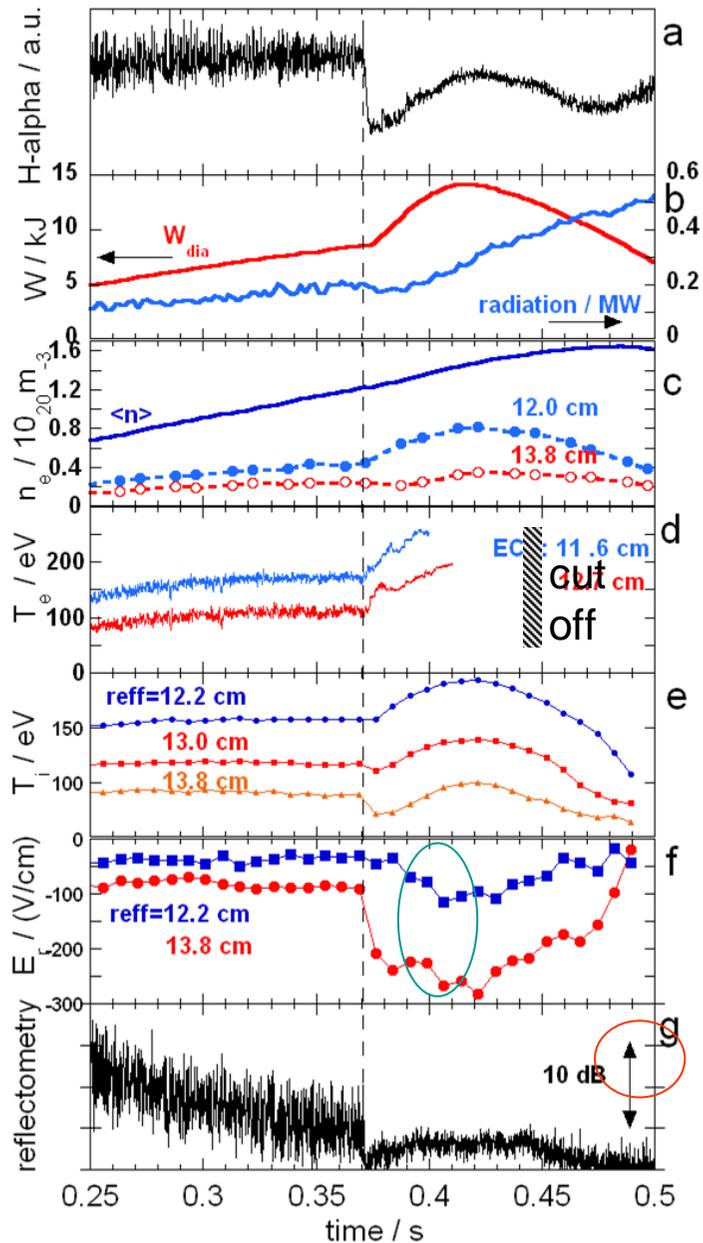


Edge transport barrier





Summary of H-mode observation



Results from W7-AS stellarator:

Implication:

H-mode is a ubiquitous operational regime in toroidal confinement



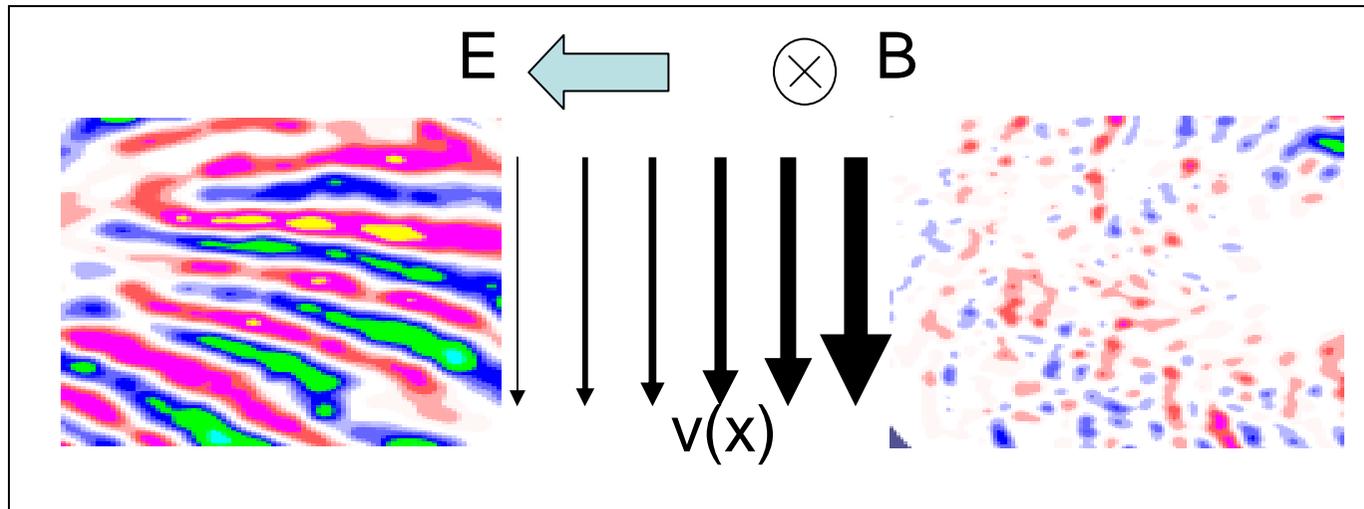
The plasma self-organizes its turbulence level



1. Step: sheared flow decorrelates turbulence

Biglary, Diamond, Terry

Bo Lehnert (1966)



