Russian Research Center" Kurchatov Institute"



Qualification of behavior of new structural composite materials - SiC under neutron and charged particle irradiation

Alexander Ryazanov

Joint ICTP/IAEA Advanced Workshop on Development of Radiation Resistant Materials

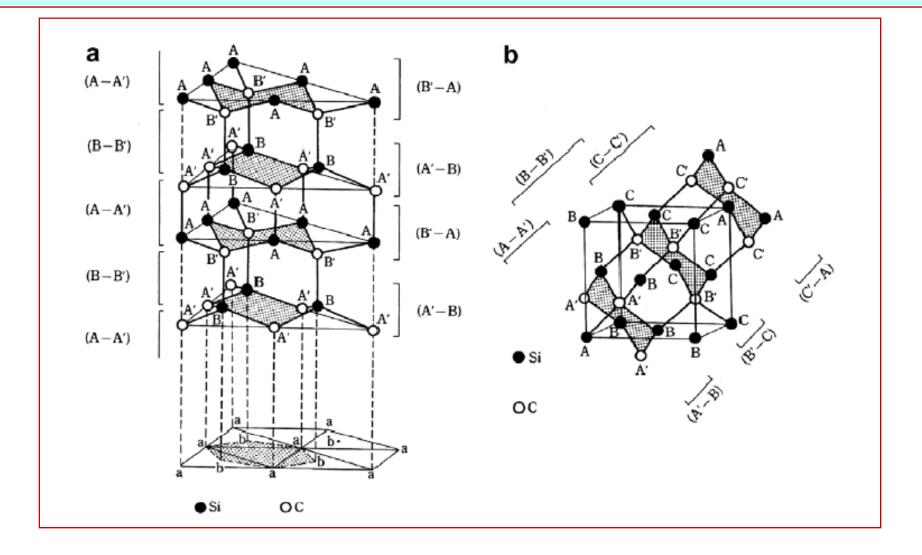
Fabrication of SiC

SiC was first synthesized in 1891 by Acheson: (E.G.Acheson, Chem.News 68 (1893) 179).

By electrochemical reaction in an electric furnace: $SiO_2 + 3C \rightarrow SiC + 2CO$

> **a-SiC** formed at T = 2373 K and **β-SiC** formed at T = 1273 - 1873 K

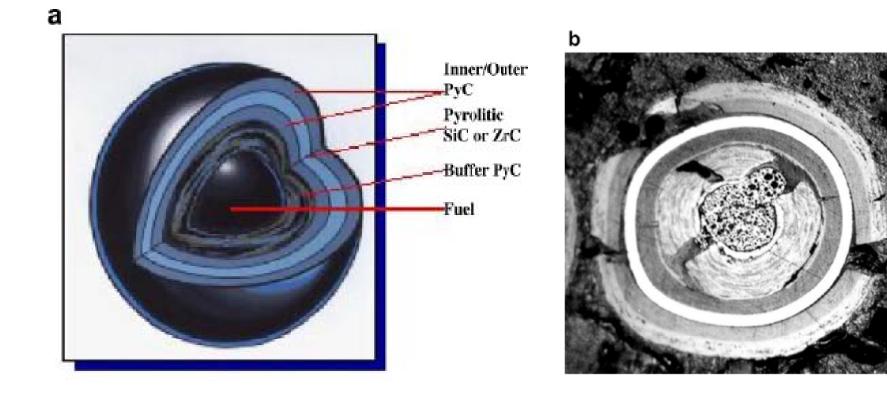
Crystal structures of (a) α-SiC and (b) β-SiC



Possibility for using of SiC materials in nuclear energy systems:

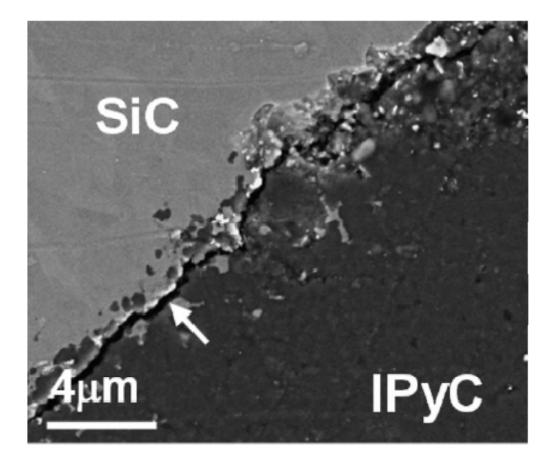
- fuel element in high temperature gas-cooled reactor (HTGR)
- fuel blocks for the gas fast reactors (GFR)
- guide tubes materials in very high temperature reactors (VHTR)
- test blanket module designs for the ITER

Schematic illustration of fuel elements: nonirradiated (a) and irradiated (b)



L. Snead, ORNL

Crack propagation at the inner Py/SiC interface



L. Snead, ORNL

Background and Objective

Solution Carbide - Superior Performance for advanced Energy systems Under the irradiation environment

- Potential irradiation resistance
- Low induced activity, Low after-heat
- At the elevated Temperature
- High strength, Chemical stability

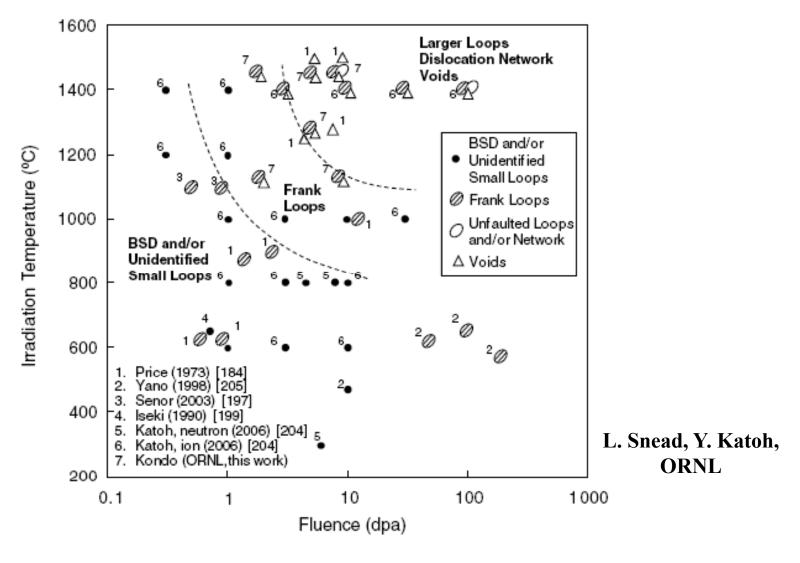
Issues under irradiation environment

- Irradiation induced volume expansion (swelling)
- Effects of helium production through (n, α) nuclear reaction (in fusion)

Objective

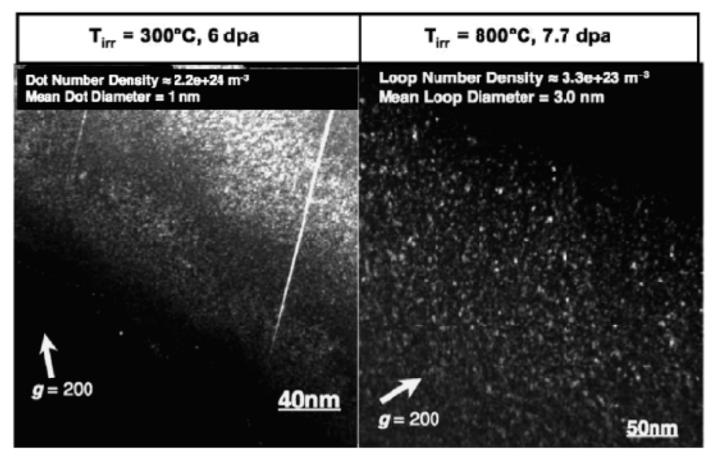
- Determine the effects of irradiation conditions on swelling in SC
- Investigation of the synergistic effects of displacement damage and helium production

Microstructural development in cubic SiC during neutron and ion irradiations



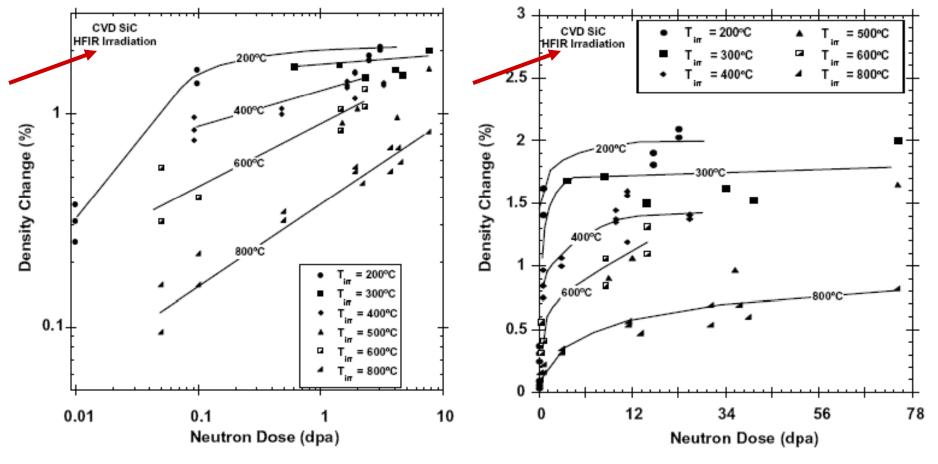
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Saturated microstructure for CVD SiC neutron irradiated at 573 K and 1073K



L. Snead, ORNL

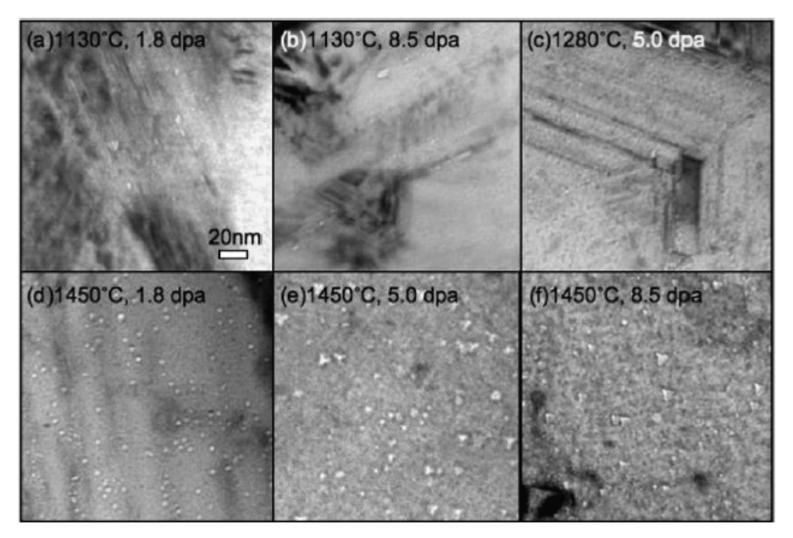
Swelling regime of CVD SiC at different irradiation doses and temperatures.