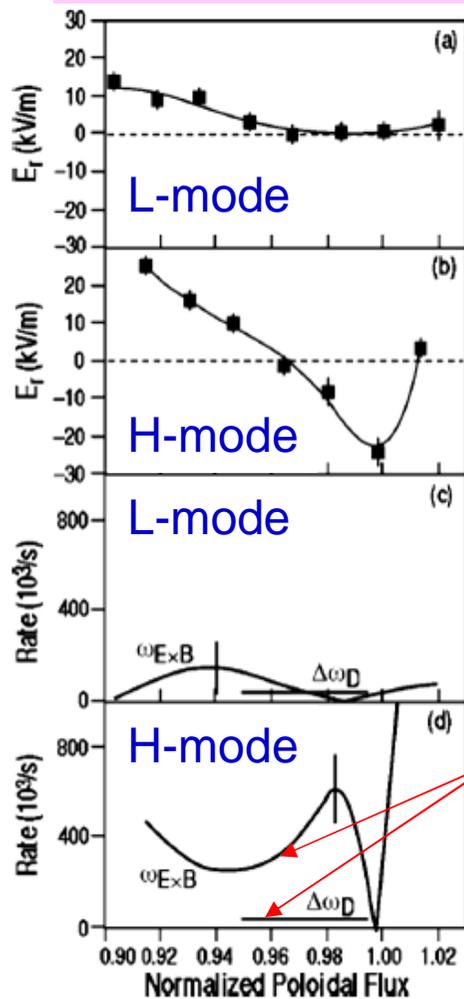


Shear flow decorrelation of turbulence

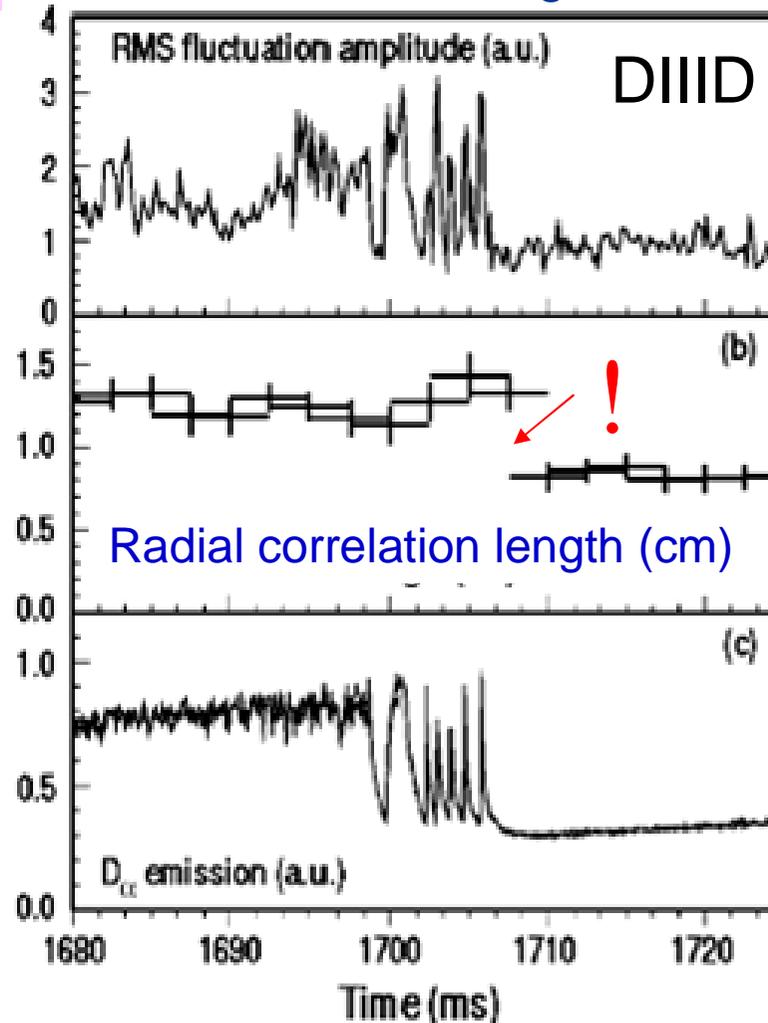


Conditions for
flow-decorrelation

$$\omega_{E \times B} > \gamma_{lin} (\Delta \omega_D)$$



Reduction of radial
correlation length



Condition:
sheared flow; ∇E_r

Probe (Textor):

$$\nabla |E_{r,crit}| = 50-80 \text{ V/cm}^2$$

DIII-D:

$$\nabla |E_{r,crit}| = 50-100 \text{ V/cm}^2$$

W7-AS:

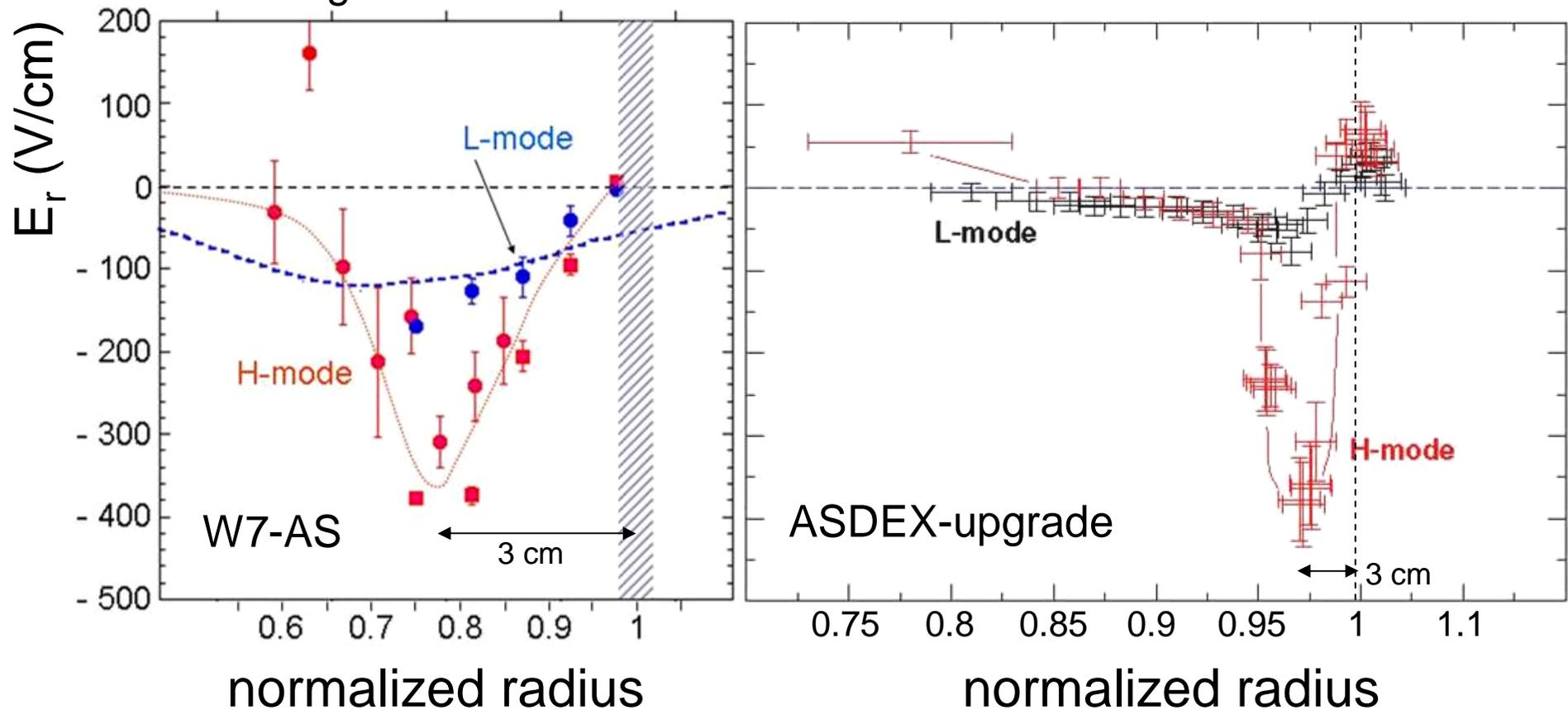
$$\nabla |E_{r,crit}| \sim 90 \text{ V/cm}^2$$



Origin of the sheared flow: $E_r(r)$



Role of the edge radial electric field



Generic feature of the H-mode: development of an E_r -well inside separatrix

Radial extent of well independent of machine size

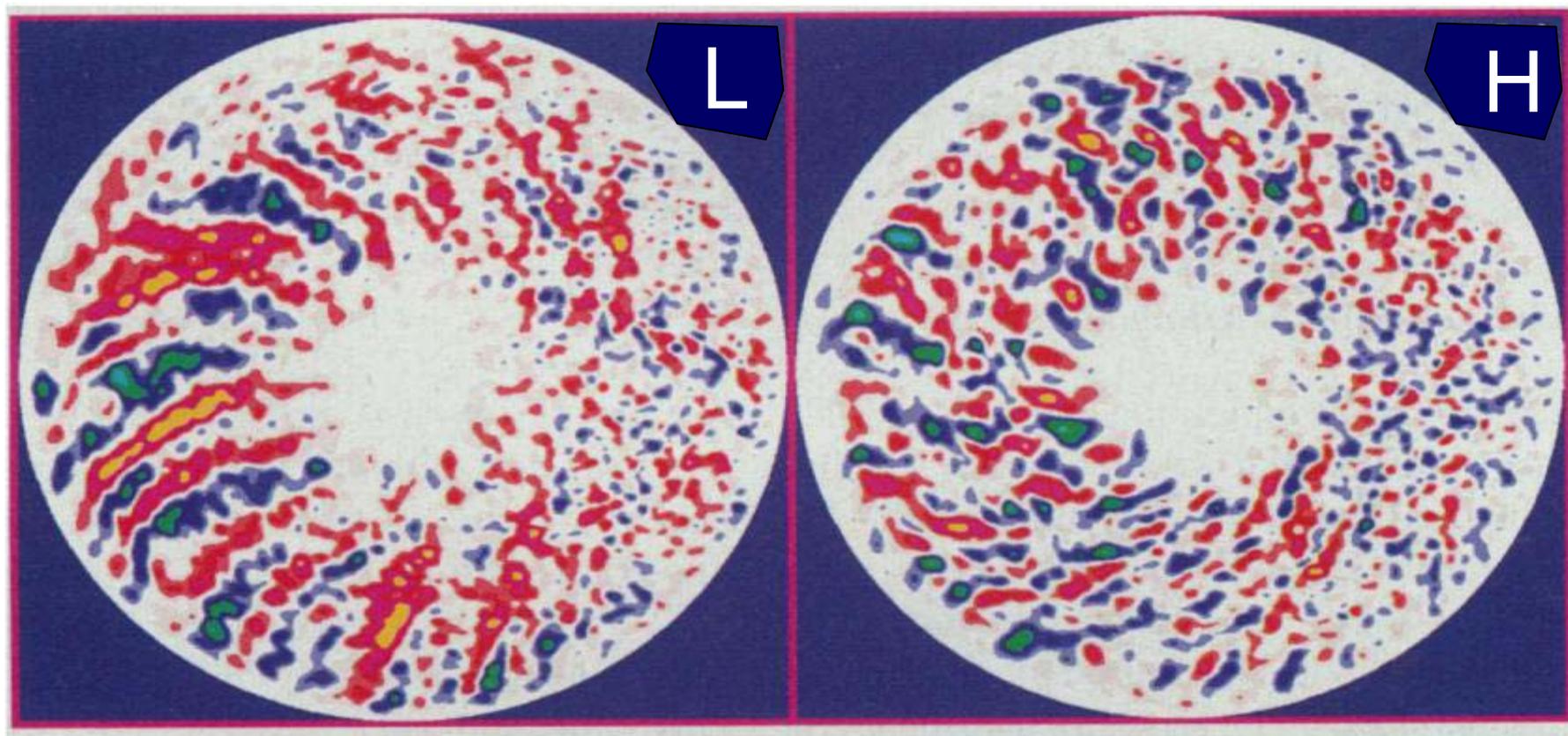
In stellarators: E_r -well in the L-phase already quite deep $\Rightarrow P_{th}^{STELL} < P_{th}^{TOK}$



Modelling of shear-flow decorrelation

IPP

Gyrokinetic particle simulation of plasma microturbulence



→ ∇B

Z. Lin et al., Science



The Origin of E_r at the edge



2D:

Fluxes, transport coefficients are intrinsically ambi-polar and do not explicitly depend on E_r

$\langle j_r \rangle = 0$, independent of E_r

3D:

$\langle j_r \rangle = 0$, ensured by $\Gamma_e = \Gamma_i$: enforced ambi-polarity

$$\Gamma = -D_1(E_r)n \left\{ \frac{1}{n} \frac{\partial n}{\partial r} - q \frac{E_r}{T} + \frac{D_{12}}{D_{11}} \frac{1}{T} \frac{\partial T}{\partial r} \right\}$$

$$E_r = \nabla p_i / en + (D_{12}/D_{11} - 1) \nabla T_i$$

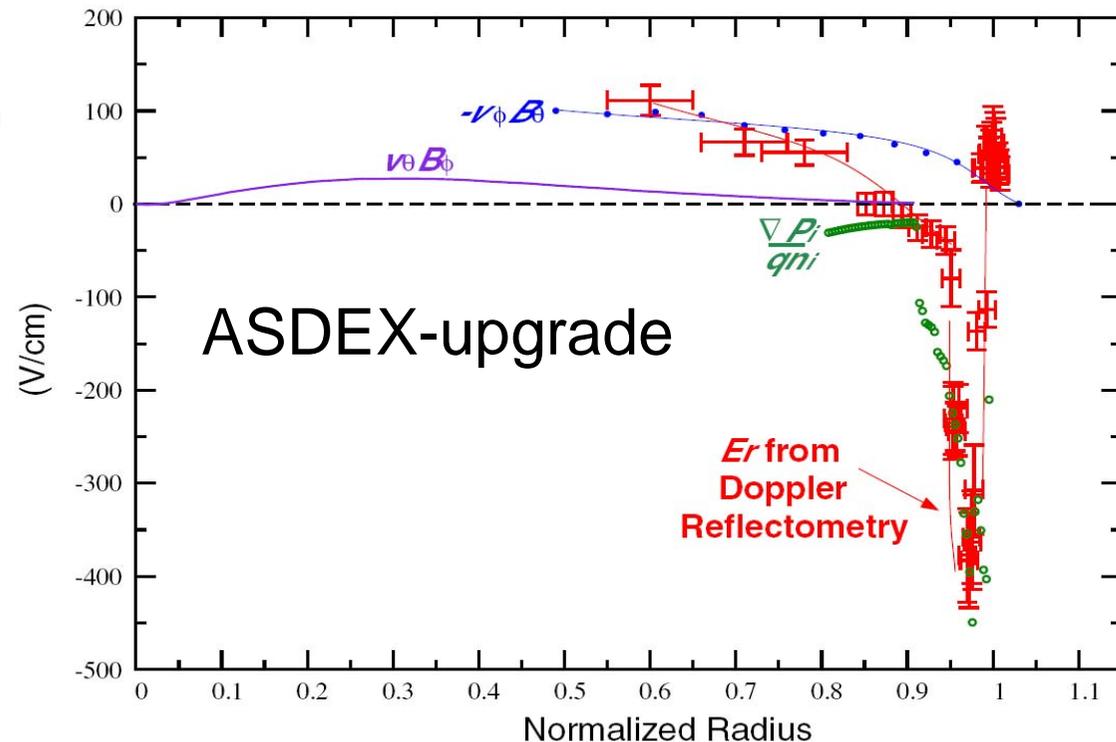


The composition of E_r



Radial force balance: $E_r = \nabla p_i / en_e - v_\theta B_\phi + v_\phi B_\theta$

Tokamak: 2D

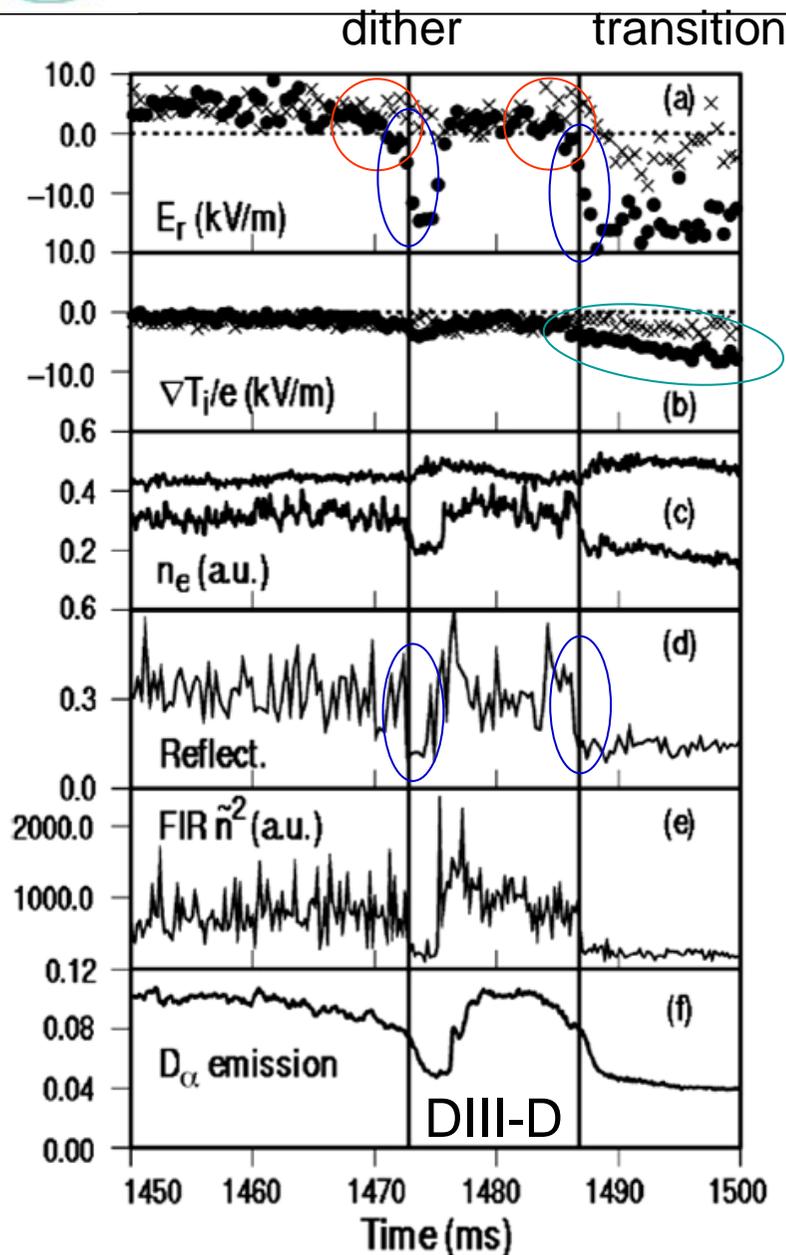


Turbulence \downarrow \Rightarrow pressure gradient \uparrow \Rightarrow flow increases \uparrow \Rightarrow turbulence \downarrow