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L. Snead, ORNL

Evolution of voids in high temperature irradiated CVD SiC



L. Snead, ORNL

Experimental

Materials: Specimen geometry:	Morton CVD-SiC Rolycrystalline β-SiC) 3.0mm ^f x 0.25mm ^t		
lon beam irradiation:	5.01111 × 0.251		
Specimensare cov	/ered with molybden	um meshes	
Facility:	HIT facility, University of Tokyo		
lons:	4MeV Ni		
	4MeV Ni + 1MeV He (degraded)		
Temperature:	T= 333K, 473K, 673K, 873 K		
Dose rate:	f = 1x10 ⁻⁵ -1x10 ⁻³ dpa/s (nominal)		
Dose:	0.005-100dpa	(nominal)	
He/dpa ratio:	60 appm He/dpa	(nominal)	

Swelling characterization :

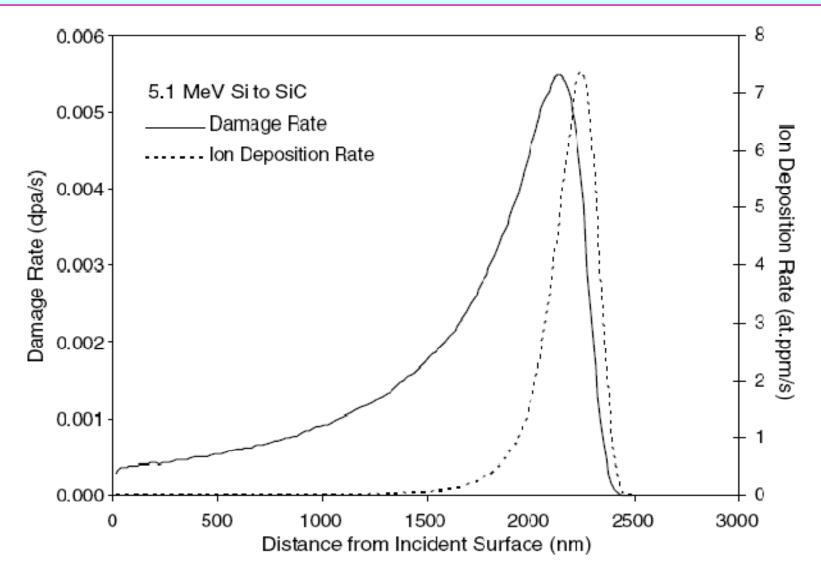
Surface profilometry by the optical interferometry

Thin film Processing :

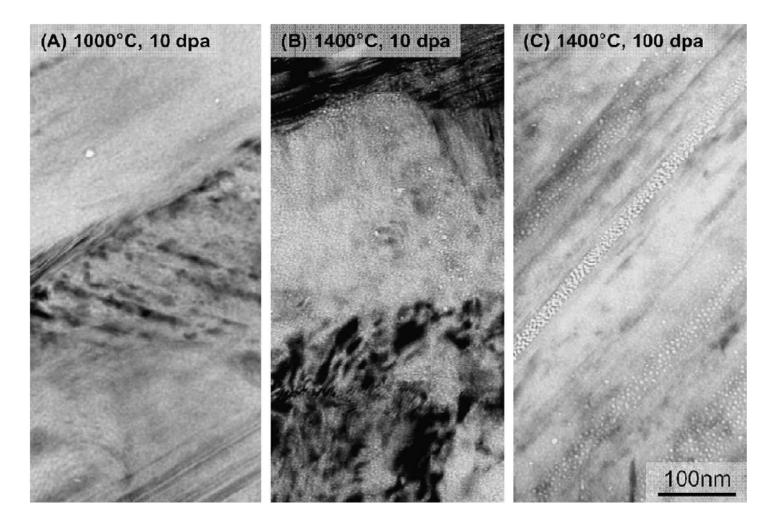
Performed by Focused ion beam (FIB) devise / JEOL/Micrion JFIB-2100

TEM observation : /JEOLJEM-2010 (200kV)

Displacement damage and ion deposited rate profiles in SiC irradiated by 5.1 MeV Si ions



Cavity microstructure in cubic SiC irradiated by 5.1 MeV Si ions



Y. Katoh, ORNL

Microstructural data for irradiated cubic SiC

Irr. temp. (°C) Dose ($\times 10^{25}$ n/m ² or dpa)		Black spot/loops			Cavities	
	Туре	Density (m ⁻³)	Radius ^a (nm)	Density (m ⁻³)	Radius (nm)	
Neutron (0.5×10^{-1})) ⁻⁶ dpa/s, HFIR, ORNL)					
300	6.0	Black spots	2.2×10^{24}	<0.5	Not detected	
800	4.5	Mix	2.6×10^{23}	1.3	Not detected	
800	7.7	Mix	3.3×10^{23}	1.5	Not detected	
Ion ($\sim 1 \times 10^{-3} dr$	oa/s, 5.1 MeV Si ²⁺ , DuET, Ky	oto University)				
600	10	Black spots	n/m ^b	n/m	Not detected	
800	10	Mix	n/m	2.2	Not detected	
1000	10	Loops	2.6×10^{23}	~ 2	${<}1{\times}10^{20}$	1.6°
1400	10	Loops	2.3×10^{21}	~ 5	2.0×10^{22} d	~ 2.0
1400	30	Loops	2.3×10^{21}	12.0	1.3×10^{24} d	~ 2.0
1400	100	Loops	5.2×10^{21}	18.1	1.8×10^{24} d	~ 2.0

^a 1/2 of the approximate mean size for the black spot defects.

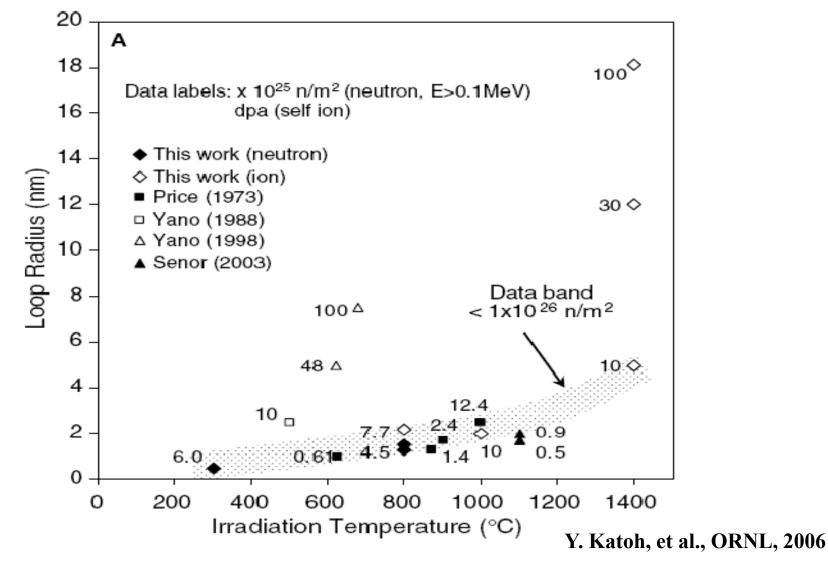
^b Not measured.

^c Grain boundary cavities.

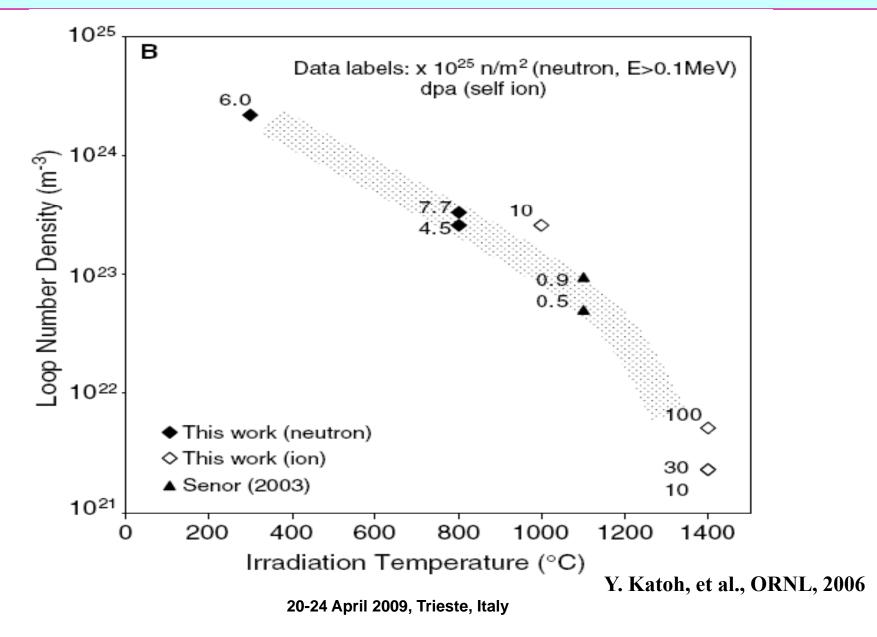
^d Local number density of grain/twin boundary cavities.