∇p_i plays an important role In a fully developed H-mode:
it stabilises the mode



Temporal characteristics of $L \Rightarrow H$



There is a pre-phase

Jump of E_r at the $L \Rightarrow H$ transition

$$(\tau \ll \tau_E)$$

W7-X, JFT-2M: $t \sim 12 \mu\text{s}$

T_i changes slowly

∇p_i cannot be the transition trigger

Short timescale indicates:

Transition trigger related to $v_\theta B_\phi$

Turbulence level drops jointly with E_r



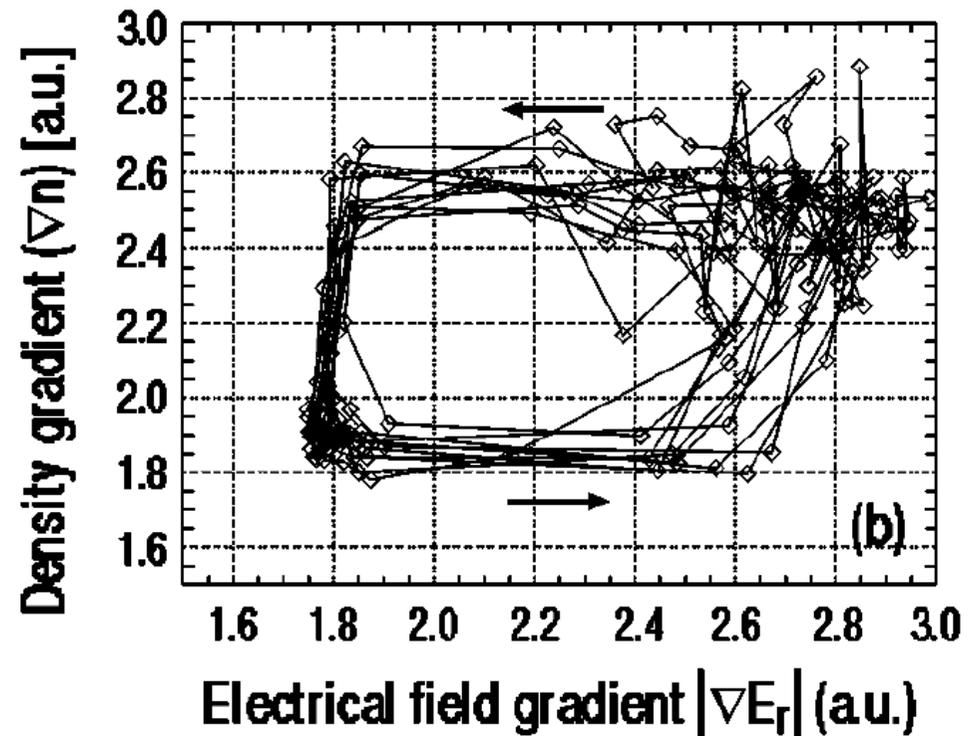
Causality between E_r and ∇p_i



TEXTOR: H-mode induced by polarisation probe

E_r is oscillating

n_e ($\text{grad} p_i$) also oscillates



Analysis done by K.H. Burrell, Phys. Plasmas

Causality: ∇E_r leads n_e by about 5 ms



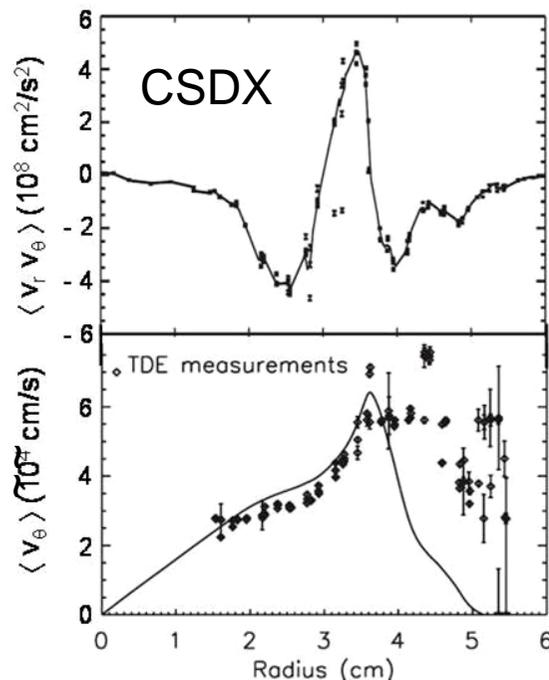
2nd step: Turbulence produces flow



Turbulence => Reynoldsstress ($\langle \tilde{v}_r \tilde{v}_\theta \rangle$) => flow => decorrelation of turbulence

$$\text{Poloidal force balance: } 0 = j_r B / n_i - m_i \mu_\theta v_{\theta i} + m_i \vartheta / \vartheta r (\langle \tilde{v}_r \tilde{v}_{\theta i} \rangle)$$

Reynolds stress
leads to steady-state flow



linear device!

Understanding parts of the H-mode

Self-induced flows from the turbulence field regulates the turbulence level.

Mechanisms:

Reynolds stress

spectral transport from small to large scales
flows, zonal flows, GAMS

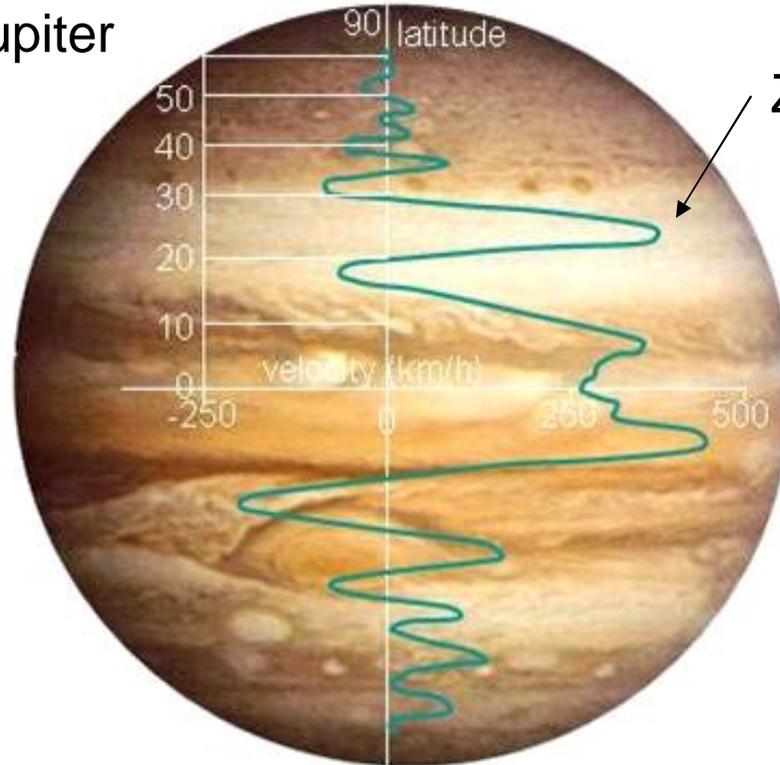
sheared flow reduces turbulence

∇p_i rises, deepens E_r well; stabilises H-mode

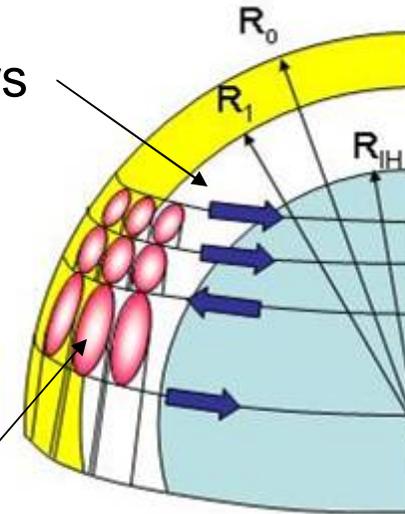


Short detour to the planets

Jupiter



Zonal flows



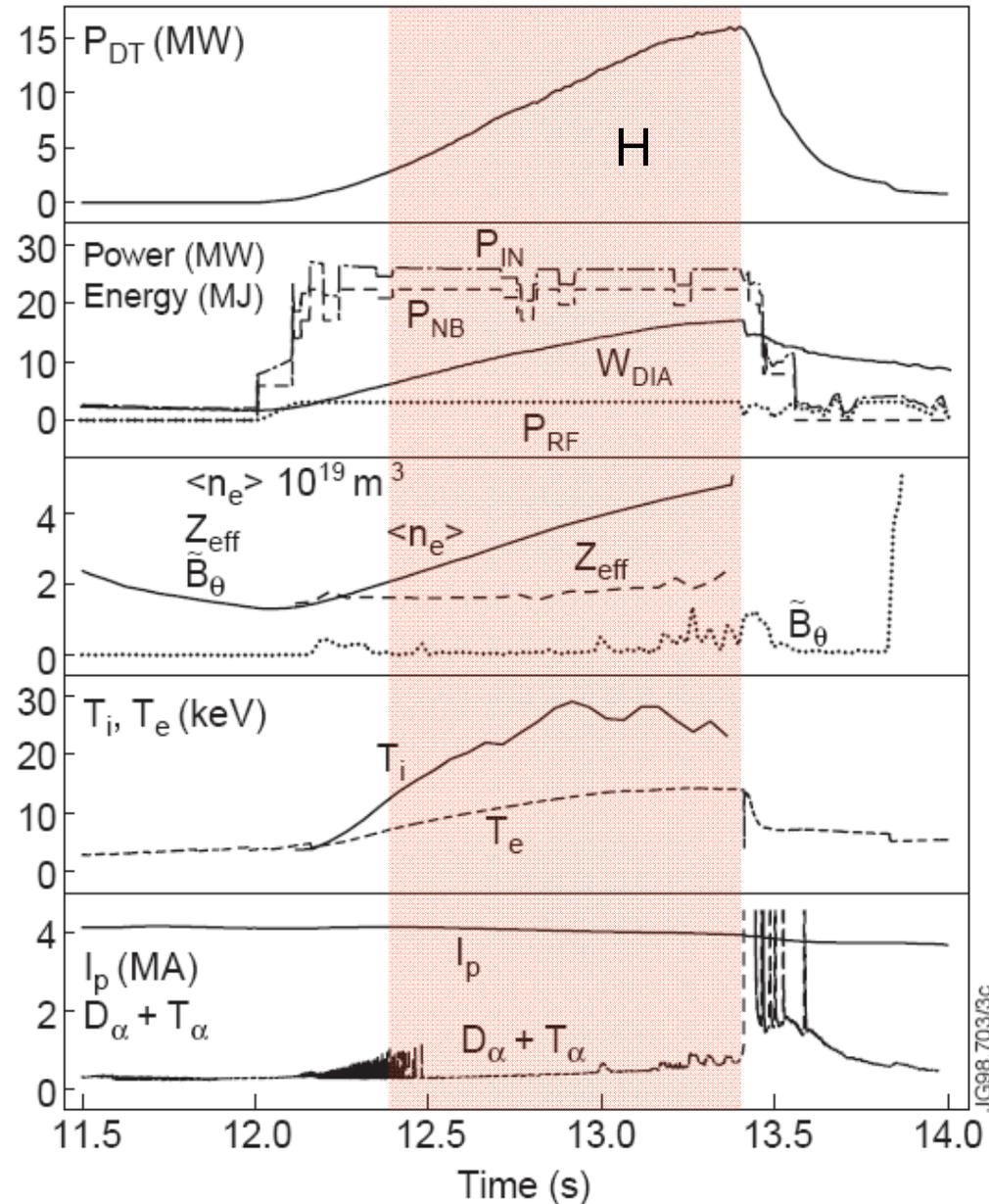
convective cells drive ZFs via Reynolds stress

Formation of large-scale flows from turbulence via RS
in laboratory experiments,
the sonic wind in gases,
meandering flows in oceans, Jet stream, in the ionosphere
e.g. Rossby waves (Coriolis force instead of Lorentz force)
in the sun