Y. Katoh, et al., ORNL, 2006

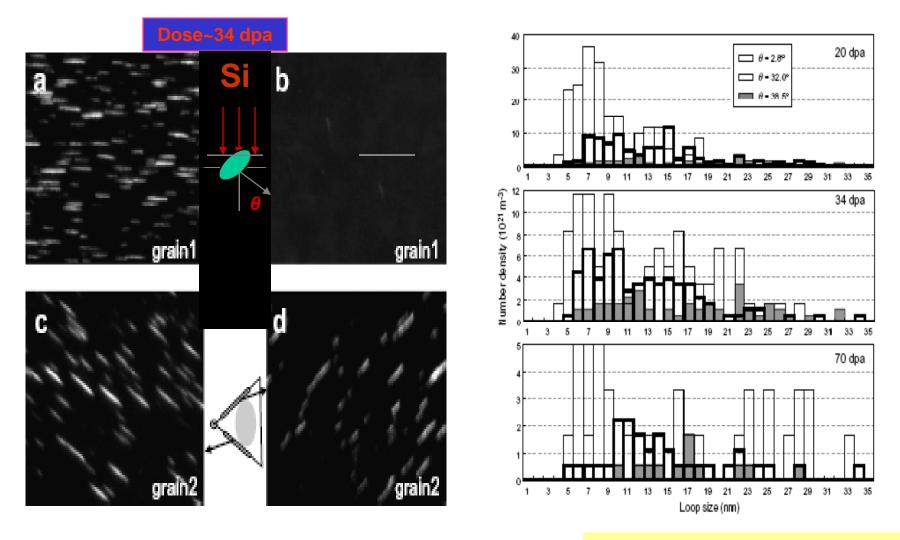
Temperature dependence of dislocation loop growth



Temperature dependence of dislocation loop density



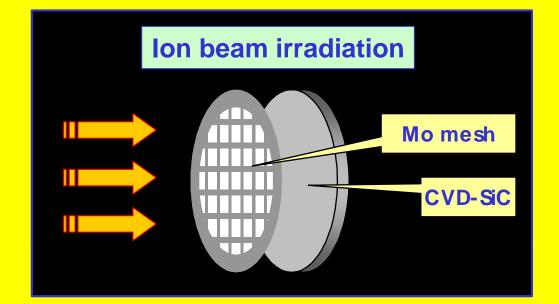
TEM images of Frank loops in SiC under 5.1 MeV Si ion irradiation at 1400 C

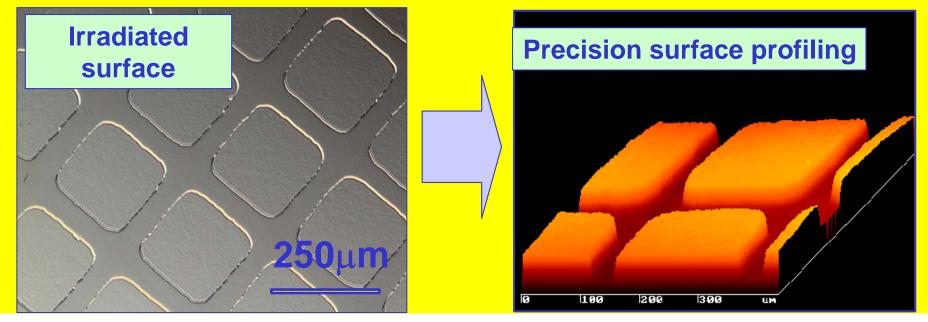


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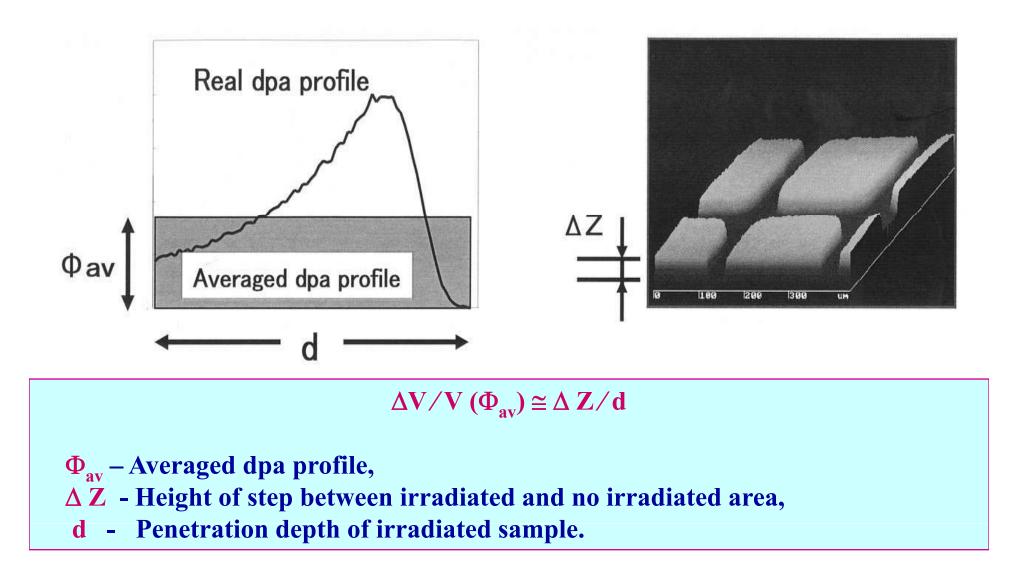
(A.Kohyama et al.,2007)

Ion beam irradiation and Surface profile characterization

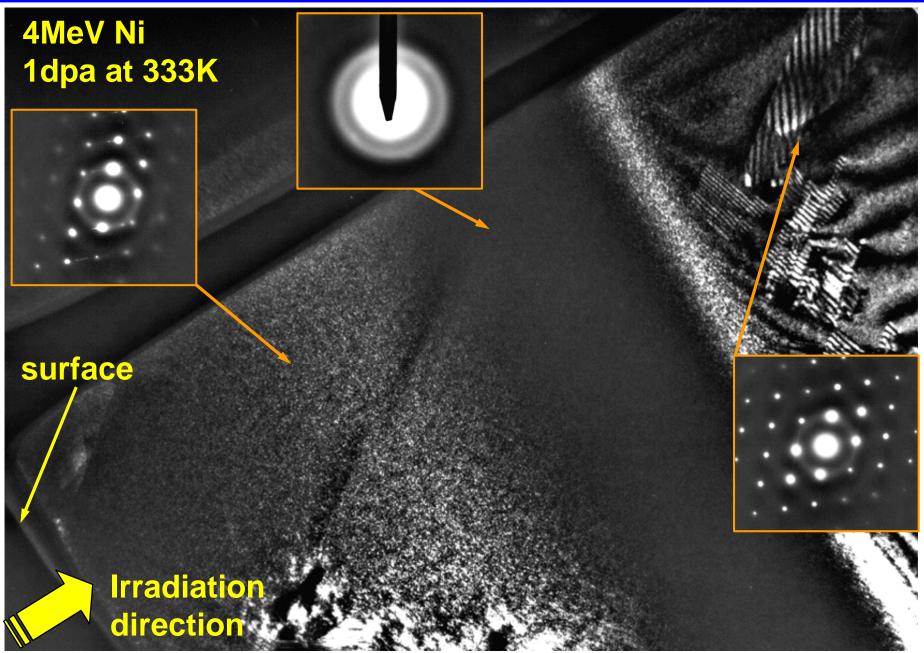




Experimental Measurement of Radiation Swelling

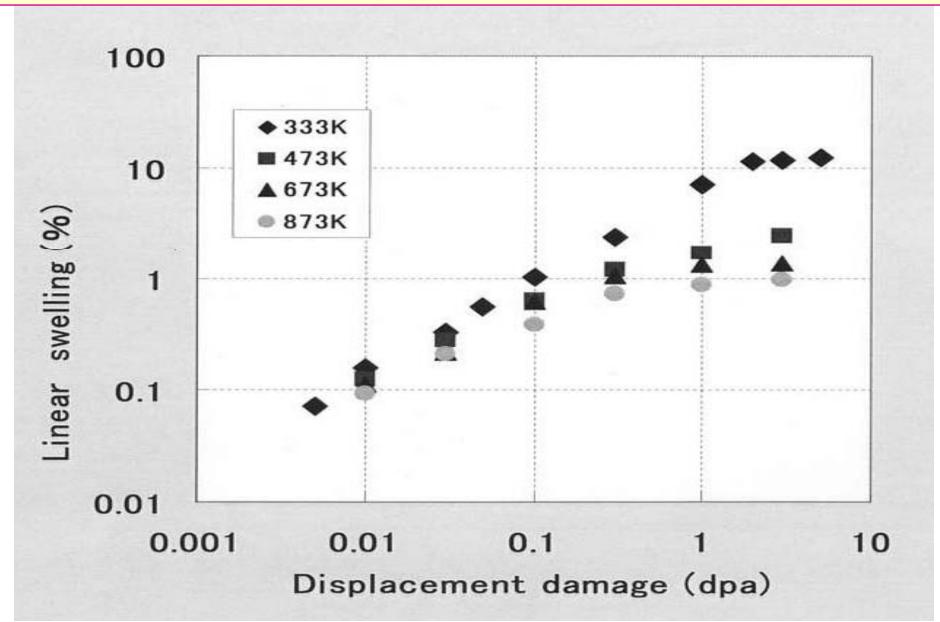


Cross-sectional TEM of ion-irradiated CVD-SiC



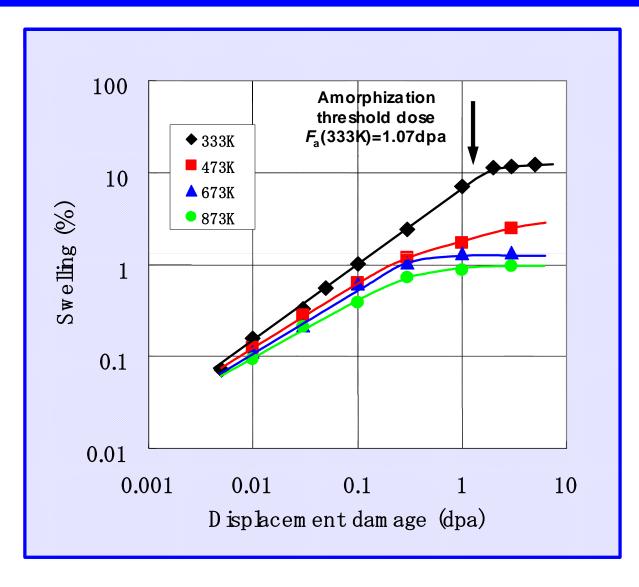
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Dose dependence of radiation swelling in SiC

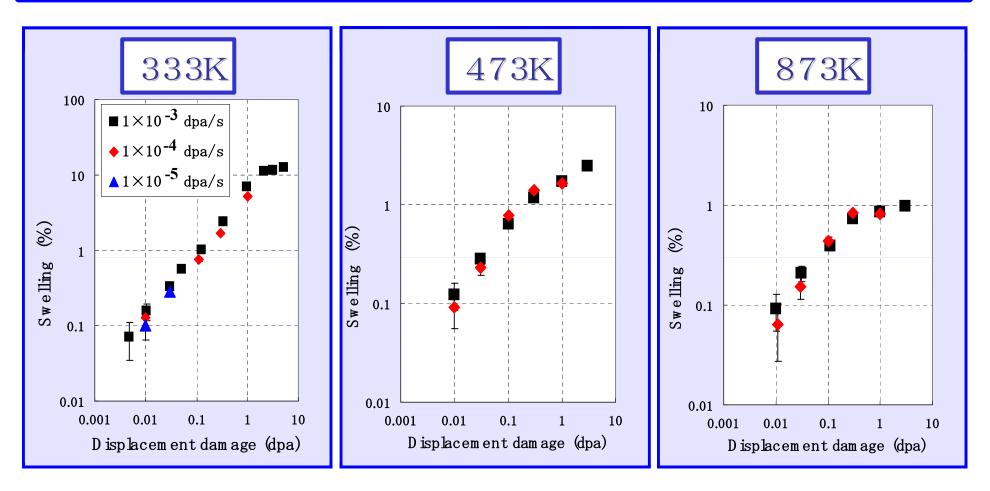


²⁰⁻²⁴ April 2009, Trieste, Italy

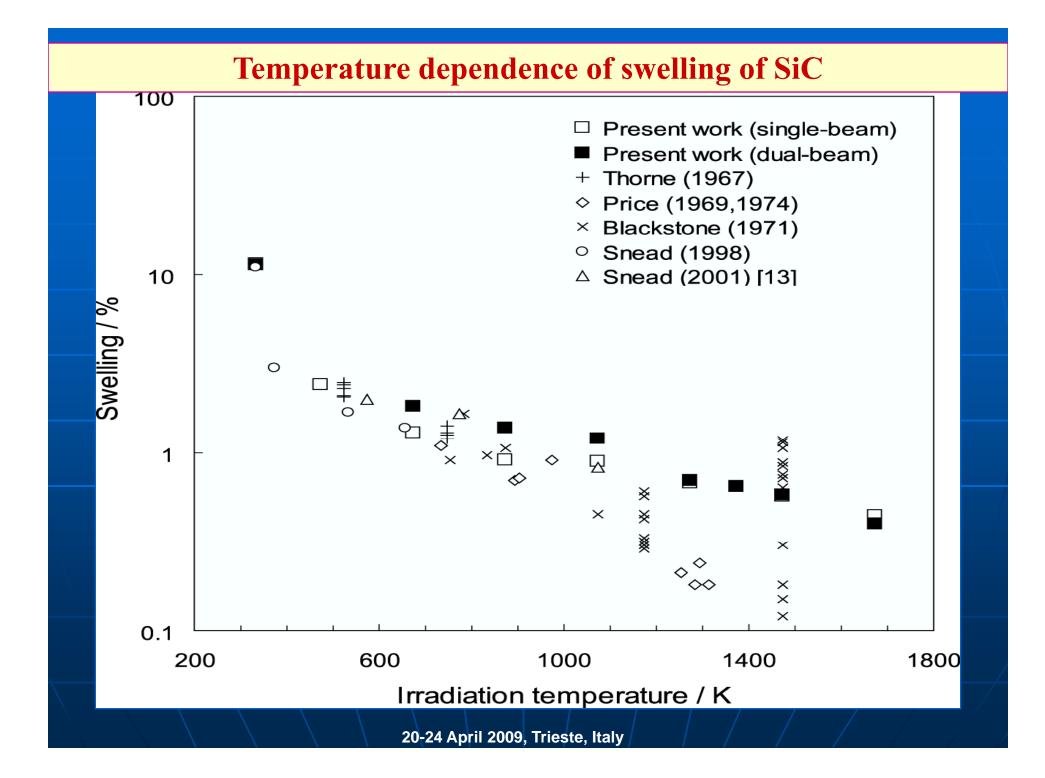
Temperature dependence of Ion-induced swelling in CVD-SiC



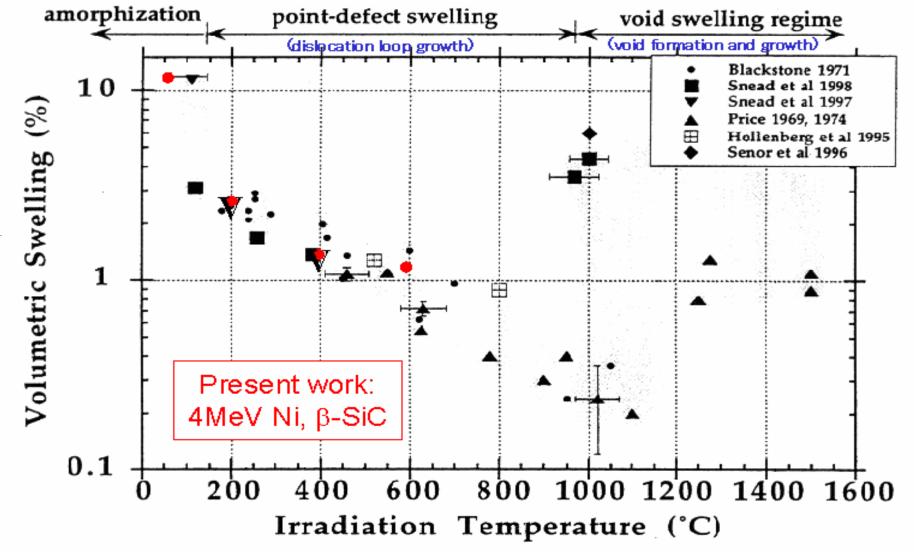
Dose rate dependence of Ion-induced swelling in CVD-SiC



The displacement damage rates were 1×10^{-4} and 1×10^{-3} dpa/sat 333K-873K and 1×10^{-5} dpa/sat 333K with single-beam irradiation. The error bars represent the 96% confidence limits for the Gaussian distribution.

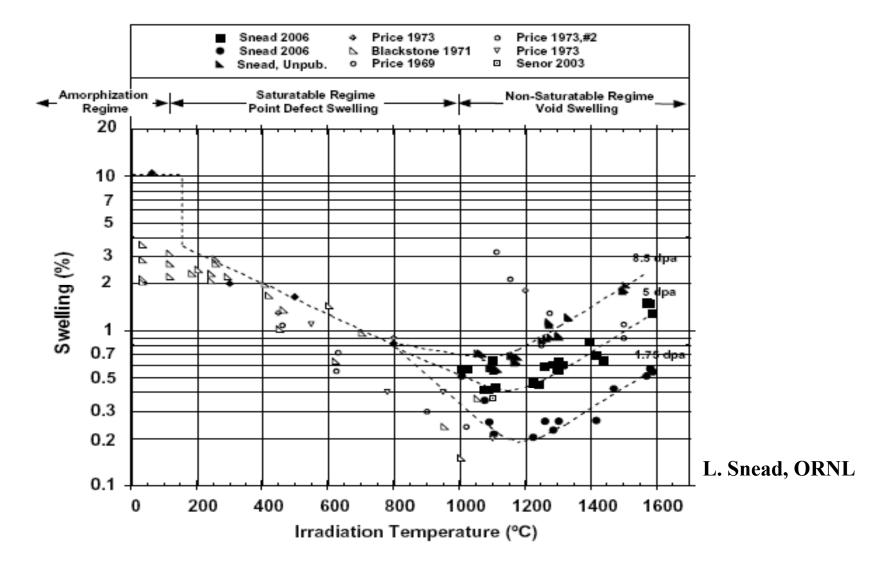


Temperature dependence of swelling of SiC



S.J.Zinkle and L.L.Snead, DOE/ER-0313/24 (1998) 93-114.