Achievements in the H-mode

The 16.1 MW DT discharge of JET





High-performance discharges: Tokamak



I_p (x10) MA
 P_{NBI} MW

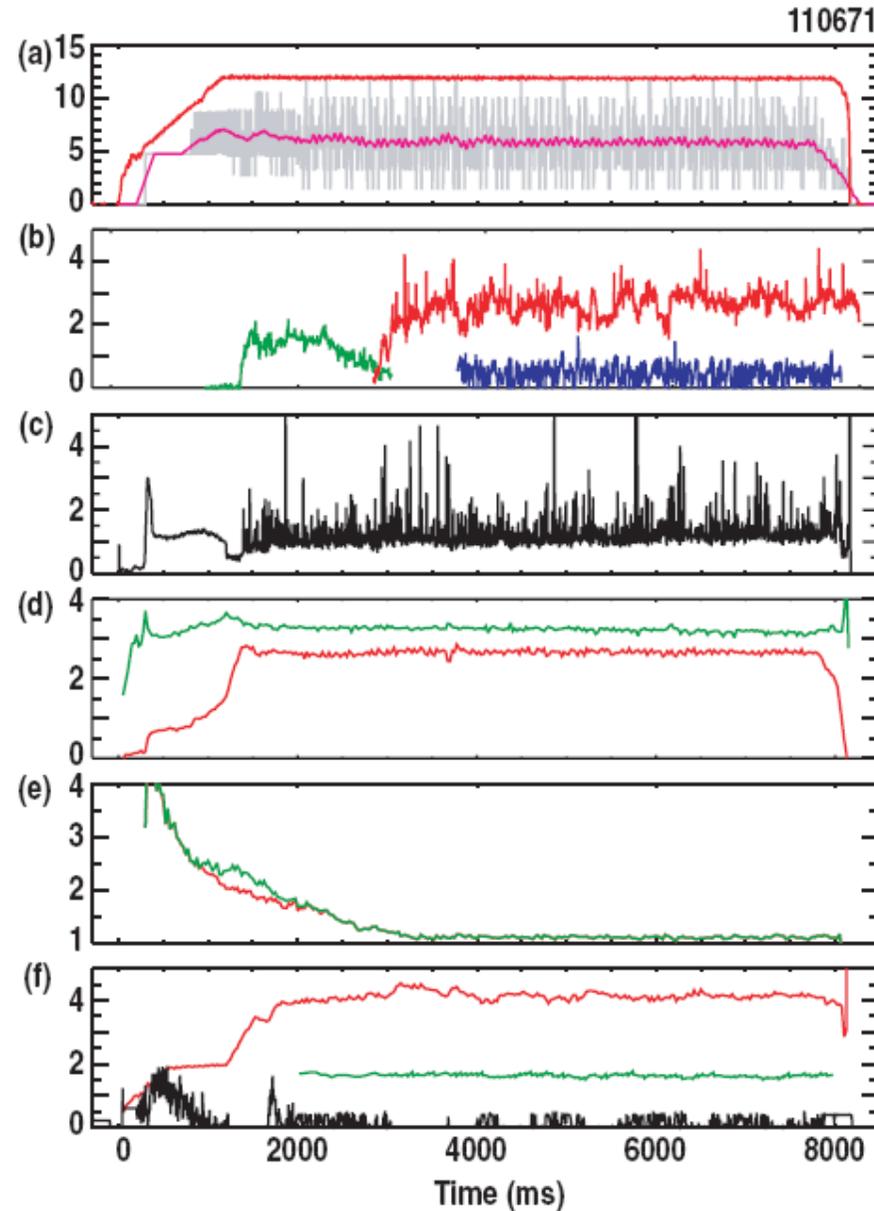
Magn. perturbation
 $n=3, n=2, n=1$

D_α , upper divertor

β_N
 $4I_i$

q_{min}
 $q(0)$

n_e
 Z_{eff}



DIII-D

$n_e = 0.4 \cdot 10^{20} \text{ m}^{-3}$
 $P_{NBI\text{labs}} = 4.8 \text{ MW.}$

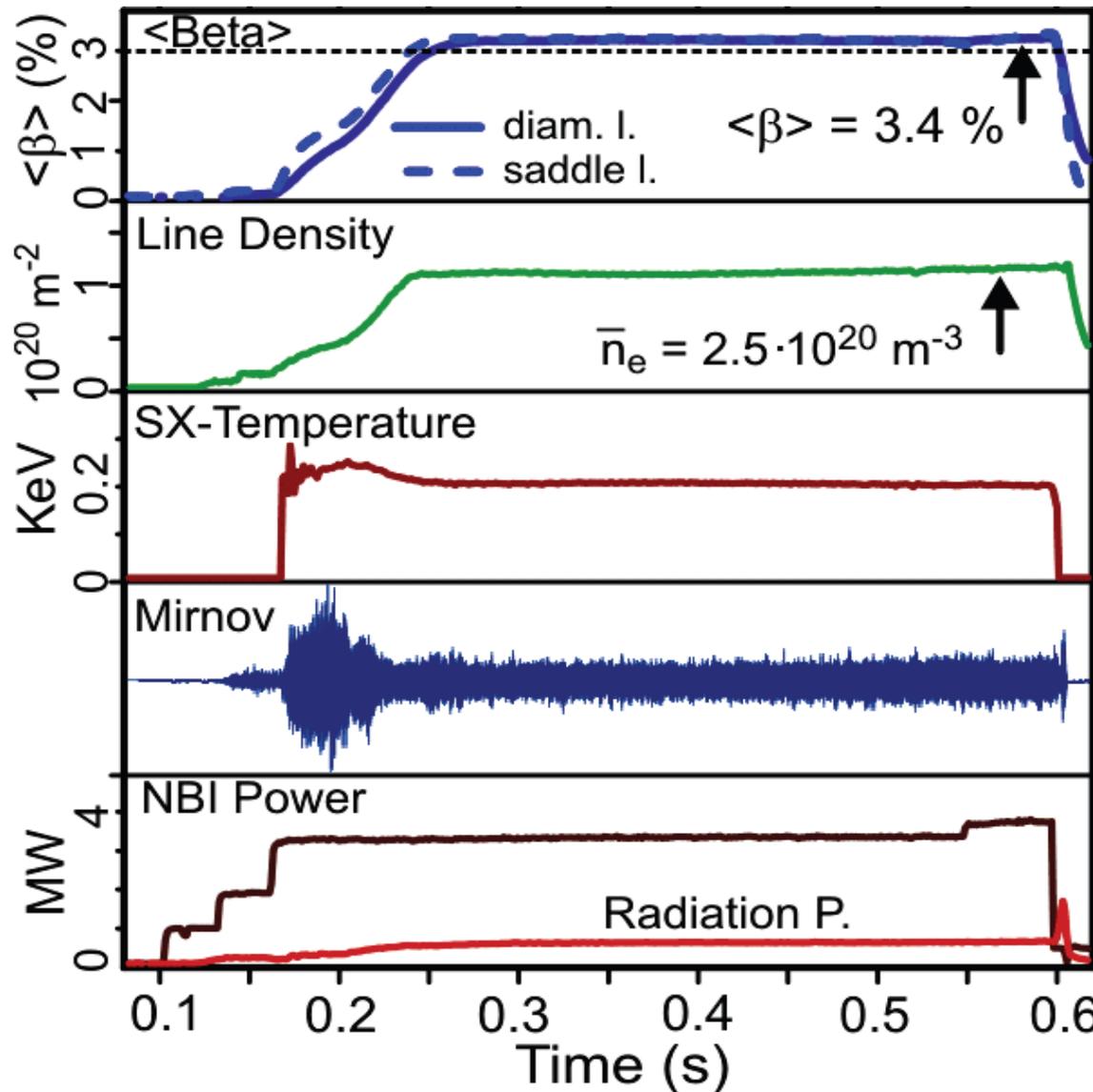
$\beta \sim 3\%$
 $\beta_N = 2.7$
 $H_{89} = 2.6$
 $n_e/n_{eGW} = 0.4$