Temperature dependence of irradiationinduced swelling of SiC



Difference between metals and dielectrics

Metals:

- Point defects are neutral
- Electric field does not exist in the matrix

Dielectrics (Ceramic Materials):

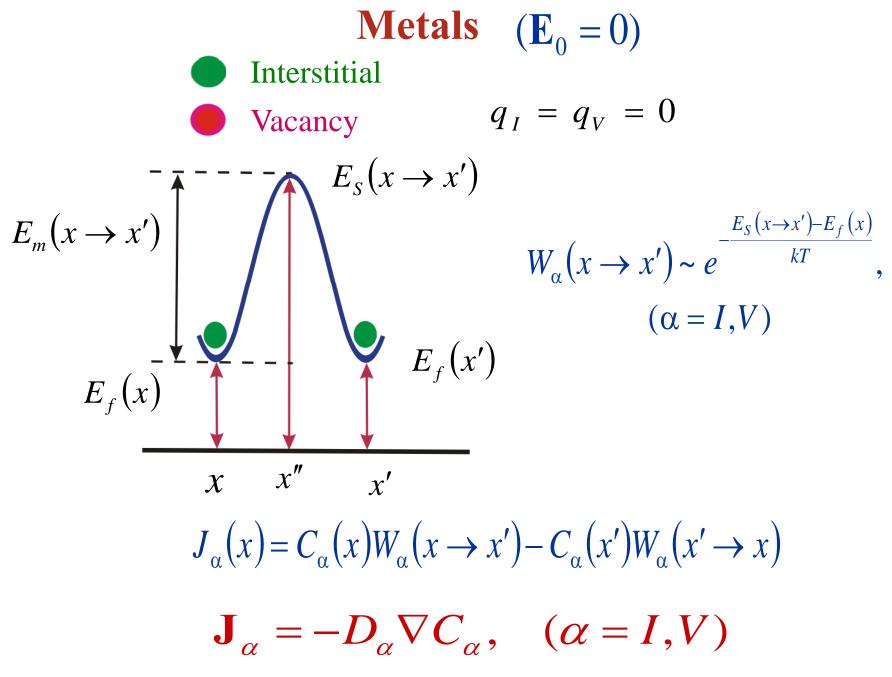
- Point defects can have effective charge
- Electric field exists in the matrix under the influence of an applied electric field

• Driving force due to an electric field can have a strong effect on diffusivity of charged point defects

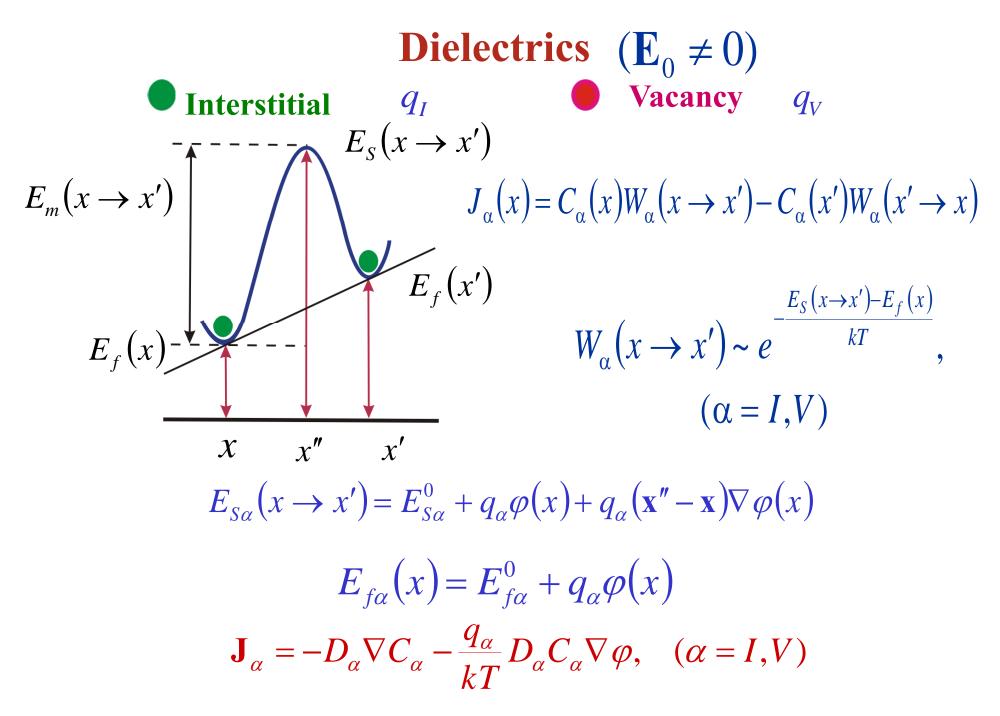
Development of theoretical models for the calculations of radiation swelling in SiC materials.

Physical Processes:

- Generation of Point Defects
- Recombination of Point Defects
- Nucleation of Dislocation Loop
- Growth of Dislocation Loops
- Radiation Swelling

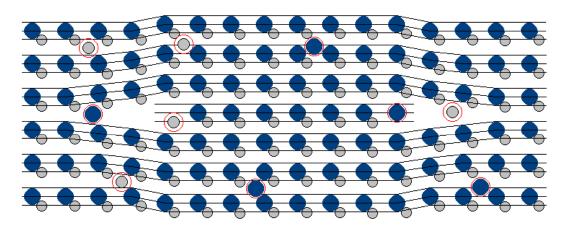


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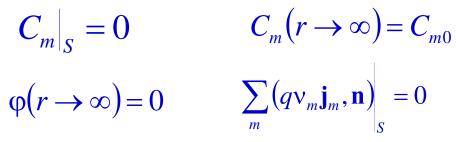
Dislocation Loop in Dielectrics



System of Equations

$$D_{m}\Delta C_{m} + \frac{qv_{m}}{kT}D_{m}\nabla(C_{m}\nabla\varphi) = 0$$
$$\Delta\varphi = -\frac{4\pi}{\varepsilon\omega}\left(\sum_{m}qv_{m}C_{m} + \rho\right)$$

Boundary Conditions



Theoretical model of radiation swelling in SiC

Radiation swelling (S_{tot}) is determined in ceramic materials by the following relation

$$S_{tot} = \sum_{K=1}^{2} C_{IK} e_{IK} + \sum_{K=1}^{2} C_{VK} e_{VK} + \omega \sum_{S,K} (n_{IK}^{S} e_{IK} + n_{VK}^{S} e_{VK})$$

 e_{α} is the dilatation of point defect type α ($\alpha = I$ for interstitial atoms, $\alpha = V$ for vacancies and $\alpha = He$ for helium atoms), ω is the atomic volume, $n_{\alpha k}^{S}$ is the total number of point defects of the type α absorbed by sinks of the type *s* (loops, voids) in an unit volume,

 $C_{\alpha k}$ is the concentration of point defects for the two components: k=1=Si , k=2=C in SiC

$$\frac{dC_{VK}}{dt} = G_{VK} - j_{VK}(\rho_D + \rho_L) - \alpha D_{IK}C_{IK}C_{VK}$$
$$\frac{dC_{IK}}{dt} = G_{IK} - j_{IK}(\rho_D + \rho_L) - \alpha D_{IK}C_{IK}C_{VK} - \mu (D_{I1} + D_{I2})C_{I1}C_{I2}$$

 G_{VK}, G_{IK} are the generation rates of vacancies and interstitial atoms k-th components, D_{IK}, D_{VK} are the diffusion coefficients of interstitial atoms and vacancies k-th component, ρ_d is the dislocation density, ρ_L is the dislocation loop density ($\rho_L = 2\pi R_L N_D$). The dislocation loop density is determined from the following relation

$$\omega \frac{dN_{L}}{dt} = \mu \left(D_{I1} + D_{I2} \right) C_{I1} C_{I2}$$

The growth rate of dislocation loop with loop radius R in ceramic materials taking into account the absorption of two types of interstitial atoms and vacancies and remaining of stoichiometric of two components in dislocation loop is given by the following relation

$$\frac{dR_L}{dt} = \frac{\pi r_0}{b} \sum_{K} \left(j_{IK}^n - j_{VK}^n \right) = \frac{4\pi}{b \ln\left(\frac{8R}{r_0}\right)} \frac{D_{I1}C_{I1}D_{I2}C_{I2} - D_{V1}C_{V1}D_{V2}C_{V2}}{D_{I1}C_{I1} + D_{I2}C_{I2} + D_{V1}C_{V1} + D_{V2}C_{V2}}$$

The initial conditions (at t = 0):

$$C_{IK}(t=0) = 0$$
 $C_{VK}(t=0) = 0$ $R_L(t=0) = a$

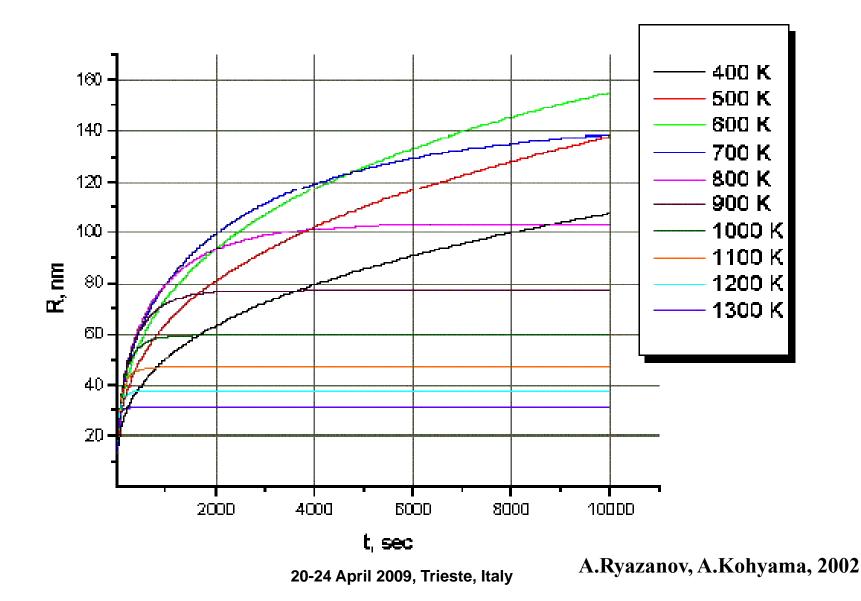
Main parameter values used for numerical calculations of radiation swelling in SiC