Mapped to
ITER
 $Q=10$

Steady-state
 $\Delta t \sim 36 \tau_E$



Long-pulse HDH discharge of W7-AS



HDH regime

$$B = 0.9 \text{ T}$$

$$n_e = 2.5 \cdot 10^{20} \text{ m}^{-3}$$

$$P_{\text{NBI labs}} = 2.5 \text{ MW}$$

$$\beta = 3.4\%$$

$$\beta_N \sim 9.3$$

$$H_{\text{ISS95}} = 1.4$$

$$n_e/n_{e\text{GW}} = 2.5$$

$$\tau_I/\tau_E \sim 2$$

$$\Delta t \sim 36 \tau_E$$



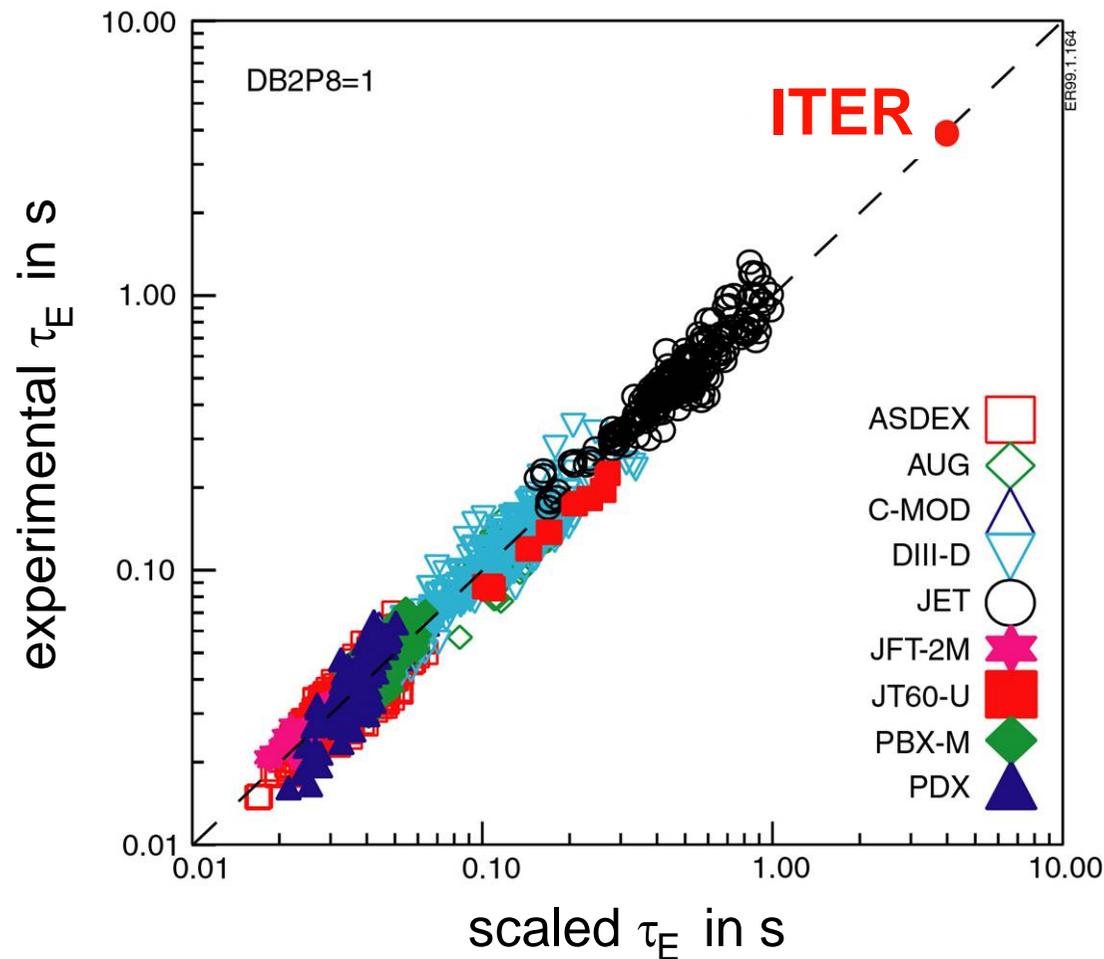
Prediction from regression analysis



Multi-experiment data base: Scaled $\tau_E = C a^{x^1} R^{x^2} B^{x^3} I^{x^4} n^{x^5} P^{x^6} A_i^{x^7} k^{x^8} \dots$

$x^1, x^2, x^4 > 0$

$x^6 < 0$ power degradation





Summary



1. The role of geometry: it allows to organise good confinement (properties)

axi-symmetric shaping: elongation, triangularity improve turbulent tokamak transport;

non-axi-symmetric shaping:

improves stellarator equilibrium, stability and non-turbulent (neo-classical) transport (what about turbulent transport?).

2. The plasma self-organises in the H-mode such that the turbulence is lower at larger driving forces and that the ignition conditions are approached.

The situation is involved however:

The understanding involves the

power balance

the toroidal momentum balance

the poloidal momentum balance

the SOL flow and viscous momentum transfer