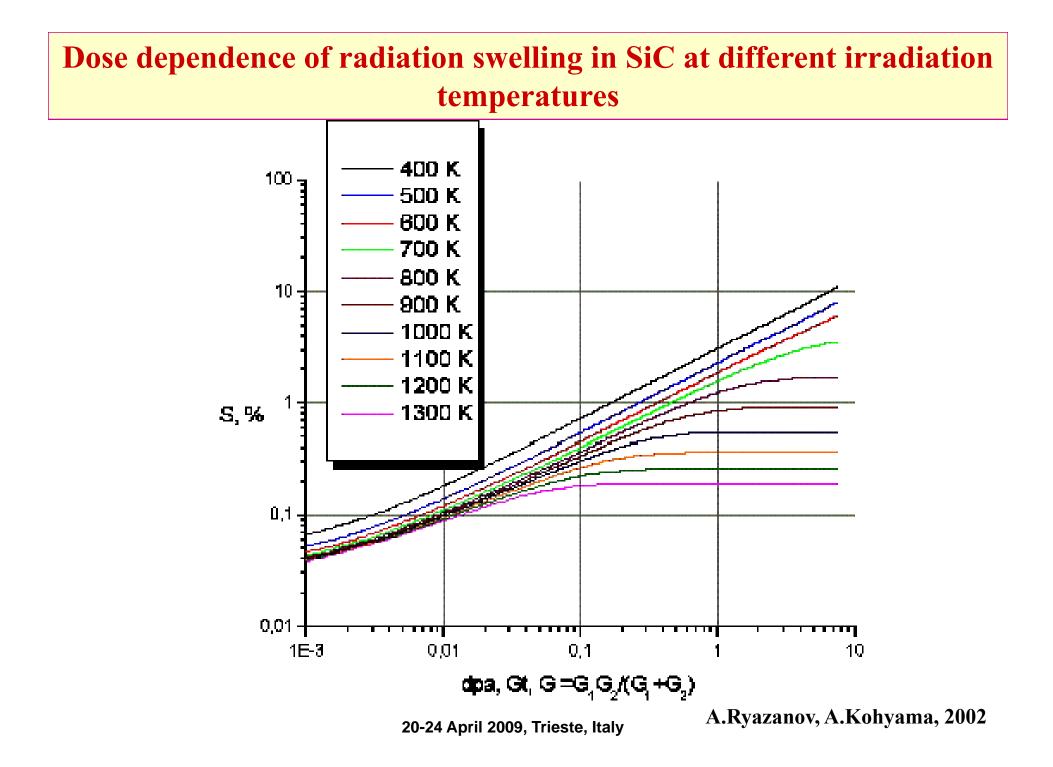
$G_1 = G_{Si}$	Point defect generation rate of Si atoms	3.10 ⁻³ dpa/s
$\mathbf{G}_2 = \mathbf{G}_C$	Point defect generation rate of C atoms	1.10 ⁻³ dpa/s
$m{E}_{mV}^{Si}$	Silicon vacancy migration energy	2.3 eV
E_{mV}^{c}	Carbon vacancy migration energy	2.0 eV
E_{ml}^{Si}	Silicon interstitial migration energy	0.4 eV
E_{ml}^{C}	Carbon interstitial migration energy	0.3 eV
${\cal E}_{FV}^{Si}$	Silicon vacancy formation energy	2.5 eV
E_{FV}^{c}	Carbon vacancy formation energy	2.4 eV
$ ho_{\scriptscriptstyle D}$	Network dislocation density	10 ¹⁰ cm ⁻²
$e_{V1} = e_{V2}$	Vacancy dilatation	-0.1
a	Lattice parameter	$5.14 \times 10^{-8} \text{ cm}$

$$D_{VK} = D_{VK}^{O} \exp(-E_{mV}^{K} / T), \text{(where } D_{V1}^{O} = D_{V2}^{C} = 10^{-2} \text{ cm}^{-2}\text{)},$$

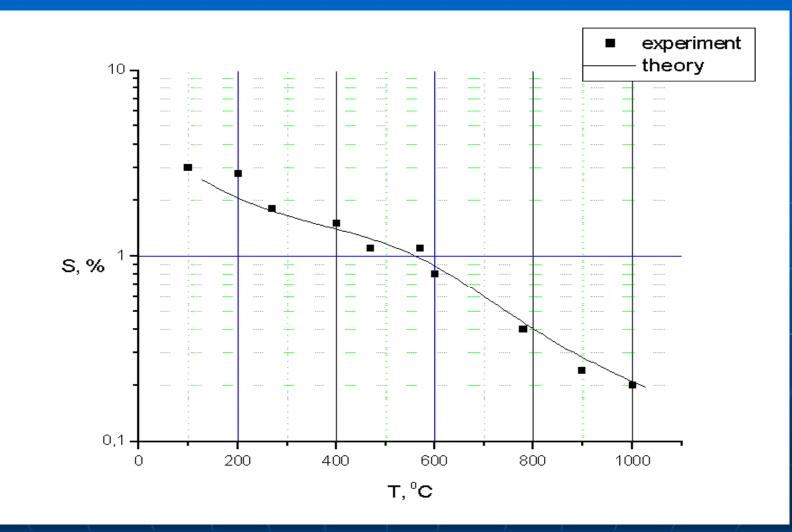
$$N_{L} = N_{L}^{O} [\exp(E_{m1}^{1} / T) + \exp(E_{m1}^{2} / T)]^{1/2}, \text{ (where } N_{L}^{O} = 3.10^{12} \text{ cm}^{-3}\text{)}.$$

The time dependence of dislocation loop growth at different irradiation temperatures





The comparison of experimental and theoretical temperature dependencies of radiation swelling in SiC.



A.Ryazanov, A.Kohyama, 2002

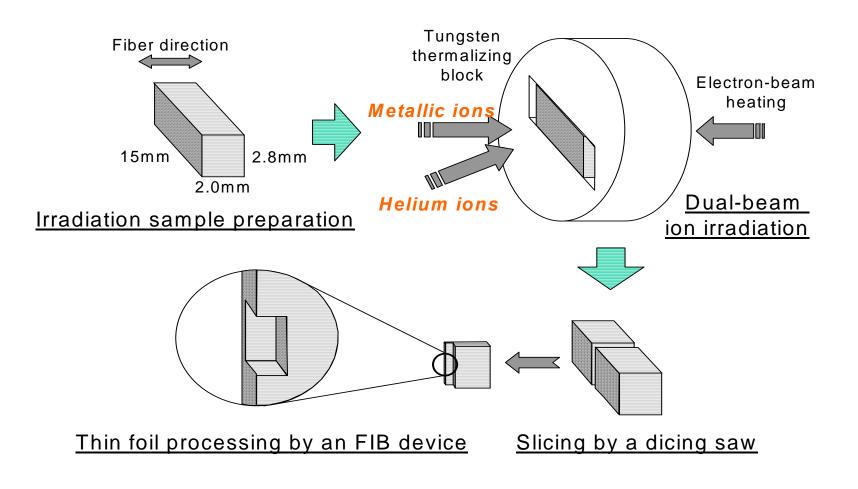
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Helium Effect on Radiation Swelling of SiC

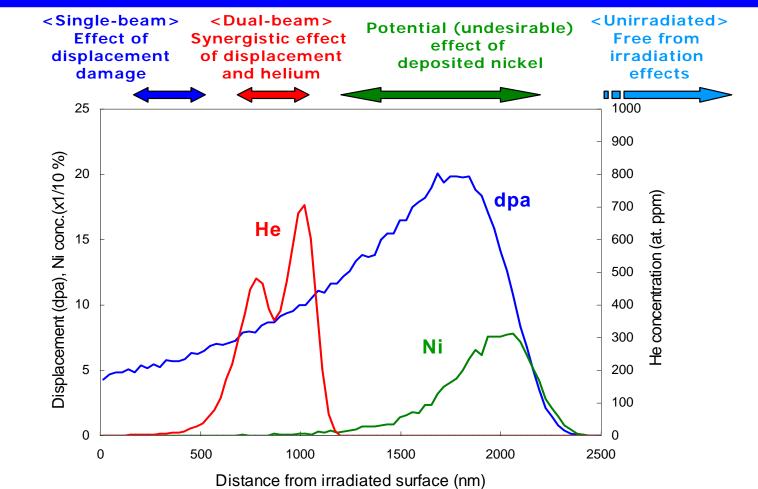
Experimental procedure illustrated

Institute of Advanced Energy

Kyoto University



Profiles of displacement damage and deposited Ni in irradiated monolithic SiC. Calculated by TRIM-92 assuming Ed=35eV, ρ =3.21g/cm³.



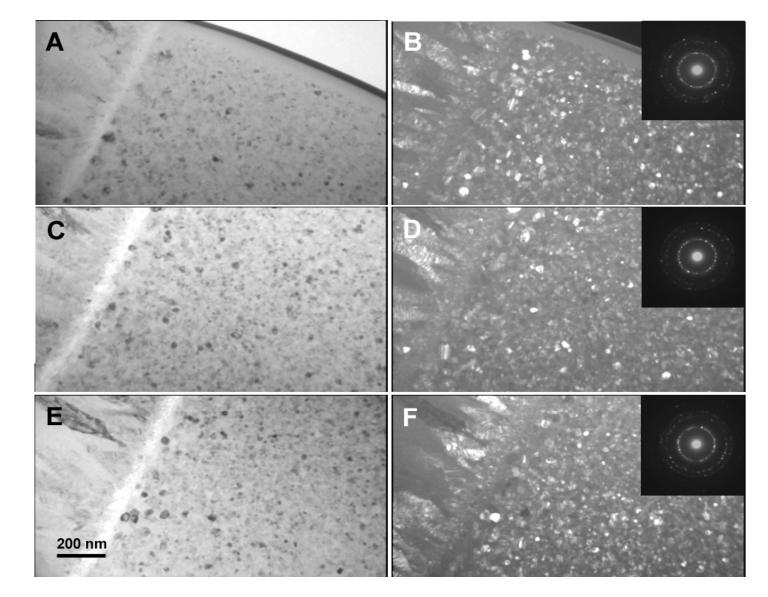
Depth-profiles of atomic displacement damage, deposited He and Ni ions in dualbeam irradiated randomly oriented micro-crystalline SiC calculated by TRIM-92 assuming target mass density of 3.21g/cm³ and average displacement threshold energy of 35eV.

TEM images and SAD patterns for single-beam (A,B), dual-beam (C,D) and unirradiated (E,F) regions of Hi-Nicalon® Type-S/C/SiC composite

Single-beam 10 dpa 873 K 1x10⁻³dpa/s

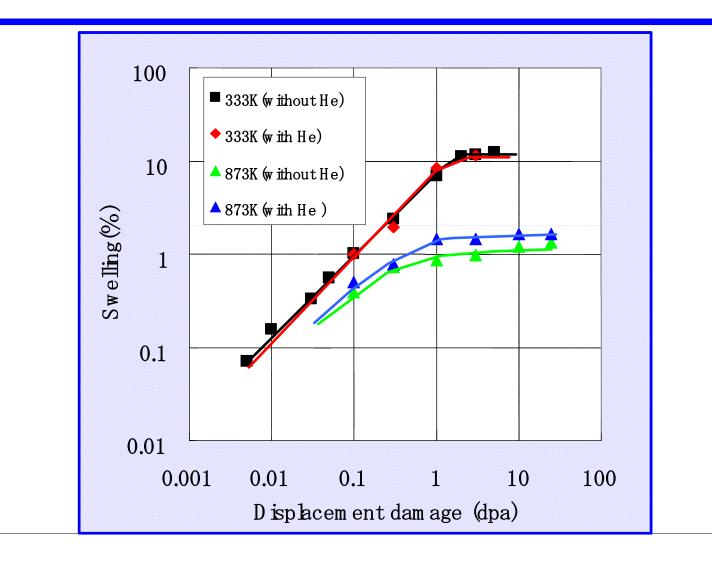
Dual-beam 10 dpa 873 K 1x10⁻³dpa/s 60appm-He/dpa

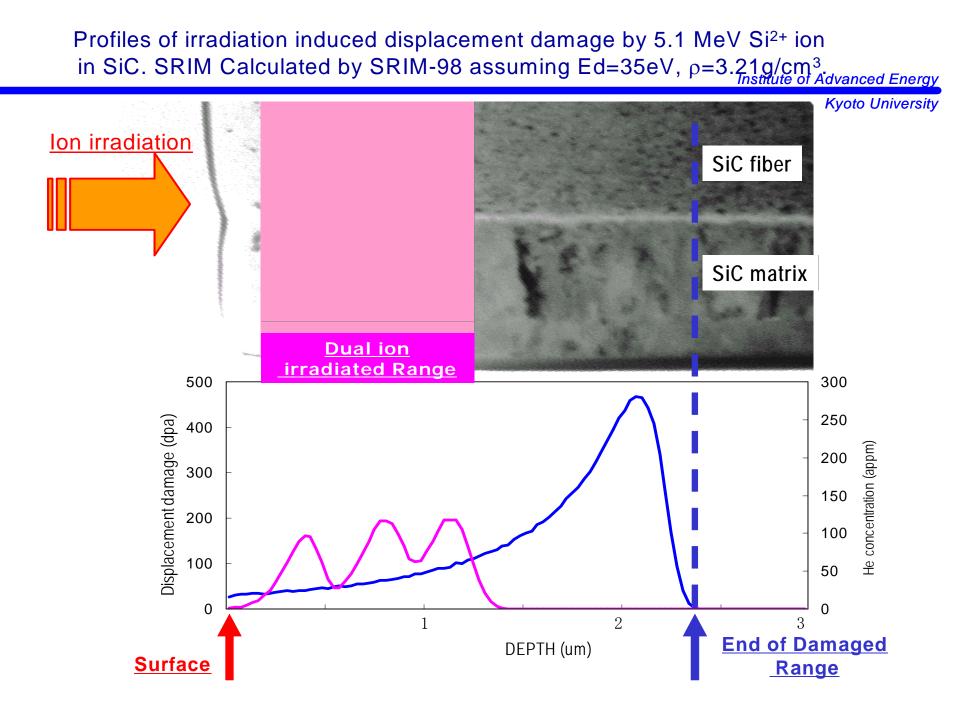
Unirradiated Dark field images from SiC <111> diffraction rings.



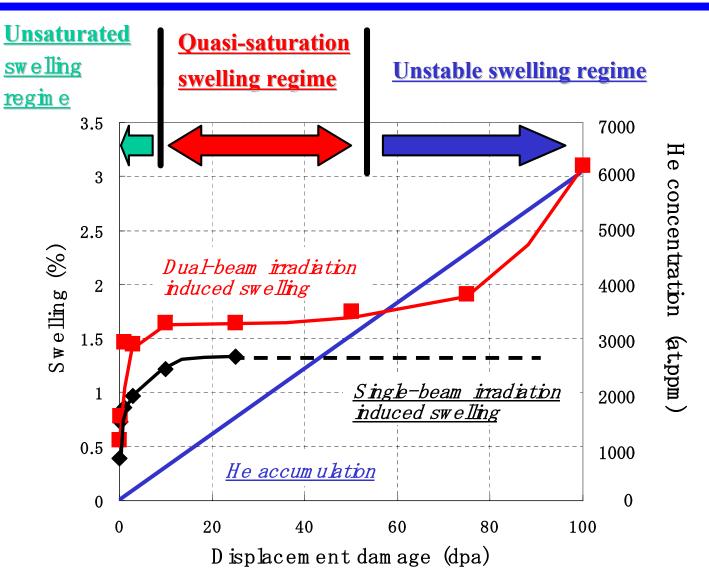
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Dual ion beam irradiation-induced swelling in CVD-SiC





He accumulation and irradiation-induced swelling in dual- and single-ion irradiated CVD-SiC at 873K



Theoretical model of helium influence on radiation swelling in SiC

Radiation swelling (S_{tot}) is determined in ceramic materials by the following relation

$$S_{tot} = \sum_{K=1}^{2} C_{IK} e_{IK} + \sum_{K=1}^{2} C_{VK} e_{VK} + C_{He} e_{He} + \sum_{K=1}^{2} C_{HeVK} e_{HeVK} + \omega \sum_{S,K} (n_{IK}^{S} e_{IK} + n_{VK}^{S} e_{VK})$$

 e_{α} is the dilatation of point defect type α ($\alpha = I$ for interstitial atoms, $\alpha = V$ for vacancies and $\alpha =$ He for helium atoms), ω is the atomic volume.

 $C_{\alpha k}$ is the concentration of point defects for the two components: k=1=Si , k=2=C in SiC $C_{\mu \rho}$ is the concentration of helium atoms.

$$\frac{dC_{VK}}{dt} = G_{VK} - j_{VK}(\rho_D + \rho_L) - \alpha D_{IK}C_{IK}C_{VK} - \nu D_{He}C_{He}C_{VK}$$
$$\frac{dC_{IK}}{dt} = G_{IK} - j_{IK}(\rho_D + \rho_L) - \alpha D_{IK}C_{IK}C_{VK} - \mu (D_{I1} + D_{I2})C_{I1}C_{I2} - \gamma (D_{IK} + D_{He})C_{IK}C_{He}$$
$$\frac{dC_{He}}{dt} = G_{He} - \nu D_{He}C_{He}C_{V1} - \nu D_{He}C_{He}C_{V2} - \gamma (D_{I1} + D_{He})C_{I1}C_{He} - \gamma (D_{I2} + D_{He})C_{I2}C_{He}$$

 G_{VK}, G_{IK}, G_{He} are the generation rates of vacancies, interstitial atoms k-th components and helium atoms respectively, D_{IK} D_{VK} are the diffusion coefficients of interstitial atoms and vacancies k-th component respectively

$$\omega \frac{dN_{L}}{dt} = \mu (D_{I1} + D_{I2})C_{I1}C_{I2} + \gamma (D_{I1} + D_{He})C_{I1}C_{He} + \gamma (D_{I2} + D_{He})C_{I2}C_{He}$$

The growth rate of dislocation loop with loop radius R in ceramic materials taking into account the absorption of two types of interstitial atoms and vacancies and remaining of stoichiometric of two components in dislocation loop is given by the following relation

$$\frac{dR_L}{dt} = \frac{\pi r_0}{b} \sum_{K} \left(j_{IK}^n - j_{VK}^n \right) = \frac{4\pi}{b \ln \left(\frac{8R}{r_0} \right)} \frac{D_{I1}C_{I1}D_{I2}C_{I2} - D_{V1}C_{V1}D_{V2}C_{V2}}{D_{I1}C_{I1} + D_{I2}C_{I2} + D_{V1}C_{V1} + D_{V2}C_{V2}}$$

The initial conditions (at t = 0):

$$C_{IK}(t=0) = 0$$
 $C_{VK}(t=0) = 0$ $R_L(t=0) = a$ $C_{He}(t=0) = 0$

The concentration of vacancy-helium atom compounds is determined by solving the following equation

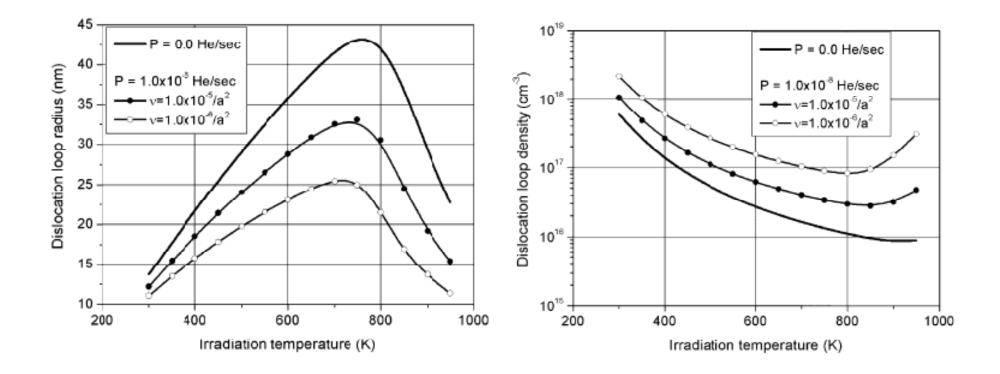
$$\frac{dC_{HeV1}}{dt} = vD_{He}C_{He}C_{V1}, \quad \frac{dC_{HeV2}}{dt} = vD_{He}C_{He}C_{V2}$$

Main parameter values used for numerical calculations of radiation swelling in SiC

$G_1 = G_{Si}$	Point defect generation rate of Si atoms	1.0×10 ⁻³ dpa/s
$G_2 = G_C$	Point defect generation rate of C atoms	2.5×10 ⁻³ dpa/s
$G_{{\scriptscriptstyle H\!e}}$	Helium atom generation rate	6.0×10 ⁻⁸ He/s
E_{mV}^{Si}	Silicon vacancy migration energy	2.9 eV
E_{mV}^{C}	Carbon vacancy migration energy	2.4 eV
E_{mI}^{Si}	Silicon interstitial migration energy	0.8 eV
E_{mI}^{C}	Carbon interstitial migration energy	0.7 eV
E_m^{He}	Helium atom migration energy	0.3 eV
$e_{I1} = e_{I2}$	Interstitial dilatation	1.2
$e_{V1} = e_{V2}$	Vacancy dilatation	-0.1
$e_{_{He}}$	Helium atom dilatation	1.2
$e_{HeV1} = e_{HeV2}$	Helium atom-vacancy complex dilatation	1.1
ρ_D	Network dislocation density	10 ⁻¹⁰ cm ⁻²
a	Lattice parameter	3.0×10 ⁻⁸ cm

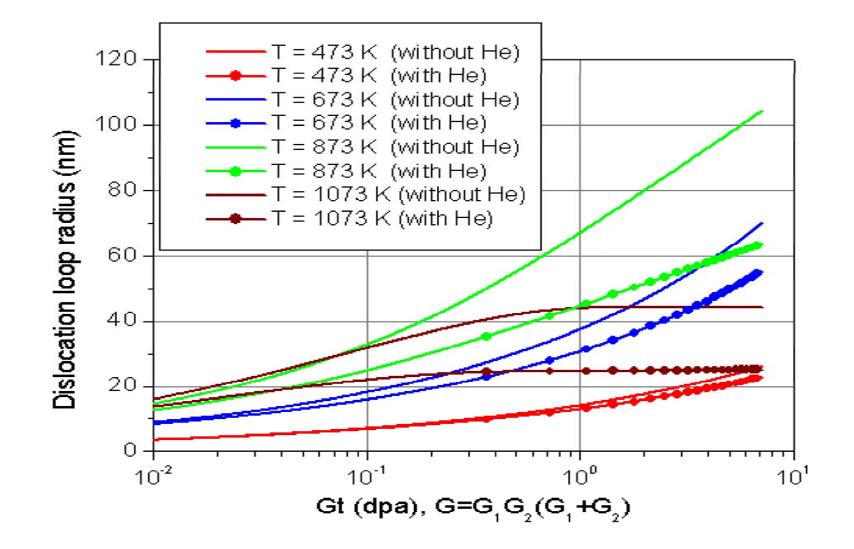
 $D_{\alpha K} = D_{\alpha K}^{O} \exp(-E_{m\alpha}^{K}/T)$, (where $D_{\alpha K}^{O} = 1 \ cm^{-2}$)

Temperature dependencies of dislocation loop radius and density



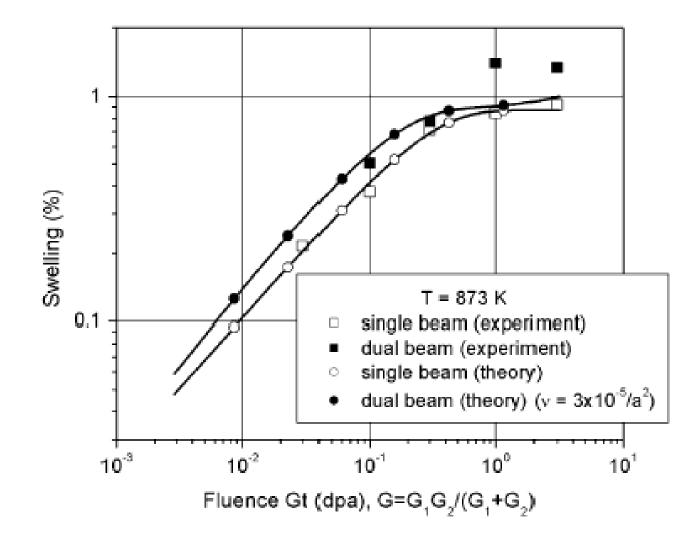
A.Ryazanov, A.Kohyama, Y.Katoh, 2004

Dose dependence of dislocation loop growth in SiC with helium effect (theory)



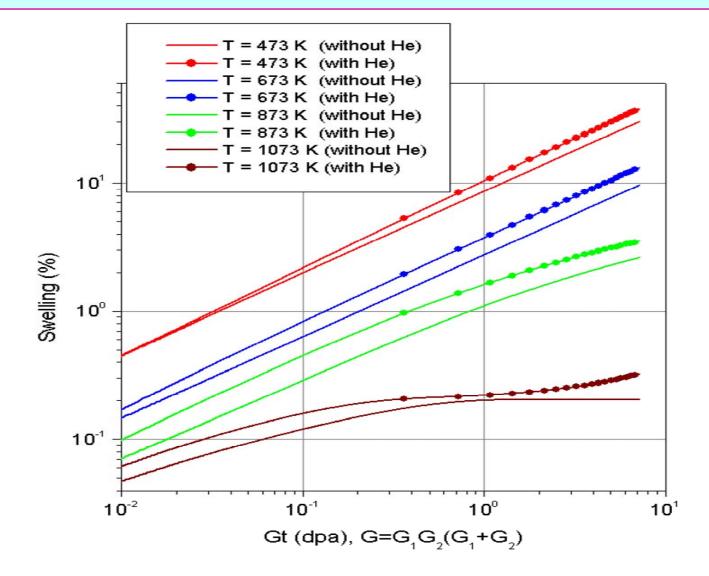
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Dose dependence of radiation swelling in SiC under ion irradiatiation



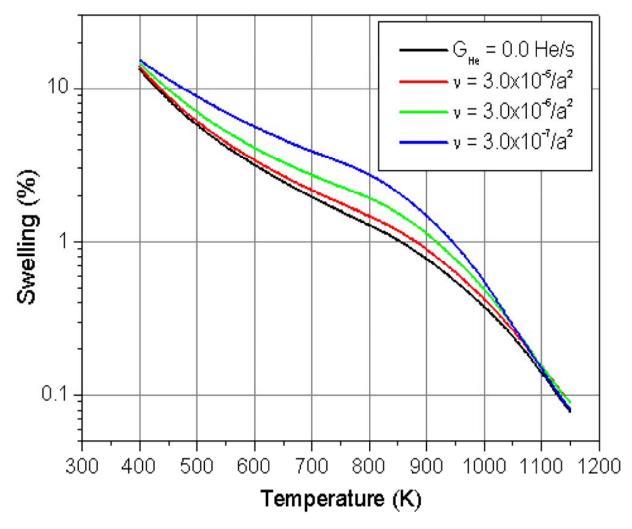
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Dose dependence of radiation swelling in SiC at different temperatures with the helium effect



20-24 April 2009, Trieste, Italy

Temperature dependence of radiation swelling in SiC at 0.7 dpa and at different coefficient of helium atom capture by vacancy



A.Ryazanov, A.Kohyama, Y.Katoh, 2004

20-24 April 2009, Trieste, Italy

- ★ Low temperature swelling in SiC due to point defect accumulation in matrix exceeding 1% at temperatures below 673K, which is supported both by experiment and theoretical model.
- ★ Low temperature swelling in SiC saturates in helium free conditions. Saturation behavior is highly predictable using the suggested theoretical model. Saturation of radiation swelling in SiC is determined based on the growth rate saturation of interstitial dislocation loops.
- ★ The recent experimental data clearly demonstrate the strong monotonic decrease of radiation swelling in SiC up to 1000C. The theoretical calculations give the same temperature dependence for decreasing of radiation swelling in SiC with temperature increase.

- Helium atoms increase the nucleation of interstitial dislocation loops and decrease an average radius of them. Helium atoms increase radiation swelling in temperature interval from 400 to 1000 K.
- The recent experimental data on dual-beam clearly demonstrate the strong temperature decrease of radiation swelling in SiC with helium up to 1000C.
- The theoretical calculations give the same temperature dependence for radiation swelling in SiC with temperature increase.